

August 8, 1989

Docket Nos. 50-424
50-425

Mr. W. G. Hairston, III
Senior Vice President -
Nuclear Operations
Georgia Power Company
P.O. Box 1295
Birmingham, Alabama 35201

Dear Mr. Hairston:

SUBJECT: ISSUANCE OF AMENDMENT NO. 21 TO FACILITY OPERATING LICENSE NPF-68
AND AMENDMENT NO. 2 TO FACILITY OPERATING LICENSE NPF-81 - VOGTLE
ELECTRIC GENERATING PLANT, UNITS 1 AND 2 (TACs 72898/72899)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 21 to Facility Operating License No. NPF-68 and Amendment No. 2 to Facility Operating License NPF-81 for the Vogtle Electric Generating Plant, Units 1 and 2. These amendments consist of changes to the Technical Specifications (TS) in response to your application dated April 5, 1989.

The amendments will modify the TS to delete footnotes that are no longer applicable.

A copy of the related safety evaluation supporting Amendment No. 21 to Facility Operating License NPF-68 and Amendment No. 2 to Facility Operating License NPF-81 is enclosed.

Notice of issuance of the amendments will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

151
Jon B. Hopkins, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 21 to NPF-68
2. Amendment No. 2 to NPF-81
3. Safety Evaluation

cc w/enclosures:
See next page

LA:PDII-3
MRood
7/21/89

PM:PDII-3
JHopkins:
7/24/89

PDII-3
DMatthews
8/8/89

8908160036 890808
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P PDC



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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50-425

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Senior Vice President -
Nuclear Operations
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P.O. Box 1295
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AND AMENDMENT NO. 2 TO FACILITY OPERATING LICENSE NPF-81 - VOGTLE
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Notice of issuance of the amendments will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, reading "Jon B. Hopkins", is written over the typed name.

Jon B. Hopkins, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 21 to NPF-68
2. Amendment No. 2 to NPF-81
3. Safety Evaluation

cc w/enclosures:
See next page

DATED: August 8, 1989

AMENDMENT NO. 21 TO FACILITY OPERATING LICENSE NPF-68-Vogtle Electric
Generating Plant, Unit 1
AMENDMENT NO. 2 TO FACILITY OPERATING LICENSE NPF-81-Vogtle Electric
Generating Plant, Unit 2

DISTRIBUTION:

Docket File

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MRood	14-H-25
JHopkins	14-H-25
OGC-WF	15-B-18
EJordan	MNBB-3302
TMeek (4)	P1-137
ACRS (10)	P-135
WJones	P-130A
JCalvo	11-F-23
GPA/PA	17-F-2
ARM/LFMB	AR-2015
DHagan	MNBB-3302
BGrimes	9-A-2

DF01
1/1

Mr. W. G. Hairston, III
Georgia Power Company

Vogtle Electric Generating Plant

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA
VOGTLE ELECTRIC GENERATING PLANT, UNIT 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 21
License No. NPF-68

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Vogtle Electric Generating Plant, Unit 1 (the facility) Facility Operating License No. NPF-68 filed by the Georgia Power Company acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensees) dated April 5, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-68 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 21, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. GPC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By:
David B. Matthews

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification Changes

Date of Issuance: August 8, 1989

OFFICIAL RECORD COPY

LA:PDII-3
MRood
7/21/89

PM:PDII-3
JHopkins:
7/24/89

OGC-WFO
S H Lewis
7/27/89

D:PDII-3
DBMatthews
8/8/89

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-68 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 21, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. GPC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification Changes

Date of Issuance: August 8, 1989



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA
VOGTLE ELECTRIC GENERATING PLANT, UNIT 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 2
License No. NPF-81

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Vogtle Electric Generating Plant, Unit 2 (the facility) Facility Operating License No. NPF-81 filed by the Georgia Power Company acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensees) dated April 5, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-81 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 2 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. GPC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By:
David B. Matthews

