

August 21, 2001

Mr. William T. Cottle
President and Chief Executive Officer
STP Nuclear Operating Company
South Texas Project Electric
Generating Station
P. O. Box 289
Wadsworth, TX 77483

SUBJECT: SOUTH TEXAS PROJECT, UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS
ON ELIMINATION OF RESPONSE TIME TESTING (TAC NOS. MB1412 AND
MB1420)

Dear Mr. Cottle:

The Commission has issued the enclosed Amendment No. 130 to Facility Operating License No. NPF-76 and Amendment No. 119 to Facility Operating License No. NPF-80 for the South Texas Project, Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated February 28, 2001.

The amendments revise the TS to eliminate periodic response time testing requirements on selected sensors and selected protection channels, and modify TS Section 1.0 Definitions for "ENGINEERED SAFETY FEATURE RESPONSE TIME" and "REACTOR TRIP SYSTEM RESPONSE TIME" to provide for verification of response time for selected components. TS Surveillances 4.3.1.2 and 4.3.2.2 are modified consistent with the new definitions. The associated TS Bases are also revised.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

/RA/

David J. Wrona, Project Manager, Section 1
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-498 and 50-499

Enclosures: 1. Amendment No. 130 to NPF-76
2. Amendment No. 119 to NPF-80
3. Safety Evaluation

cc w/encls: See next page

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ACCESSION NO: ML

*no substantive change from SE input

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South Texas, Units 1 & 2

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June 2001

STP NUCLEAR OPERATING COMPANY

DOCKET NO. 50-498

SOUTH TEXAS PROJECT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 130
License No. NPF-76

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by STP Nuclear Operating Company* acting on behalf of itself and for Houston Lighting & Power Company (HL&P), the City Public Service Board of San Antonio (CPS), Central Power and Light Company (CPL), and the City of Austin, Texas (COA) (the licensees), dated February 28, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

*STP Nuclear Operating Company is authorized to act for Houston Lighting & Power Company (HL&P), the City Public Service Board of San Antonio, Central Power and Light Company, and the City of Austin, Texas, and has exclusive responsibility and control over the physical construction, operation, and maintenance of the facility.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-76 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 130 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Robert A. Gramm, Chief, Section 1
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: August 21, 2001

STP NUCLEAR OPERATING COMPANY

DOCKET NO. 50-499

SOUTH TEXAS PROJECT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 119
License No. NPF-80

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by STP Nuclear Operating Company* acting on behalf of itself and for Houston Lighting & Power Company (HL&P), the City Public Service Board of San Antonio (CPS), Central Power and Light Company (CPL), and the City of Austin, Texas (COA) (the licensees), dated February 28, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

*STP Nuclear Operating Company is authorized to act for Houston Lighting & Power Company (HL&P), the City Public Service Board of San Antonio, Central Power and Light Company, and the City of Austin, Texas, and has exclusive responsibility and control over the physical construction, operation, and maintenance of the facility.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-80 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 119 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Robert A. Gramm, Chief, Section 1
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: August 21, 2001

ATTACHMENT TO LICENSE AMENDMENT NOS. 130 AND 119

FACILITY OPERATING LICENSE NOS. NPF-76 AND NPF-80

DOCKET NOS. 50-498 AND 50-499

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

1-3
1-5
3/4 3-1
3/4 3-17
B 3/4 3-1
B 3/4 3-2

INSERT

1-3
1-5
3/4 3-1
3/4 3-17
B 3/4 3-1
B 3/4 3-2

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 130 AND 119 TO

FACILITY OPERATING LICENSE NOS. NPF-76 AND NPF-80

STP NUCLEAR OPERATING COMPANY, ET AL.

SOUTH TEXAS PROJECT, UNITS 1 AND 2

DOCKET NOS. 50-498 AND 50-499

1.0 INTRODUCTION

By application dated February 28, 2001, South Texas Project Nuclear Operating Company (the licensee) proposed a license amendment to change the Technical Specifications (TS) for South Texas Project (STP) Units 1 and 2. The proposed amendments reflect changes necessary to modify the surveillances of pressure sensor response time and periodic protection channel response time for the Reactor Trip System (RTS) and the Engineered Safety Features Actuation System (ESFAS) to permit verification by means other than testing. This safety evaluation describes the NRC staff's bases for approving the requested license amendments.

