

February 15, 1995

Mr. Gary J. Taylor
Senior Vice President, Nuclear Operations
South Carolina Electric & Gas Company
Virgil C. Summer Nuclear Station
Post Office Box 88
Jenkinsville, South Carolina 29065

SUBJECT: ISSUANCE OF AMENDMENT NO. 122 TO FACILITY OPERATING LICENSE NO. NPF-12 REGARDING SEISMIC MONITORING INSTRUMENTATION - VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1 (TAC NO. M90765)

Dear Mr. Taylor:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 122 to Facility Operating License No. NPF-12 for the Virgil C. Summer Nuclear Station, Unit No. 1. The amendment changes the Technical Specifications in response to your application dated October 17, 1994, as supplemented January 30, 1995.

The amendment changes the Technical Specifications to relocate the seismic monitoring instrumentation (SMI) Limiting Condition for Operation (LCO), Surveillance Requirements (SRs), and associated tables and bases contained in Technical Specifications (TS) sections 3.3.3.3 and 4.3.3.3 to the Final Safety Analysis Report (FSAR) or an equivalent controlled document.

A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's Bi-weekly Federal Register notice.

Sincerely,

(Original Signed By)

Stephen Dembek, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-395

Enclosures:

1. Amendment No. 122 to NPF-12
2. Safety Evaluation

cc w/enclosures:
See next page

DOCUMENT NAME: G:\SUMMER\SUM90765.AMD - *See previous concurrence

OFFICE	LA:PDII-1	PM:PDII-1	D:PDII-1	ECGB:DE	OGC
NAME	PAAnderson	SDembek	WBateman	RRothman*	MZobler*
DATE	02/15/95	02/15/95	02/15/95	01/31/95	02/08/95
COPY	Yes/No	Yes/No	Yes/No	Yes/No	Yes/No

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Virgil C. Summer Nuclear Station

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AMENDMENT NO. 122 TO FACILITY OPERATING LICENSE NO. NPF-12 - SUMMER, UNIT NO. 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SOUTH CAROLINA ELECTRIC & GAS COMPANY

SOUTH CAROLINA PUBLIC SERVICE AUTHORITY

DOCKET NO. 50-395

VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 122
License No. NPF-12

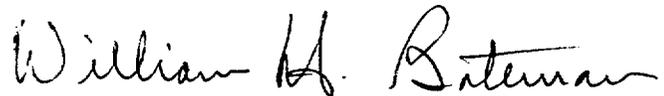
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by South Carolina Electric & Gas Company (the licensee), dated October 17, 1994, as supplemented January 30, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-12 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 122 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. South Carolina Electric & Gas Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



William H. Bateman, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 15, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 122
TO FACILITY OPERATING LICENSE NO. NPF-12
DOCKET NO. 50-395

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are indicated by marginal lines.

<u>Remove Pages</u>	<u>Insert Pages</u>
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Pages 3/4 3-47, 3/4 3-48, and 3/4 3-49 have been deleted.

INSTRUMENTATION

BASES

3/4.3.3.3 SEISMIC INSTRUMENTATION

Deleted.

3/4.3.3.4 METEOROLOGICAL INSTRUMENTATION

The OPERABILITY of the meteorological instrumentation ensures that sufficient meteorological data is available for estimating potential radiation doses to the public as a result of routine or accidental release of radioactive materials to the atmosphere. This capability is required to evaluate the need for initiating protective measures to protect the health and safety of the public and is consistent with the recommendations of Regulatory Guide 1.23, "Onsite Meteorological Programs," February 1972.

3/4.3.3.5 REMOTE SHUTDOWN INSTRUMENTATION

The OPERABILITY of the remote shutdown instrumentation ensures that sufficient capability is available to permit shutdown and maintenance of HOT STANDBY of the facility from locations outside of the control room. This capability is required in the event control room habitability is lost and is consistent with General Design Criteria 19 of 10 CFR 50.

3/4.3.3.6 ACCIDENT MONITORING INSTRUMENTATION

The PAM Instrumentation LCO provides OPERABILITY requirements for Regulatory Guide 1.97 Type A monitors, which provide information required to perform certain manual actions specified in the Emergency Operating Procedures. These manual actions ensure that a system can accomplish its safety function and are credited in the safety analyses. Additionally, this LCO addresses Regulatory Guide 1.97 instruments that have been designated Category I, non-Type A.

The OPERABILITY of the PAM instrumentation ensures there is sufficient information available on selected unit parameters to monitor and assess unit status following an accident.

LCO 3.3.3.6 requires two OPERABLE channels for most Functions. Two OPERABLE channels ensure no single failure prevents operators from getting the information necessary for them to determine the safety status of the unit, and to bring the unit to and maintain it in a safe condition following an accident.

Furthermore, OPERABILITY of two channels allows a CHANNEL CHECK during the post accident phase to confirm the validity of displayed information.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 122 TO FACILITY OPERATING LICENSE NO. NPF-12

SOUTH CAROLINA ELECTRIC & GAS COMPANY

SOUTH CAROLINA PUBLIC SERVICE AUTHORITY

VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-395

1.0 INTRODUCTION

By letter dated October 17, 1994, as supplemented January 30, 1995, South Carolina Electric & Gas Company (the licensee) submitted a request for changes to the Virgil C. Summer Nuclear Station, Unit No. 1 (Summer Station), Technical Specifications (TS). The proposed changes would relocate the seismic monitoring instrumentation (SMI) Limiting Condition for Operation (LCO), Surveillance Requirements (SRs), and associated tables and bases contained in TS sections 3.3.3.3 and 4.3.3.3 to the Final Safety Analysis Report (FSAR) or an equivalent controlled document. The January 30, 1995, supplement did not affect the NRC staff's finding of no significant hazards consideration.

2.0 BACKGROUND

The TS cannot be changed by the licensees without prior NRC approval. However, since 1969, there has been a trend towards including in the TS not only those requirements derived from the analyses and evaluations included in the safety analysis report, but other requirements governing the operation of nuclear power reactors. This trend has contributed to the volume of TS and to the increase in the number of license amendment applications to effect changes to the TS.

In the policy statement published in the Federal Register on July 22, 1993 (58 FR 39132), dated July 16, 1993, effective, July 22, 1993, "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors," the NRC adopted criteria for defining the scope of the TS as required by 10 CFR 50.36. These criteria are used by the NRC and each of the nuclear steam supply system vendor owners' groups to completely rewrite and streamline the existing standard TS. As a result of this process, many requirements are being transferred from the TS to other licensing documents (e.g., the Final Safety Analysis Report (FSAR), Operating Procedures, Quality Assurance (QA) Plan) which, when transferred, will not require a license amendment or prior NRC approval when changes are needed, unless these changes involve an unreviewed safety question (see 10 CFR 50.59(c)).

The following criteria from the "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors," delineate those constraints on design and operation of nuclear power plants that belong in the TS in accordance with 10 CFR 50.36.

Criterion 1: "Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary."

Criterion 2: "A process variable, design feature, or operating restriction that is an initial condition of a Design Basis Accident or Transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier."

Criterion 3: "A structure, system or component that is part of the primary success path and which functions or actuates to mitigate a Design Basis Accident or Transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier."

Criterion 4: "A structure, system or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety."

3.0 EVALUATION

The staff has reviewed the request to relocate the SMI Limiting Condition for Operation, Surveillance Requirements, and associated tables and Bases contained in TS Sections 3.3.3.3 and 4.3.3.3 to the FSAR or an equivalent controlled document. The request was evaluated utilizing the four criteria set forth in the above NRC policy statement. In addition, the proposed amendment does not involve a change in the manner in which the plant will be operated, maintained, or tested. The requirements described in the affected TS will be maintained (with the exception that a Special Report will not be required for equipment inoperable for greater than 30 days) and any subsequent changes to the FSAR or equivalent controlled document, related to these instruments, will be made in accordance with 10 CFR 50.59.

Appendix A to 10 CFR Part 100 requires that seismic monitoring instrumentation be provided to promptly determine the magnitude of a seismic event and evaluate the response of those features important to safety. This capability is required to allow for a comparison of the measured response to that used in the design basis for the unit. Comparison of such data is needed to (1) determine whether the plant can continue to be operated safely and (2) permit such timely action as may be appropriate. However, these components are not factored into accident analyses at Summer Station nor do they affect the margin of safety of the plant. Seismic instrumentation does not actuate any protective equipment or play any direct role in the mitigation of an accident. The capability of the plant to withstand a seismic event or other design-basis accident is determined by the initial design and construction of systems, structures, and components. The instrumentation is used to alert operators to

the seismic event and evaluate the plant response. Therefore, requirements related to the seismic monitoring instrumentation do not satisfy any of the above final policy statement criteria and need not be included in the TS. In addition, the proposed amendment does not involve a change in the manner in which the plant will be operated, maintained, or tested. The requirements described in the affected TS will be maintained with the exception of a Special Report that will not be required for equipment inoperable for greater than 30 days, and any subsequent changes to the plant procedures, the FSAR, or the equivalent controlled documents related to these instruments will be made in accordance with TS 6.8.1 and 10 CFR 50.59.

On this basis, the staff concludes that TS LCO 3.3.3.3, SRs 4.3.3.3.1 and 4.3.3.3.2, Tables 3.3-7 and 4.3-4 and the Bases for TS 3/4.3.3.3, which are related to seismic monitoring instrumentation, do not need to be controlled by TS; changes to these requirements are adequately controlled by 10 CFR 50.59. Should the licensee's determination conclude that an unreviewed safety question is involved, due to either (1) an increase in the probability or consequences of accidents or malfunctions of equipment important to safety, (2) the creation of a possibility for an accident or malfunction of a different type than any evaluated previously, or (3) a reduction in the margin of safety, as defined for any TS, NRC approval and a license amendment would be required prior to implementation of the change. NRC inspection and enforcement programs also enable the staff to monitor facility changes and licensee adherence to commitments and to take any remedial action that may be appropriate.

The staff has concluded, therefore, that relocation of requirements related to seismic monitoring instrumentation (TS LCO 3.3.3.3, SRs 4.3.3.3.1 and 4.3.3.3.2; Tables 3.3-7 and 4.3-4; and the Bases for TS 3/4.3.3.3) and deletion of the requirement for a Special Report when a SMI is inoperable for more than 30 days are acceptable because (1) their inclusion in technical specifications is not specifically required by 10 CFR 50.36 or other regulations, (2) these requirements are not required to avert an immediate threat to the public health and safety, and (3) changes that are deemed to involve an unreviewed safety question will require prior NRC approval in accordance with 10 CFR 50.59(c).

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of South Carolina official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 [and changes the Surveillance Requirements]. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released

offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (59 FR 55717). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Dembek

Date: February 15, 1995