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification Changes

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JHopkins:
7/24/89

OGC-WF
S H Lewis
7/27/89

D:PDII-3
DBMatthews
8/8/89

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-81 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 2, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. GPC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification Changes

Date of Issuance: August 8, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 21

FACILITY OPERATING LICENSE NO. NPF-68

AND LICENSE AMENDMENT NO. 2

FACILITY OPERATING LICENSE NO. NPF-81

DOCKET NOS. 50-424 AND 50-425

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

Amended Page

3/4 3-46
3/4 3-48
3/4 3-58
3/4 4-10
3/4 7-4
3/4 7-17
3/4 9-1

Overleaf Page

3/4 3-45
3/4 3-47
3/4 3-57
3/4 4-9
3/4 7-3
3/4 7-18
3/4 9-2

INSTRUMENTATION

3/4.3.3 MONITORING INSTRUMENTATION

RADIATION MONITORING FOR PLANT OPERATIONS

LIMITING CONDITION FOR OPERATION

3.3.3.1 The radiation monitoring instrumentation channels for plant operations shown in Table 3.3-4 shall be OPERABLE with their Alarm/Trip Setpoints within the specified limits.

APPLICABILITY: As shown in Table 3.3-4.

ACTION:

- a. With a radiation monitoring channel Alarm/Trip Setpoint for plant operations exceeding the value shown in Table 3.3-4, adjust the Setpoint to within the limit within 4 hours or declare the channel inoperable.
- b. With one or more radiation monitoring channels for plant operations inoperable, take the ACTION shown in Table 3.3-4.
- c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.1 Each radiation monitoring instrumentation channel for plant operations shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL CALIBRATION and ANALOG CHANNEL OPERATIONAL TEST for the MODES and at the frequencies shown in Table 4.3-3.

TABLE 3.3-4

RADIATION MONITORING INSTRUMENTATION FOR PLANT OPERATIONS

<u>FUNCTIONAL UNIT</u>	<u>CHANNELS TO TRIP/ALARM</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ALARM/TRIP SETPOINT</u>	<u>ACTION</u>
1. Containment					
a. Containment Area (High Range) (RE-0005, RE-0006)	1	See Table 3.3-8	1, 2, 3, 4	100 R/hr	See Table 3.3-8
b. RCS Leakage Detection					
1) Gaseous Activity (RE-2562C)	1	1	1, 2, 3, 4	< 2 x back- ground	29
2) Particulate Activity (RE-2562A)	1	1	1, 2, 3, 4	< 2 x back- ground	29
2. Containment Ventilation	1	2 ^c	1, 2, 3, 4, 6 ^a	See Table 3.3-3	See Table 3.3-2
Area Low Range (RE-0002, RE-0003) Gaseous Activity (RE-2565C) Particulate Activity (RE-2565A) Iodine Activity (RE-2565B)					
3. Control Room Air Intake (RE-12116, RE-12117)	1	2	Either Unit in 1, 2, 3, 4 5 ^b , 6 ^b	See Table 3.3-3	See Table 3.3-2

VOGTELE UNITS - 1 & 2

3/4 3-46

Amendment No. 21 (Unit 1)
Amendment No. 2 (Unit 2)

TABLE 3.3-4 (Continued)

TABLE NOTATIONS

- a During movement of irradiated fuel or movement of loads over irradiated fuel within containment.
- b During movement of irradiated fuel or movement of loads over irradiated fuel.
- c RE-2565 is considered OPERABLE if the Particulate (RE-2565A) and Iodine (RE-2565B) Monitors are OPERABLE, or the noble gas monitor (RE-2565C) is OPERABLE.

ACTION STATEMENTS

- ACTION 29 - With the number of OPERABLE Channels less than the Minimum Channels OPERABLE requirement, satisfy the ACTION requirements of Specification 3.4.6.1.

TABLE 4.3-3
RADIATION MONITORING INSTRUMENTATION FOR PLANT
OPERATIONS SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>ANALOG CHANNEL OPERATIONAL TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
1. Containment				
a. Containment Area (High Range) (RE-0005, RE-0006)	See Specification 4.3.3.6		M	1, 2, 3, 4
b. RCS Leakage Detection				
1) Gaseous Activity (RE-2562C)	S	R	M	1, 2, 3, 4
2) Particulate Activity (RE-2562A)	S	R	M	1, 2, 3, 4
2. Containment Ventilation Area Low Range (RE-0002, RE-0003) Gaseous Activity (RE-2565C) Particulate Activity (RE-2565A) Iodine Activity (RE-2565B)	See Table 4.3-2			
3. Control Room Air Intake (RE-12116, RE-12117)	See Table 4.3-2			

TABLE 3.3-7 (Continued)

REMOTE SHUTDOWN SYSTEM MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>Function</u>	<u>READOUT¹</u> <u>LOCATION</u>	<u>CHANNELS</u> <u>AVAILABLE</u>	<u>MINIMUM</u> <u>CHANNELS</u> <u>OPERABLE</u>
10.	BAST Level	L	1 (PI-10115 ²)	1 ³
11.	CST Level	L	2 (Tank 1 LI-5100) (Tank 2 LI-5115)	2 ³
12.	Auxiliary Feedwater Flow	A, B	1/LOOP (LOOP1 FI-5152B, Panel A) (LOOP2 FI-5151B, Panel B) (LOOP3 FI-5153B, Panel B) (LOOP4 FI-5150B, Panel A)	1/LOOP
13.	Steam Generator Pressure	A, B	1/LOOP (LOOP1 PI-0514C, Panel A) (LOOP2 PI-0525B, Panel B) (LOOP3 PI-0535B, Panel B) (LOOP4 PI-0544C, Panel A)	1/LOOP

¹ A - Remote Shutdown Panel PSDA

B - Remote Shutdown Panel PSDB

L - Local Indication

² Graph will be provided to determine level from pressure reading³ Alternate local level indication may be established to fulfill the minimum channels OPERABLE.

INSTRUMENTATION

ACCIDENT MONITORING INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.6 The accident monitoring instrumentation channels shown in Table 3.3-8 shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTION: As shown in Table 3.3-8

SURVEILLANCE REQUIREMENTS

4.3.3.6 Each accident monitoring instrumentation channel shall be demonstrated OPERABLE:

- a. Every 31 days by performance of a CHANNEL CHECK, and
- b. Every 18 months by performance of a CHANNEL CALIBRATION.

REACTOR COOLANT SYSTEM

3/4.4.3 PRESSURIZER

LIMITING CONDITION FOR OPERATION

3.4.3 The pressurizer shall be OPERABLE with a water volume of less than or equal to 92% (1656 cubic feet), (LI-0459A, LI-0460A, LI-0461A) and at least two groups of pressurizer heaters each having a capacity of at least 150 kW.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

- a. With only one group of pressurizer heaters OPERABLE, restore at least two groups to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.
- b. With the pressurizer otherwise inoperable, be in at least HOT STANDBY with the Reactor Trip System breakers open within 6 hours and in HOT SHUTDOWN within the following 6 hours.

SURVEILLANCE REQUIREMENTS

4.4.3.1 The pressurizer water volume shall be determined to be within its limit at least once per 12 hours.

4.4.3.2 The capacity of each of the above required groups of pressurizer heaters shall be verified by energizing the heaters and measuring circuit current at least once per 92 days.

