

September 18, 1990

Docket No. 50-424
and 50-425

DISTRIBUTION
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Mr. W. G. Hairston, III
Senior Vice President -
Nuclear Operations
Georgia Power Company
P. O. Box 1295
Birmingham, Alabama 35201

Dear Mr. Hairston:

SUBJECT: ISSUANCE OF AMENDMENT NO. 35 TO FACILITY OPERATING LICENSE NPF-68
AND AMENDMENT NO. 15 TO FACILITY OPERATING LICENSE NPF-81 -
VOGTLE ELECTRIC GENERATING PLANT, UNITS 1 AND 2 (TACs 76133/76134)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 35 to Facility Operating License No. NPF-68 and Amendment No. 15 to Facility Operating License No. NPF-81 for the Vogtle Electric Generating Plant, Units 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated March 1, 1990.

The amendments revise administrative TS 6.7.4.a by removing the parenthetical reference to the Boron Recycle System from the description of systems included in a program of leakage inspection and testing.

A copy of the related Safety Evaluation supporting the amendments is also enclosed. Notice of issuance of the amendments will be included in the Commission's biweekly Federal Register notice.

Sincerely,

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Timothy A. Reed, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 35 to NPF-68
2. Amendment No. 15 to NPF-81
3. Safety Evaluation

cc w/enclosures:
See next page

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Document Name: AMEND 76133/76134

DATED: September 18, 1990

AMENDMENT NO. 35 TO FACILITY OPERATING LICENSE NPF-68 - Vogtle Electric
Generating Plant, Unit 1
AMENDMENT NO. 15 TO FACILITY OPERATING LICENSE NPF-81 - Vogtle Electric
Generating Plant, Unit 2

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Georgia Power Company

Vogtle Electric Generating Plant

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA
VOGTLE ELECTRIC GENERATING PLANT, UNIT 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 35
License No. NPF-68

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Vogtle Electric Generating Plant, Unit 1 (the facility) Facility Operating License No. NPF-68 filed by the Georgia Power Company acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensees) dated March 1, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-68 is hereby amended to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 35, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. GPC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specifications Changes

Date of Issuance: September 18, 1990



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA
VOGTLE ELECTRIC GENERATING PLANT, UNIT 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 15
License No. NPF-81

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Vogtle Electric Generating Plant, Unit 2 (the facility) Facility Operating License No. NPF-81 filed by the Georgia Power Company acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensees) dated March 1, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-81 is hereby amended to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 15, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. GPC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification Changes

Date of Issuance: September 18, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 35

FACILITY OPERATING LICENSE NO. NPF-68

AND LICENSE AMENDMENT NO. 15

FACILITY OPERATING LICENSE NO. NPF-81

DOCKET NOS: 50-424 AND 50-425

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Amended Page

6-15

Overleaf Page

6-16

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

- c. The change is documented, reviewed in accordance with Specification 6.7.2 and approved by the General Manager-Nuclear Plant or department head of the responsible department within 14 days of implementation.

6.7.4 The following programs shall be established, implemented, and maintained:

- a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the following:

- 1) Residual Heat Removal System
- 2) Containment Spray System (excluding NaOH Subsystem)
- 3) Safety Injection (excluding Boron Injection & Accumulators)
- 4) Chemical and Volume Control System (Letdown and Charging Pumps)
- 5) Post Accident Processing System
- 6) Gaseous Waste Processing System
- 7) Nuclear Sampling System (Pressurizer steam and liquid sample lines, Reactor Coolant sample lines, RHR sample lines, CVCS Demineralizer and Letdown Heat Exchanger sample lines only)

The program shall include the following:

- 1) Preventive maintenance and periodic visual inspection requirements, and
- 2) Leak test requirements for each system at refueling cycle intervals or less.

- b. In-Plant Radiation Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

- 1) Training of personnel,
- 2) Procedures for monitoring, and
- 3) Provisions for maintenance of sampling and analysis equipment.

- c. Secondary Water Chemistry

A program for monitoring of secondary water chemistry to inhibit steam generator tube degradation. This program shall include:

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

- 1) Identification of a sampling schedule for the critical variables and control points for these variables,
 - 2) Identification of the procedures used to measure the values of the critical variables,
 - 3) Identification of process sampling points,
 - 4) Procedures for the recording and management of data,
 - 5) Procedures defining corrective actions for all off-control point chemistry conditions, and
 - 6) A procedure identifying: (a) the authority responsible for the interpretation of the data, and (b) the sequence and timing of administrative events required to initiate corrective action.
- d. Post-Accident Sampling
- A program which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:
- 1) Training of personnel,
 - 2) Procedures for sampling and analysis, and
 - 3) Provisions for maintenance of sampling and analysis equipment.
- e. A program which will ensure the capability to monitor plant variables and systems operating status during and following an accident. This program shall include those instruments provided to indicate system operating status and furnish information regarding the release of radioactive materials (Category 2 and 3 instrumentation as defined in Regulatory Guide 1.97 Revision 2) and provide the following:
- 1) preventive maintenance and periodic surveillance of instrumentation.
 - 2) pre-planned operating procedures and back-up instrumentation to be used if one or more monitoring instruments become inoperable.
 - 3) administrative procedures for returning inoperable instruments to OPERABLE status as soon as practicable.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 35 TO FACILITY OPERATING LICENSE NPF-68
AND AMENDMENT NO. 15 TO FACILITY OPERATING LICENSE NPF-81

GEORGIA POWER COMPANY, ET AL.

DOCKET NOS. 50-424 AND 50-425

VOGTLE ELECTRIC GENERATING PLANT, UNITS 1 AND 2

1.0 INTRODUCTION

By letter dated March 1, 1990, Georgia Power Company (GPC), et al. (the licensee) requested a change to the Technical Specifications (TSs) for Vogtle Electric Generating Plant (VEGP), Units 1 and 2. The proposed change would revise administrative TS Section 6.7.4.a by removing the parenthetical reference to the Boron Recycle System (BRS) from the description of systems included in a program of leakage inspection and testing.

2.0 EVALUATION

Technical Specification 6.7.4 identifies programs required to be established, implemented, and maintained for VEGP. Specifically, TS 6.7.4.a describes an inspection and testing program to reduce radioactive leakage from those portions of fluid systems located outside containment during a transient or accident. The requirement to establish such a program stems from Item III.D.1.1 of the TMI-2 Action Plan Requirements (NUREG-0737).

By letter dated March 21, 1986, GPC originally committed to implement the leakage reduction program. By letter dated July 2, 1986, GPC provided justification for exclusion of certain fluid systems from the provisions of NUREG-0737, Item III.D.1.1. The NRC staff, in Supplement 3 to the VEGP Safety Evaluation Report (SER) (Section 11.5.3, pages 11-2 and 11-3) accepted GPC's justification for excluding the boric acid transfer system, the boron recycle system, and the filter backwash system from the program to reduce radioactive leakage from fluid systems located outside containment. Specifically, the Boron Recycle System is not used during an accident and is isolated by check valves and normally closed valves.

Therefore, removal of the reference to the Boron Recycle System from TS 6.7.4.a results in consistency between the VEGP TSs and the VEGP SER. Since this change was previously accepted by the NRC staff in SER Supplement 3, the current TS change is administrative in nature. The NRC staff concludes that the change to TS 6.7.4.a is acceptable.

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3.0 ENVIRONMENTAL CONSIDERATION

The amendments relate to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

4.0 CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register on April 18, 1990 (55 FR 14506), and consulted with the State of Georgia. No public comments were received, and the State of Georgia did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: T. A. Reed, PD II-3/DRP I/II

Dated: September 18, 1990