2.0 BACKGROUND

The Westinghouse Owner's Group (WOG) performed three analyses to assess the impact of elimination of response time testing (RTT) for instruments and instrument loops. These analyses also discussed alternate test methodologies that would confirm instrumentation was functioning correctly. The first of these analyses was documented in Westinghouse Owners Group Licensing Topical Report WCAP-13632, "Elimination of Pressure Sensor Response Time Testing Requirements," dated August 1995, which was approved by a safety evaluation report (SER) dated September 5, 1995. The second analysis, WCAP-14036-P Revision 1, "Elimination of Periodic Protection Channel Response Time Tests," dated December 1995, was approved in an SER dated October 6, 1998. The third analysis, WCAP-15413, "Westinghouse 7300A ASIC-Based Replacement Module Licensing Summary Report," Revision 0, dated June 21, 2000, was approved in an SER dated February 8, 2001. Each of these SERs stipulates certain conditions that individual plant licensees must meet when implementing the guidelines in WCAP-13632, WCAP-14036, and WCAP-15413 on a plant-specific basis. These stipulations form the basis for approval of the licensee's requested license amendments, as discussed in the following section.

3.0 EVALUATION

The licensee proposes to eliminate periodic pressure sensor RTT in accordance with WCAP-13632-P-A, Revision 2, and to eliminate periodic protection channel RTT in accordance with WCAP-14036-P-A, Revision 1. The proposed license amendments modify TS

Definitions 1.13, "Engineered Safety Features Response Time," and 1.28, "Reactor Trip System Response Time," Surveillance Requirement (SR) 4.3.1.2 and SR 4.3.2.2 of TS 3/4.3.1 - "Reactor Trip System Instrumentation," and TS 3/4.3.2.2 - "Engineered Safety Feature Actuation System Instrumentation," and TS Bases B 3/4.3.1 and B 3/4.3.2 - "Reactor Trip System and Engineered Safety Features Actuation System Instrumentation," to indicate that the response time for the pressure sensors, process racks and trip logic will be determined based on the analysis and testing presented in WCAP-13632-P-A, Revision 2, and WCAP-14036-P-A, Revision 1, for these systems.

WCAP-13632-P-A, Revision 2, contains the technical basis and methodology for eliminating RTT requirements on selected pressure sensing instruments. When submitting a plant-specific license amendment request, the licensee must confirm the applicability of the generic analysis to its plant and must commit to the following actions:

- (a) Perform a hydraulic RTT prior to installation of a new transmitter/switch or following refurbishment of the transmitter/switch (e.g., sensor cell or variable damping components) to determine an initial sensor-specific response time value.

Consistent with the proposed changes to TS 3.3.1 and TS 3.3.2 (including the associated Bases) and EPRI Report NP-7243, Revision 1,(2) "Investigation of Response Time Testing Requirements," the licensee states that applicable plant procedures will include requirements that pressure sensor response times be verified by performance of an appropriate response time test prior to placing a sensor into operational service and re-verified following maintenance that may adversely affect sensor response time. These actions are consistent with the WCAP-13632-P-A, Revision 2, technical basis and methodology for eliminating RTT requirements on selected pressure sensing instruments and, therefore, are acceptable.

- (b) For transmitters and switches that use capillary tubes, perform a RTT after initial installation and after any maintenance or modification activity that could damage the capillary tubes.

The licensee states that plant procedure revisions (and/or other appropriate administrative controls) will stipulate that pressure sensors (transmitters and switches) utilizing capillary tubes, e.g., containment pressure, must be subjected to RTT after initial installation and following any maintenance or modification activity which could damage the transmitter capillary tubes. When sensor RTT is required, the resultant pressure sensor response times will be documented in the plant procedure data packages. These actions are consistent with the WCAP-13632-P-A, Revision 2, technical basis and methodology for response time testing pressure sensing instruments with capillary tubes and, therefore, are acceptable.

- (c) If variable damping is used, implement a method to assure that the potentiometer is at the required setting and cannot be inadvertently changed, or perform hydraulic RTT of the sensor following each calibration.