REACTOR COOLANT SYSTEM

3/4.4.4 RELIEF VALVES

LIMITING CONDITION FOR OPERATION

3.4.4 All power-operated relief valves (PORVs) and their associated block valves shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

- a. With one or more PORV(s) inoperable, because of excessive seat leakage, within 1 hour either restore the PORV(s) to OPERABLE status or close the associated block valve(s); otherwise, be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.
- b. With one or more PORV(s) inoperable due to causes other than excessive seat leakage, within 1 hour either restore the PORV(s) to OPERABLE status or close the associated block valve(s) and remove power from the block valve, and
 1. With only one PORV OPERABLE, restore at least a total of two PORVs to OPERABLE status within the following 72 hours or be in HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours, or
 2. With no PORVs OPERABLE, restore at least one PORV to OPERABLE status within 1 hour or be in HOT STANDBY within the next 6 hours and HOT SHUTDOWN within the following 6 hours.
- c. With one or more block valve(s) inoperable, within 1 hour (1) restore the block valve(s) to OPERABLE status or close the block valve(s) and remove power from the block valve(s) or close the PORV and remove power from its associated solenoid valve; and (2) apply ACTION b above, as appropriate, for the isolated PORV(s).
- d. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.4.4.1 Each PORV shall be demonstrated OPERABLE at least once per 18 months by:

- a. Operating the valve through one complete cycle of full travel, and
- b. Performing a CHANNEL CALIBRATION.

TABLE 3.7-2
STEAM LINE SAFETY VALVES PER LOOP

<u>VALVE NUMBER</u>					<u>LIFT SETTING</u> <u>(± 1%)*</u>	<u>ORIFICE SIZE</u>
SG-1	SG-2	SG-3	SG-4			
1. PSV 3001	PSV 3011	PSV 3021	PSV 3031	1185 psig	16.0 in ² .	
2. PSV 3002	PSV 3012	PSV 3022	PSV 3032	1200 psig	16.0 in ² .	
3. PSV 3003	PSV 3013	PSV 3023	PSV 3033	1210 psig	16.0 in ² .	
4. PSV 3004	PSV 3014	PSV 3024	PSV 3034	1220 psig	16.0 in ² .	
5. PSV 3005	PSV 3015	PSV 3025	PSV 3035	1235 psig	16.0 in ² .	

*The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

PLANT SYSTEMS

AUXILIARY FEEDWATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.1.2 At least three independent steam generator auxiliary feedwater pumps and associated flow paths shall be OPERABLE with:

- a. Two motor-driven auxiliary feedwater pumps, each capable of being powered from separate emergency busses, and
- b. One steam turbine-driven auxiliary feedwater pump capable of being powered from an OPERABLE steam supply system.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

- a. With one auxiliary feedwater pump inoperable, restore the required auxiliary feedwater pumps to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.
- b. With two auxiliary feedwater pumps inoperable, be in at least HOT STANDBY within 6 hours and in HOT SHUTDOWN within the following 6 hours.
- c. With three auxiliary feedwater pumps inoperable, immediately initiate corrective action to restore at least one auxiliary feedwater pump to OPERABLE status as soon as possible.

SURVEILLANCE REQUIREMENTS

4.7.1.2.1 Each auxiliary feedwater pump shall be demonstrated OPERABLE:

- a. At least once per 31 days on a STAGGERED TEST BASIS by:
 - 1) Verifying that each motor-driven pump develops a discharge pressure of greater than or equal to 1605 psig at a flow of greater than or equal to 150 gpm (FI-15101, FI-15102);
 - 2) Verifying that the steam turbine-driven pump develops a discharge pressure of greater than or equal to 1675 psig at a flow of greater than or equal to 145 gpm (FI-15100) when the secondary steam supply pressure (PI-5105A, PI-5105B) is greater than 900 psig. The provisions of Specification 4.0.4 are not applicable for entry into MODE 3.

