May 21, 1991

Docket Nos. 50-424
and 50-425

Mr. W. G. Hairston, III
Senior Vice President
Nuclear Operations
Georgia Power Company
P. O. Box 1295
Birmingham, Alabama 35201

Dear Mr. Hairston:

SUBJECT: VOGTLE UNITS 1 AND 2 - METHOD OF MEASURING REACTOR COOLANT SYSTEM DELTA TEMPERATURE (TAC NOS. 80082 AND 80083)

The Commission has forwarded the enclosed "Notice of Consideration of Issuance of Amendments to Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination, and Opportunity for Hearing" to the Office of the Federal Register for publication.

This notice relates to your amendment request dated November 29, 1990, as supplemented January 29 and March 6, 1991, and as revised March 29, 1991, to change the Vogtle Technical Specifications to reflect a planned modification to the method of measuring reactor coolant system delta temperature.

Sincerely,

/S/

Darl S. Hood, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc e/encl.:
See next page

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Mr. W. G. Hairston, III
Georgia Power Company

Vogtle Electric Generating Plant

cc:
Mr. J. A. Bailey
Manager - Licensing
Georgia Power Company
P. O. Box 1295
Birmingham, Alabama 35201

Mr. W. B. Shipman
General Manager, Vogtle Electric
Generating Plant
P. O. Box 1600
Waynesboro, Georgia 30830

Regional Administrator, Region II
U. S. Nuclear Regulatory Commission
101 Marietta Street, NW., Suite 2900
Atlanta, Georgia 30323

Office of the County Commissioner
Burke County Commission
Waynesboro, Georgia 30830

Office of Planning and Budget
Room 615B
270 Washington Street, SW.
Atlanta, Georgia 30334

Mr. C. K. McCoy
Vice President - Nuclear, Vogtle Project
Georgia Power Company
P. O. Box 1295
Birmingham, Alabama 35201

Resident Inspector
U. S. Nuclear Regulatory Commission
P. O. Box 572
Waynesboro, Georgia 30830

Mr. R. P. McDonald
Executive Vice President -
Nuclear Operations
Georgia Power Company
P. O. Box 1295
Birmingham, Alabama 35201

Mr. J. Leonard Ledbetter, Director
Environmental Protection Division
Department of Natural Resources
205 Butler Street, SE., Suite 1252
Atlanta, Georgia 30334

Attorney General
Law Department
132 Judicial Building
Atlanta, Georgia 30334

Mr. Alan R. Herdt
Project Branch #3
U. S. Nuclear Regulatory Commission
101 Marietta Street, NW., Suite 2900
Atlanta, Georgia 30323

Mr. Dan Smith
Program Director of
Power Production
Oglethorpe Power Corporation
100 Crescent Centre
Tucker, Georgia 30085

Charles A. Patrizia, Esq.
Paul, Hastings, Janofsky & Walker
12th Floor
1050 Connecticut Avenue, NW.
Washington, DC 20036

UNITED STATES NUCLEAR REGULATORY COMMISSION

GEORGIA POWER COMPANY, ET AL.

DOCKET NO. 50-424 AND 50-425

NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES, PROPOSED NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION, AND OPPORTUNITY FOR HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendments to Facility Operating Licenses Nos. NPF-68 and NPF-81 issued to Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, City of Dalton, Georgia (the licensee) for operation of the Vogtle Electric Generating Plant, Units 1 and 2, located in Burke County, Georgia.

The proposed amendments would change the Technical Specifications (TSs) to reflect a planned modification to the method of measuring reactor coolant system (RCS) delta temperature (DT). The current method uses resistance temperature detectors (RTDs) located in a bypass manifold to provide signals for the Overtemperature DT and Overpower DT reactor trip instrumentation. The modification would eliminate the bypass manifold and locate fast-response RTDs in thermowells directly in the hot and cold legs of the RCS loops to provide these signals.

Specifically, the definition of DT in Notes 1 (Overtemperature DT) and 3 (Overpower DT) of TS Table 2.2-1 would be revised to delete the phrase "by RTD Manifold Instrumentation." Additionally, footnote 12 of TS Table 4.3-1, which states that channel calibration of Overtemperature DT instrumentation shall include the RTD bypass loops flow rate, would be deleted. Because Vogtle Units 1 and 2 share a common TS document and the planned modifications are to

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occur at different times, the TS changes would be expressed both before and after the modification and annotated as needed for each Vogtle unit based upon modification status.

