MAY 28 1992

Docket Nos. 50-424
and 50-425

Mr. W. G. Hairston, III
Executive Vice President -
Nuclear Operations
Georgia Power Company
P.O. Box 1295
Birmingham, Alabama 35201

Dear Mr. Hairston:

SUBJECT: ISSUANCE OF AMENDMENT - VOGTLE ELECTRIC GENERATING PLANT, UNITS 1 AND 2 (TACS M82130/M82131)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 51 to Facility Operating License No. NPF-68 and Amendment No. 30 to Facility Operating License No. NPF-81 for the Vogtle Electric Generating Plant, Units 1 and 2. These amendments consist of changes to the Technical Specifications (TS) in response to your application dated November 12, 1991, as supplemented April 21, 1992.

The amendments revise the minimum required thermal design flow (TDF) specified in the TS for Vogtle Units 1 and 2. Your application also included TS change pages which would have applied only before the completion of the third refueling outage for Vogte Unit 2. Since this refueling outage has now been completed, these proposed changes are no longer needed and are not included in these amendments. The omission of this part of your request is in accordance with our previous discussions with Mr. Jim Bailey of your company.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Darl S. Hood, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 51 to NPF-68
2. Amendment No. 30 to NPF-81
3. Safety Evaluation

cc w/enclosures:
See next page

LA:PDII-3	PE:PDII-3	PM:PDII-3	OGC	D:PDII-3
L.Berry	LRaghavan:cw	DHood		DMatthews
5/19/92	5/19/92	5/19/92	5/22/92	5/19/92

KNJ for KNJ for see clipped page

DOCUMENT NAME: C:/VOG82130.AMD

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

May 28, 1992

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Executive Vice President -
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P.O. Box 1295
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The amendments revise the minimum required thermal design flow (TDF) specified in the TS for Vogtle Units 1 and 2. Your application also included TS change pages which would have applied only before the completion of the third refueling outage for Vogte Unit 2. Since this refueling outage has now been completed, these proposed changes are no longer needed and are not included in these amendments. The omission of this part of your request is in accordance with our previous discussions with Mr. Jim Bailey of your company.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly <u>Federal Register</u> notice.

Sincerely,

Darl Hood

Darl S. Hood, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 51 to NPF-68
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3. Safety Evaluation

cc w/enclosures:
See next page

Mr. W. G. Hairston, III
Georgia Power Company

Vogtle Electric Generating Plant

cc:
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U. S. Nuclear Regulatory Commission
P. O. Box 572
Waynesboro, Georgia 30830

DATED: MAY 2 8 1992

AMENDMENT NO. 51 TO VOGTLE ELECTRIC PLANT, UNIT 1
AMENDMENT NO. 30 TO VOGTLE ELECTRIC PLANT, UNIT 2

DISTRIBUTION:

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D. Hood	14-B-25
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

VOGTLE ELECTRIC GENERATING PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 51
License No. NPF-68

1. The Nuclear Regulatory Commission (the Commission) has found that:

A. The application for amendment to the Vogtle Electric Generating Plant, Unit 1 (the facility) Facility Operating License No. NPF-68 filed by the Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated November 12, 1991, as supplemented April 21, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-68 is hereby amended to read as follows:

   Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 51 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. GPC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

DBMatthews

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
  Changes

Date of Issuance: **May 28, 1992**

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

VOGTLE ELECTRIC GENERATING PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 30
License No. NPF-81

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Vogtle Electric Generating Plant, Unit 2 (the facility) Facility Operating License No. NPF-81 filed by the Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated November 12, 1991, as supplemented April 21, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-81 is hereby amended to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 30 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. GPC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

DBMatthews

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: **May 28, 1992**

ATTACHMENT TO LICENSE AMENDMENT NO. 51

FACILITY OPERATING LICENSE NO. NPF-68

DOCKET NO. 50-424

AND

TO LICENSE AMENDMENT NO. 30

FACILITY OPERATING LICENSE NO. NPF-81

DOCKET NO. 50-425

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove Pages	Insert Pages
2-5	2-5
3/4 2-13	3/4 2-13
B 3/4 2-5	B 3/4 2-5