The licensee states that there are no pressure transmitters with variable damping in any RTS or ESFAS application for which RTT is required. Therefore, the licensee concludes, no STP procedure changes or enhanced administrative controls are necessary. Further, the licensee states that, should any of these transmitters be replaced with transmitters having variable damping capability, the licensee will perform hydraulic RTT of the sensor following each calibration, or will implement plant procedures (and/or other administrative controls) to assure the variable damping potentiometer cannot be inadvertently changed. Examples of such administrative controls may include use of pressure transmitters that are factory set and hermetically sealed to prohibit tampering or in-situ application of a tamper seal (or sealant) on the potentiometer to secure and give a visual indication of the potentiometer position. These actions are consistent with the WCAP-13632-P-A, Revision 2, methodology for assuring that variable damping potentiometers cannot be inadvertently changed and, therefore, are acceptable.

- (d) Perform periodic drift monitoring of all Model 1151, 1152, 1153, and 1154 Rosemount pressure and differential pressure transmitters, for which RTT elimination is proposed, in accordance with the guidance contained in Rosemount Technical Bulletin No. 4 and continue to remain in full compliance with any prior commitments to Bulletin 90-01, Supplement 1. As an alternative to performing periodic drift monitoring of Rosemount transmitters, licensees may complete the following actions: (1) ensure that operators and technicians are aware of the Rosemount transmitter loss of fill-oil issue and make provisions to ensure that technicians monitor for sensor response time degradation during the performance of calibrations and functional tests of these transmitters, and (2) review and revise surveillance testing procedures, if necessary, to assure that calibrations are being performed using equipment designed to provide a step function or fast ramp in the process variable and that calibrations and functional tests are being performed in a manner that allows simultaneous monitoring of both the input and output response of the transmitter under test, thus allowing, with reasonable assurance, the recognition of significant response time degradation.

As stated in Rosemount Technical Bulletin No. 4, only Rosemount Transmitters manufactured prior to July 1989 are suspect for potential loss of fill-oil. The licensee states that STP has no Rosemount transmitters that were manufactured prior to July 1989, in any RTS or ESFAS application, and therefore no periodic drift monitoring of Rosemount transmitters, for which response time testing elimination is proposed, is required. The Rosemount transmitters currently used in RTS and ESFAS applications were manufactured no earlier than year 1999, and were not installed until year 2000.

WCAP-14036-P-A and WCAP-15413-A contain the technical basis and methodology for eliminating periodic RTT requirements on RTS and ESFAS functions. The NRC safety evaluations approving WCAP-14036 and WCAP-15413 require that, when submitting a plant-specific license amendment request, the licensee must verify that the failure modes and effects analysis (FMEA) performed by the WOG is applicable to the equipment actually installed in the licensee facility, and that the analysis is valid for the versions of the boards used in the protection system.

The licensee verified that the FMEA presented in WCAP-14036-P-A and WCAP-15413-A is applicable to and valid for the equipment actually installed at STP. Allocations for system response times were obtained from the bounding criteria in WCAP-14036 and WCAP-15413 or plant-specific configurations.

The NRC staff reviewed the functions and response time allocations identified by the licensee in the license amendments submittal. The sensors identified for RTT elimination are part of the set of sensors identified in WCAP-13632-P-A, WCAP-14036-P-A, and WCAP-15413-A and, therefore, are acceptable for RTT elimination. The functions identified by the licensee for elimination of RTT are acceptable on the basis of the response time allocations for the 7300 nuclear instrumentation system string plus solid-state protection system (SSPS) input relays, the SSPS logic, and the ASIC-based replacement modules.

The NRC staff reviewed the proposed revisions of TS Definitions 1.13 and 1.28, and Surveillance Requirements 4.3.1.2 and 4.3.2.2, which incorporate the methodology approved in WCAP-13632-P-A, Revision 2, WCAP-14036-P-A, Revision 1, and WCAP-15413-A. Specifically, the changes revise the TS Definitions to permit verification as opposed to measurement of response time, and replace the words "demonstrated," "testing," and "tested" in the Surveillance Requirements with the words "verified" and "verification." A discussion of the proposed changes was added to TS Bases Sections B 3/4.3.1 and B 3/4.3.2. These changes are applicable to selected components provided both the components and the methodology for verification meet the criteria reviewed and approved by the NRC. The specific sections of the STP Units 1 and 2 TS to be changed are as follows:

- a. Section 1.0, Definitions, ENGINEERED SAFETY FEATURE (ESF) RESPONSE TIME, page 1-3.

Proposed Change: Change the definition to incorporate verification of response time in lieu of measurement. The definition currently states:

1.13 The ENGINEERED SAFETY FEATURES (ESF) RESPONSE TIME shall be that time interval from when the monitored parameter exceeds its ESF Actuation Setpoint at the channel sensor until the ESF equipment is capable of performing its safety function (i.e., the valves travel to their required positions, pump discharge pressures reach their required values, etc.). Times shall include diesel generator starting and sequence loading delays where applicable.

With addition of the proposed sentences, the revised definition states:

1.13 The ENGINEERED SAFETY FEATURES (ESF) RESPONSE TIME shall be that time interval from when the monitored parameter exceeds its ESF Actuation Setpoint at the channel sensor until the ESF equipment is capable of performing its safety function (i.e., the valves travel to their required positions, pump discharge pressures reach their required values, etc.). Times shall include diesel generator starting and sequence loading delays where applicable. The response time may be measured by means of any series of sequential, overlapping, or total steps so that the entire response time is measured. In lieu of measurement, response time may be verified for selected components provided that the components and methodology for verification have been previously reviewed and approved by the NRC.

This change is consistent with the WCAP-14036-P-A, Revision 1, approved TS definition 1.13 and, therefore, is acceptable.

- b. Section 1.0, Definitions, REACTOR TRIP SYSTEM RESPONSE TIME, page 1-5.

Proposed Change: Change the definition to incorporate verification of response time in lieu of measurement. The definition currently states:

1.28 The REACTOR TRIP SYSTEM RESPONSE TIME shall be the time interval from when the monitored parameter exceeds its Trip Setpoint at the channel sensor until loss of stationary gripper coil voltage.

The revised definition states:

1.28 The REACTOR TRIP SYSTEM RESPONSE TIME shall be the time interval from when the monitored parameter exceeds its Trip Setpoint at the channel sensor until loss of stationary gripper coil voltage. The response time may be measured by means of any series of sequential, overlapping, or total steps so that the entire response time is measured. In lieu of measurement, response time may be verified for selected components provided that the components and methodology for verification have been previously reviewed and approved by the NRC.

This change is consistent with the WCAP-14036, Revision 1, approved TS definition 1.28 and, therefore, is acceptable.

- c. Change TS Surveillance Requirement 4.3.1.2, page 3/4 3-1, which states:

4.3.1.2 The REACTOR TRIP SYSTEM RESPONSE TIME of each Reactor trip function shall be demonstrated to be within its limit at least once per 18 months. Each test shall include at least one train such that both trains are tested at least once per 36 months and one channel per function such that all channels are tested at least once every N times 18 months where N is the total number of

redundant channels in a specific Reactor trip function as shown in the "Total No. of Channels" column of Table 3.3-1.

The revised TS Surveillance Requirement 4.3.1.2 states:

4.3.1.2 The REACTOR TRIP SYSTEM RESPONSE TIME of each Reactor trip function shall be verified to be within its limit at least once per 18 months. Each verification shall include at least one train such that both trains are verified at least once per 36 months and one channel per function such that all channels are verified at least once every N times 18 months where N is the total number of redundant channels in a specific Reactor trip function as shown in the "Total No. of Channels" column of Table 3.3-1.

This change is consistent with the WCAP-14036, Revision 1, approved TS surveillance requirement 4.3.1.2 and, therefore, is acceptable.

- d. Change TS Surveillance Requirement 4.3.1.2, page 3/4 3-17, which states:

4.3.2.2 The ENGINEERED SAFETY FEATURES RESPONSE TIME of each ESFAS function shall be demonstrated to be within the limit at least once per 18 months. Each test shall include at least one train so that:

- a. Each logic train is tested at least once per 36 months,
- b. Each actuation train is tested at least once per 54 months*, and
- c. One channel per function so that all channels are tested at least once per N times 18 months where N is the total number of redundant channels in a specific ESFAS function as shown in the "Total No. of Channels" column of Table 3.3-3.

The footnote states:

*If an ESFAS instrumentation channel is inoperable due to response times exceeding the required limits, perform an engineering evaluation to determine if the test failure is a result of degradation of the actuation relays. If degradation of the actuation relays is determined to be the cause, increase the ENGINEERED SAFETY FEATURES RESPONSE TIME surveillance frequency such that all trains are tested at least once per 36 months.