PLANT SYSTEMS

3/4.7.7 PIPING PENETRATION AREA FILTRATION AND EXHAUST SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.7 Two independent Piping Penetration Area Filtration and Exhaust Systems shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4

ACTION:

With one Piping Penetration Area Filtration and Exhaust System inoperable, restore the inoperable system to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.7.7 Each Piping Penetration Area Filtration and Exhaust System shall be demonstrated OPERABLE:

- a. At least once per 31 days on a STAGGERED TEST BASIS by initiating, from the control room, flow (FI-12629, FI-12542) through the HEPA filters and charcoal adsorbers and verifying that the system operates for at least 10 continuous hours with the heater control circuit energized;
- b. At least once per 18 months or (1) after any structural maintenance on the HEPA filter or charcoal adsorber housings, or (2) following painting, fire, or chemical release in any ventilation zone communicating with the system by:
 1. Verifying that the cleanup system satisfies the in-place testing acceptance criteria of greater than or equal to 99.95% filter retention while operating the system at a flow rate of 15,500 cfm \pm 10% and performing the following tests:
 - (a) A visual inspection of the piping penetration area filtration and exhaust system shall be made before each DOP test or activated carbon adsorber section leak test in accordance with Section 5 of ANSI N510-1980.
 - (b) An in-place DOP test for the HEPA filters shall be performed in accordance with Section 10 of ANSI N510-1980.
 - (c) A charcoal adsorber section leak test with a gaseous halogenated hydrocarbon refrigerant shall be performed in accordance with Section 12 of ANSI N510-1980.

PLANT SYSTEMS

3/4.7.7 PIPING PENETRATION AREA FILTRATION AND EXHAUST SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

- 2) Verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Section 13 of ANSI N510-1980 meets the laboratory testing criterion of greater than or equal to 99.8% when tested with methyl iodide at 30°C and 70% relative humidity.
 - 3) Verifying a system flow rate of 15,500 cfm \pm 10% during system operation when tested in accordance with Section 8 of ANSI N510-1980.
- c. After every 720 hours of charcoal adsorber operation, by verifying, within 31 days after removal, that a laboratory analysis of a representative carbon sample obtained in accordance with Section 13 of ANSI N510-1980 meets the laboratory testing criteria of greater than or equal to 99.8% when tested with methyl iodide at 30°C and 70% relative humidity;
- d. At least once per 18 months by:
- 1) Verifying that the pressure drop across the combined HEPA filters and charcoal adsorber banks is less than 6 inches Water Gauge while operating the system at a flow rate of 15,500 cfm \pm 10%.
 - 2) Verifying that the system starts on a Containment Ventilation Isolation test signal,
 - 3) Verifying that the system maintains the Piping Penetration Filtration Exhaust Unit Room at a negative pressure of greater than or equal to 1/4 inch Water Gauge relative to the outside atmosphere (PDI-2550, PDI-2551), and
 - 4) Verifying that the heaters dissipate 80 \pm 4 kW when tested in accordance with Section 14 of ANSI N510-1980.
- e. After each complete or partial replacement of a HEPA filter bank by verifying that the HEPA filter banks remove greater than or equal to 99.95% of the DOP when they are tested in-place in accordance with Section 10 of ANSI N510-1980 while operating the system at a flow rate of 15,500 cfm \pm 10%.
- f. After each complete or partial replacement of a charcoal adsorber bank by verifying that the charcoal adsorbers remove greater than or equal to 99.95% of a halogenated hydrocarbon refrigerant test gas when they are tested in-place in accordance with Section 12 of ANSI N510-1980 while operating the system at a flow rate of 15,500 cfm \pm 10%.

3/4.9 REFUELING OPERATIONS

3/4.9.1 BORON CONCENTRATION

LIMITING CONDITION FOR OPERATION

3.9.1 The boron concentration of all filled portions of the Reactor Coolant System and the refueling canal shall be maintained uniform and sufficient to ensure that the more restrictive of the following reactivity conditions are met:

- a. A K_{eff} of 0.95 or less, or
- b. A boron concentration of greater than or equal to 2000 ppm.

Additionally, valves 1208-U4-175, 1208-U4-177, 1208-U4-183, and 1208-U4-176 shall be closed and secured in position.

APPLICABILITY: MODE 6.

ACTION:

- a. With the requirements of a. and b. above not satisfied, immediately suspend all operations involving CORE ALTERATIONS or positive reactivity changes and initiate and continue boration at greater than or equal to 30 gpm of a solution containing greater than or equal to 7000 ppm boron or its equivalent until K_{eff} is reduced to less than or equal to 0.95 or the boron concentration is restored to greater than or equal to 2000 ppm, whichever is the more restrictive.
- b. With valves 1208-U4-175, 1208-U4-177, 1208-U4-183, and 1208-U4-176 not closed and secured in position, immediately close and secure in position.

SURVEILLANCE REQUIREMENTS

4.9.1.1 The boron concentration of the Reactor Coolant System and the refueling canal shall be determined by chemical analysis at least once per 72 hours.

4.9.1.2 Valves 1208-U4-175, 1208-U4-177, 1208-U4-183, and 1208-U4-176 shall be verified closed and secured in position by mechanical stops at least once per 31 days.

REFUELING OPERATIONS

3/4.9.2 INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.9.2 As a minimum, two Source Range Neutron Flux Monitors (NI-0031B,D&E, NI-0032B,D&G) shall be OPERABLE, each with continuous visual indication in the control room and one with audible indication in the containment and control room.

APPLICABILITY: MODE 6.

ACTION:

- a. With one of the above required monitors inoperable or not operating, immediately suspend all operations involving CORE ALTERATIONS or positive reactivity changes.
- b. With both of the above required monitors inoperable or not operating, determine the boron concentration of the Reactor Coolant System at least once per 12 hours.

SURVEILLANCE REQUIREMENTS

4.9.2 Each Source Range Neutron Flux Monitor (NI-0031B,D&E, NI-0032B,D&G) shall be demonstrated OPERABLE by performance of:

- a. A CHANNEL CHECK at least once per 12 hours,
- b. An ANALOG CHANNEL OPERATIONAL TEST within 8 hours prior to the initial start of CORE ALTERATIONS, and
- c. An ANALOG CHANNEL OPERATIONAL TEST at least once per 7 days.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 21 TO FACILITY OPERATING LICENSE NPF-68
AND AMENDMENT NO. 2 TO FACILITY OPERATING LICENSE NPF-81

GEORGIA POWER COMPANY, ET AL.

DOCKET NOS. 50-424 AND 50-425

VOGTLE ELECTRIC GENERATING PLANT, UNITS 1 AND 2

1.0 INTRODUCTION

By letter dated April 5, 1989, Georgia Power Company, et al., requested a change to the Technical Specifications (TS) for Vogtle Electric Generating Plant (VEGP), Units 1 and 2. The proposed change deletes footnotes which are no longer applicable after the initial entry of Vogtle Unit 2 into Mode 2 operation. The Technical Specification sections involved are 3.3.3.6, 3.4.4, 3.7.1.2, 3.7.7, 3.9.1, Table 3.3-4, and Table 4.3-3.

2.0 EVALUATION

The subject footnotes were included in the combined Vogtle Unit 1 and Unit 2 TS issued with the Unit 2 low power operating license. The footnotes delay the applicability of certain requirements to Unit 2 during initial fuel load and during initial operation in Modes 3 and 4. After the initial entry of Unit 2 into Mode 2, the footnotes would no longer be applicable.

Unit 2 has completed initial fuel loading and initial entry into Mode 2. In order to prevent confusion with respect to initial entries into Mode 2 after refueling and to remove footnotes that are no longer applicable, Georgia Power Company is proposing to remove the footnotes regarding initial applicability.

The NRC staff has reviewed this request and finds that it deletes footnotes that are no longer applicable. Therefore, the NRC staff concludes that the change is acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

The amendments involve a change in administrative requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register on May 17, 1989 (54 FR 21308), and consulted with the state of Georgia. No public comments were received, and the state of Georgia did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Jon B. Hopkins, PDII-3/DRP-I/II

Dated: August 8, 1989