TABLE 2.2-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

FUNCTIONAL UNIT	TOTAL ALLOWANCE (TA)	Z	SENSOR ERROR (S)	TRIP SETPOINT	ALLOWABLE VALUE
9. Pressurizer Pressure-Low (PI-0455A,B&C, PI-0456 & PI-0456A, PI-0457 & PI-0457A, PI-0458 & PI-0458A)	3.1	0.71	1.67	≥1960 psig**	≥1950 psig
10. Pressurizer Pressure-High (PI-0455A,B&C, PI-0456 & PI-0456A, PI-0457 & PI-0457A, PI-0458 & PI-0458A)	3.1	0.71	1.67	≤2385 psig	≤2395 psig
11. Pressurizer Water Level-High (LI-0459A, LI-0460A, LI-0461)	8.0	2.18	1.67	≤92% of instrument span	≤93.9% of instrument span
12. Reactor Coolant Flow-Low (LOOP1 LOOP2 LOOP3 LOOP4 FI-0414 FI-0424 FI-0434 FI-0444 FI-0415 FI-0425 FI-0435 FI-0445 FI-0416 FI-0426 FI-0436 FI-0446)	2.5	1.87	0.60	≥90% of loop design flow*	≥89.4% of loop design flow*
13. Steam Generator Water Level Low-Low (LOOP1 LOOP2 LOOP3 LOOP4 LI-0517 LI-0527 LI-0537 LI-0547 LI-0518 LI-0528 LI-0538 LI-0548 LI-0519 LI-0529 LI-0539 LI-0549 LI-0551 LI-0552 LI-0553 LI-0554)	18.5 (21.8)***	17.18 (18.21)***	1.67	≥18.5% (37.8)*** of narrow range instrument span	≥17.8% (35.9)*** of narrow range instrument span
14. Undervoltage - Reactor Coolant Pumps	6.0	0.58	0	≥9600 volts (70% bus voltage)	≥9481 volts (69% bus voltage)
15. Underfrequency - Reactor Coolant Pumps	3.3	0.50	0	≥57.3 Hz	≥57.1 Hz

*Loop design flow = 93,600 gpm

**Time constants utilized in the lead-lag controller for Pressurizer Pressure-Low are 10 seconds for lead and 1 second for lag. CHANNEL CALIBRATION shall ensure that these time constants are adjusted to these values.

***The value stated inside the parenthesis is for instrument that has the lower tap at elevation 333"; the value stated outside the parenthesis is for instrumentation that has the lower tap at elevation 438".

3/4.2.5 DNB PARAMETERS

LIMITING CONDITION FOR OPERATION

3.2.5 The following DNB-related parameters shall be maintained within the limits:

a. Reactor Coolant System T_{avg} (TI-0412, TI-0422, TI-0432, TI-0442), $\leq$ 592.5°F

b. Pressurizer Pressure (PI-0455A,B&C, PI-0456 & PI-0456A, PI-0457 & PI-0457A, PI-0458 & PI-0458A), $\geq$ 2199 psig*

c. Reactor Coolant System Flow (FI-0414, FI-0415, FI-0416, FI-0424, FI-0425, FI-0426, FI-0434, FI-0435, FI-0436, FI-0444, FI-0445, FI-0446) $\geq$ 384,509 gpm**

APPLICABILITY: MODE 1.

ACTION:

With any of the above parameters exceeding its limit, restore the parameter to within its limit within 2 hours or reduce THERMAL POWER to less than 5% of RATED THERMAL POWER within the next 4 hours.

SURVEILLANCE REQUIREMENTS

4.2.5.1 Reactor Coolant System T_{avg} and Pressurizer Pressure shall be verified to be within their limits at least once per 12 hours. RCS flow rate shall be monitored for degradation at least once per 12 hours. In the event of flow degradation, RCS flow rate shall be determined by precision heat balance within 7 days of detection of flow degradation.

4.2.5.2 The RCS flow rate indicators shall be subjected to CHANNEL CALIBRATION at each fuel loading and at least once per 18 months.

4.2.5.3 After each fuel loading, the RCS flow rate shall be determined by precision heat balance within 7 days after exceeding 90% RATED THERMAL POWER. The RCS flow rate shall also be determined by precision heat balance at least once per 18 months. Within 7 days prior to performing the precision heat balance flow measurement, the instrumentation used for performing the precision heat balance shall be calibrated. The provisions of 4.0.4 are not applicable for performing the precision heat balance flow measurement.

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*Limit not applicable during either a THERMAL POWER ramp in excess of 5% of RATED THERMAL POWER per minute or a THERMAL POWER step in excess of 10% of RATED THERMAL POWER.

**Includes a 2.7% flow measurement uncertainty.

3/4.2.5 DNB PARAMETERS (Continued)

The measurement uncertainty for the RCS total flow is based upon performing a precision heat balance flow measurement above 90% RATED THERMAL POWER and using the results to correlate the flow indication channels with the measured flow. If a precision heat balance flow measurement is performed below 90% RATED THERMAL POWER, the effect on the measurement uncertainty shall be taken into account. Potential fouling of the feedwater venturis which might not be detected could bias the results from the precision heat balance in a non-conservative manner. Therefore, a penalty of 0.1% for undetected feedwater venturi fouling is included in the measurement uncertainty. Any fouling which might bias the RCS flow rate measurement greater than 0.1% may be detected by monitoring and trending various plant performance parameters. If detected, action shall be taken before performing subsequent precision heat balance flow measurements, i.e., either the effect of the fouling shall be quantified and accounted for in the RCS flow rate measurement or the affected venturis shall be cleaned to eliminate the fouling. The indicated RCS flow value of 384,509 gpm corresponds to an analytical value of 374,400 gpm with allowance for measurement and indication uncertainties.

The 12-hour periodic surveillance of these parameters through instrument readout is sufficient to ensure that the parameters are restored within their limits following load changes and other expected transient operation. The 18 month periodic measurement of the RCS total flow rate is adequate to detect flow degradation and ensure correlation of the flow indication channels with measured flow such that the indicated percent flow will provide sufficient verification of the flow rate degradation on a 12 hour basis. A change in indicated percent flow which is greater than the instrument channel inaccuracies and parallax errors is an appropriate indication of RCS flow degradation.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 51 TO FACILITY OPERATING LICENSE NPF-68

AND AMENDMENT NO. 30 TO FACILITY OPERATING LICENSE NPF-81

GEORGIA POWER COMPANY, ET AL.

VOGTLE ELECTRIC GENERATING PLANT, UNITS 1 AND 2

DOCKET NOS. 50-424 AND 50-425

1.0 INTRODUCTION

By letter dated November 12, 1991, as supplemented April 21, 1992, Georgia Power Company, et al. (the licensee) proposed license amendments to change the minimum required thermal design flow (TDF) specified in the Technical Specifications (TSs) for Vogtle Electric Generating Plant (Vogtle or the facility), Units 1 and 2. Specifically, the footnote in TS Table 2.2.1 for "Loop Design Flow" would be changed to reduce the specified flow from 95,700 gpm to 93,600 gpm. Similarly, in TS 3.2.5.c, the "Reactor Coolant System (RCS) Flow" specified in the limiting condition for operation (LCO) and associated TS Bases 3/4.2.5 would be revised from 393,136 gpm to 384,509 gpm (including flow uncertainty). The licensee's application also included related changes which would apply only prior to completion of the third refueling outage for Vogtle Unit 2. However, since that refueling outage has now been completed, these proposed changes are no longer needed and are not included in these amendments. The April 21, 1992 letter provided additional information which did not change the initial proposed no significant hazards consideration determination.

2.0 BACKGROUND

During the third refueling outage for Vogtle Unit 1 in late 1991, the licensee removed the resistance temperature detector (RTD) bypass system used to measure the hot leg temperature and replaced it with direct immersion RTDs. During this outage, the licensee also began a transition in fuel type by replacing one third of Unit 1's core with Westinghouse's VANTAGE-5 fuel using a low leakage fuel loading pattern. The same changes were made to Unit 2 during its third refueling outage which was recently completed. These changes were accomplished in accordance with Amendments 43 through 46 for Unit 1, and Amendments 23 through 25 for Unit 2. The low leakage fuel loading pattern has resulted in increased hot leg streaming which causes an erroneous reduction in the RCS flow rate measured via the calorimetric heat balance. To compensate for this problem and ensure that the RCS flow rate TS limit can be met, the licensee has proposed the above TS changes which reduce the allowable loop and RCS flow rate.

3.0 EVALUATION

In support of these proposed TS amendments, the licensee and Westinghouse have examined each safety analysis that uses TDF as an input parameter. Each analysis was either reanalyzed with a reduced RCS TDF of 374,400 gpm to determine the effect of the flow reduction, or evaluated to determine that the impact of the flow reduction was insignificant. These analyses and evaluations assumed a power level of 3565 MWt, which provides results that are conservative with respect to the power level of 3411 MWt authorized by the current operating licenses.

As noted in Section 2 of this evaluation, the licensee had previously submitted a program to NRC for implementation of VANTAGE-5 fuel which has been approved by the NRC. The analyses in that program were for a reduced loop TDF of 93,600 gpm (374,400 gpm for four loops) and a power level of 3565 MWt. This included accident analyses for large and small break LOCAs, steam generator tube rupture, and a large spectrum of non-LOCA events dependant on fuel-related parameters.

Also, as part of an earlier program to relocate the lower steam generator instrument taps that are used to determine narrow-range level, the licensee had previously analyzed several non-LOCA events that are dependent on steam generator level based upon the reduced TDF. These analyses were submitted in the licensee's application of May 29, 1990, and were approved by the NRC upon issuance of Amendments 34 (Unit 1) and 14 (Unit 2) on August 30, 1990. These analyses included:

FSAR Section 15.2.6	Loss of Nonemergency AC Power to Plant Auxiliaries
FSAR Section 15.2.7	Loss of Normal Feedwater Flow
FSAR Section 15.2.8	Feedwater System Pipe Break

In its application of November 12, 1991, the licensee identified four additional events or evaluations which had not been addressed with the lower TDF in previously approved steam generator level tap relocation and VANTAGE-5 programs. The revised evaluations are based upon the lower TDF. The four issues are:

1) Inadvertent Opening of a Steam Generator Relief or Safety Valve Event (FSAR Section 15.1.4) - To address this issue, the licensee referenced a reanalysis of the event assuming the lower TDF included as part of a power uprate submittal dated February 28, 1992. Additional details of the reanalysis were provided in the licensee's letter of April 21, 1992. The analysis was performed using approved Westinghouse computer codes LOFTRAN and THINC-IV, and the approved Westinghouse W-3 departure from nucleate boiling (DNB) correlation (The W-3 correlation is based upon a minimum DNB ratio (DNBR) limit of 1.30). The licensee indicated that the reanalysis accounted for all DNBR penalties (e.g., mixed core penalty) and was performed for both types of fuel presently in the Vogtle cores (Westinghouse's 17 x 17 low parasitic and VANTAGE-5). The licensee reported that the minimum DNBR remained above the 1.30 limit.

The licensee's reanalysis used approved methodologies and appropriate assumptions, and provided acceptable results. The staff finds the analysis acceptable.

2) Main Steamline Break Event (FSAR Section 15.1.5) - To address this issue the licensee referenced a reanalysis of the event assuming the lower TDF included in the February 28, 1992 power uprate submittal, as supplemented by information in the April 21, 1992, letter. The methodologies used in this reanalysis are the same as used for Item (1) above, except that a DNBR limit criterion of 1.45 was applied to account for the calculation of primary system pressure to drop below 1000 psia. The licensee reported that the minimum DNBR for this event, considering DNBR penalties, remained above the 1.45 criterion.

   The licensee's reanalysis used approved methodologies and appropriate assumptions, and provided acceptable results. The staff finds the analysis acceptable.

3) Main Steamline Break Information Used for Superheat Study for Vogtle Units 1 and 2 (WCAP-11285) - This issue relates to the environmental qualification envelope for equipment located outside containment. The licensee's studies of this issue were originally performed for the currently licensed power level and were subsequently updated in the licensee's power uprate submittal. The licensee finds that TDF has a negligible impact on the environmental consequences to equipment located outside containment. The staff agrees with the licensee's conclusion and finds the lower TDF acceptable with respect to the environmental qualification of equipment, based upon the current authorized power level. The staff has not reviewed the licensee's conclusion with respect to proposed power level increases.

4) Containment Design Evaluation (FSAR Section 6.2.1.1 and 6.2.1.4) - The licensee addressed this item involving Steamline Break and Loss of Coolant Accident containment conditions by referencing containment analyses reported in the February 28, 1992, power uprate submittal. The referenced analyses were performed by Westinghouse using its COCO computer code. COCO is a containment analysis code that has been previously approved by the NRC.

   The licensee's analyses used approved methodologies and appropriate assumptions, and provided acceptable results. The staff finds the analyses acceptable.

The licensee has examined the effect of the reduction in allowable RCS flow on trip setpoints and has determined that no changes are required to the current settings.

Based on the information presented above, the licensee has concluded, and the staff agrees, that reduction in TDF and the LCO RCS flow value do not involve a significant increase in the probability or consequences of an accident previously evaluated. Moreover, the proposed lower TDF value does not cause any acceptance criteria for safety analyses, or the environmental envelopment for equipment qualification, to be exceeded. The staff therefore finds the proposed change to be acceptable for operation at the current authorized power level of 3411 MWt.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Georgia State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (56 FR 61263 dated December 2, 1991). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: D. Hood, PDII-3/DPRE
H. Balukjian, SRXB/DST

Dated: **May 28, 1992**