The revised TS Surveillance Requirement 4.3.2.2 states:

4.3.2.2 The ENGINEERED SAFETY FEATURES RESPONSE TIME of each ESFAS function shall be verified to be within the limit at least once per 18 months. Each verification shall include at least one train so that:

- a. Each logic train is verified at least once per 36 months,
- b. Each actuation train is verified at least once per 54 months*, and
- c. One channel per function so that all channels are verified at least once per N times 18 months where N is the total number of redundant channels in a specific ESFAS function as shown in the "Total No. of Channels" column of Table 3.3-3.

The revised footnote states:

*If an ESFAS instrumentation channel is inoperable due to response times exceeding the required limits, perform an engineering evaluation to determine if the verification failure is a result of degradation of the actuation relays. If degradation of the actuation relays is determined to be the cause, increase the ENGINEERED SAFETY FEATURES RESPONSE TIME surveillance frequency such that all trains are verified at least once per 36 months.

These changes are consistent with WCAP-14036, Revision 1, approved TS surveillance requirement 4.3.2.2 and, therefore, are acceptable.

- e. Change TS Bases 3/4.3.1 and 3/4.3.2, Reactor Trip System and Engineered Safety Features Actuation System Instrumentation, page B 3/4.3-1, which, in part, states:

The measurement of response time at the specified frequencies provides assurance that the Reactor trip and the Engineered Safety Features actuation associated with each channel is completed within the time limit assumed in the safety analyses. No credit was taken in the analyses for those channels with response times indicated as not applicable. Response time may be demonstrated by any series of sequential, overlapping, or total channel test measurements provided that such tests demonstrate the total channel response time as defined. Sensor response time verification may be demonstrated by either: (1) in place, onsite, or offsite test measurements, or (2) utilizing replacement sensors with certified response times.

The revised TS Bases 3/4.3.1 and 3/4.3.2, Reactor Trip System and Engineered Safety Features Actuation System Instrumentation, pages B 3/4 3-1 and B 3/4 3-2, in part, state:

The measurement of response time at the specified frequencies provides assurance that the Reactor trip and the Engineered Safety Features actuation associated with each channel is completed within the time limit assumed in the safety analyses. No credit was taken in the analyses for those channels with response times indicated as not applicable.

Response time may be verified by actual response time tests in any series of sequential, overlapping or total channel measurements, or by the summation of allocated sensor, signal processing and actuation logic response times with

actual response time tests on the remainder of the channel. Allocations for sensor response times may be obtained from: (1) historical records based on acceptable response time tests (hydraulic, noise, or power interrupt tests), (2) in place, onsite, or offsite (e.g., vendor) test measurements, or (3) utilizing vendor engineering specifications. WCAP-13632-P-A, Revision 2, "Elimination of Pressure Sensor Response Time Testing Requirements" provides the basis and methodology for using allocated sensor response times in the overall verification of the channel response time for specific sensors identified in the WCAP.

Response time verification for other sensor types must be demonstrated by test. WCAP-14036P-A Revision 1, "Elimination of Periodic Protection Channel Response Time Tests" and WCAP-15413, "Westinghouse 7300A ASIC-Based Replacement Module Licensing Summary Report" provide the basis and methodology for using allocated signal processing and actuation logic response times in the overall verification of the protection system channel response time. The allocations for sensor, signal conditioning and actuation logic response times must be verified prior to placing the component in operational service and re-verified following maintenance that may adversely affect response time. In general, electrical repair work does not impact response time provided the parts used for repair are of the same type and value. Specific components identified in the WCAP may be replaced without verification testing. One example where response time could be affected is replacing the sensing assembly of a transmitter. WCAP-15413 provides bounding response times where 7300 cards have been replaced with ASIC cards.

These changes are consistent with WCAP-14036, Revision 1, approved TS Bases 3/4.3.1 and 3/4.3.2, Reactor Trip System and Engineered Safety Features Actuation System Instrumentation, and, therefore, are acceptable.

On the basis of the above review and justifications for TS changes, the NRC staff concludes that the licensee has implemented the provisions of the generic safety evaluation report for RTT elimination and satisfied the applicable plant specific conditions in accordance with the approved topical reports WCAP-13632, WCAP-14036 and WCAP-15413. Therefore, the NRC staff concludes that the proposed TS modifications for selected instrument RTT elimination are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Texas State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding

that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (66 FR 31716, dated June 12, 2001). The amendments also relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

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