Other changes in the licensee's amendment requests have been previously noticed and are outside the scope of this notice.

Before issuance of the proposed license amendments, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the request for amendments involve no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendments would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration which is presented below:

> RTD bypass elimination does not significantly increase the probability of an accident previously evaluated. The integrity of the reactor coolant pressure boundary is maintained by design and installation procedures adhering to appropriate codes and standards. In addition, an instrumentation and control evaluation has concluded that the fast response RTD system remains in compliance with industry standards and criteria for single failure, independence, separation, and qualification considerations. No new accident initiators are created by this modification. Therefore, the modification has no effect on the probability of previously analyzed accidents involving the integrity of the reactor coolant pressure boundary or performance of the control and protection system.
>
> The consequences of an accident previously evaluated are not significantly increased due to RTD bypass manifold elimination. Although the pressure boundary will be modified, proper welding techniques and tests will ensure the integrity of the pressure boundary and thus it will not contribute to any additional radiological consequences. Protection and mitigation

systems will continue to operate as assumed in the safety analyses and will not result in any additional challenges to fuel integrity or changes in mass/energy releases.

RTD bypass does not create the possibility of a new accident or one different from any previously evaluated. The modification creates no new accident initiators and no new single failures have been identified. The installation operations minimize the potential for debris escaping into the RCS, and the small amount of debris introduced has been determined to be inconsequential. Additionally, the reliability and performance of the protection system remains consistent with that assumed in the safety analyses. Therefore, no new accidents have been created.

The margin of safety is not significantly reduced as a result of this modification. The resultant effects on the Overtemperature Delta-T and Overpower Delta-T reactor trip setpoints defined in TS Table 2.2-1 have been included in the safety analyses presented in reference 2 [reference 2 is the VANTAGE-5 Reload Transition Safety Report for Vogtle Electric Generating Plant forwarded by the licensee's letter of November 29, 1990]. The reason for footnote 12 in TS Table 4.3-1 has been eliminated and therefore the footnote is no longer appropriate. It has been confirmed that the changes identified for the VANTAGE-5 fuel program (reference 2) remain bounding for elimination of the bypass system and replacing it with the fast-response RTDs. Additionally, the integrity of the reactor coolant pressure boundary and protection system is maintained. Therefore, this change does not result in a significant reduction in a margin of safety.

The Commission's staff has reviewed the licensee's analysis, and based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the Commission's staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination. The Commission will not normally make a final determination unless it receives a request for a hearing.

Written comments may be submitted by mail to the Regulatory Publications Branch, Division of Freedom of Information and Publications Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and should cite the publication date and page number of this FEDERAL REGISTER

notice. Written comments may also be delivered to Room P-223, Phillips Building, 7920 Norfolk Avenue, Bethesda, Maryland, from 7:30 a.m. to 4:15 p.m. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555. The filing of requests for hearing and petitions for leave to intervene is discussed below.

By June 27, 1991 , the licensee may file a request for a hearing with respect to issuance of the amendments to the subject facility operating licenses and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written petition for leave to intervene. Request for a hearing and petitions for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555 and at the Local Public Document Room located at Burke County Public Library, 412 Fourth Street, Waynesboro, Georgia 30830. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature

of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendments under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a

supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the request for amendments involves no significant hazards consideration, the Commission may issue the amendments and make them effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendments.

If a final determination is that the amendments involves a significant hazards consideration, any hearing held would take place before the issuance of any amendments.

Normally, the Commission will not issue the amendments until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendments before the expiration of the 30-day notice period, provided that its final determination is that the amendments involve no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555, by the above date. Where petitions are filed during the last ten (10) days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 325-6000 (in Missouri 1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number 3737 and the following message addressed to David B. Matthews: (petitioner's name and telephone number), (date petition was mailed), (plant name), and (publication date and page number of this FEDERAL REGISTER notice). A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and to Mr. Arthur H. Domby, Troutman, Sanders, Lockerman and Ashmore, Candler Building, Suite 1400, 127 Peachtree Street, NE., Atlanta, Georgia 30043, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendments dated November 29, 1990; as supplemented January 29 and March 6, 1991; and as revised March 29, 1991, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555 and at the Local Public Document Room located at

Burke County Public Library, 412 Fourth Street, Waynesboro, Georgia 30830.

Dated at Rockville, Maryland, this 21st day of May 1991.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert Martin

Robert E. Martin, Acting
Project Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation