

March 29, 1993

Mr. Gary J. Taylor
Vice President, Nuclear Operations
South Carolina Electric & Gas Company
Virgil C. Summer Nuclear Station
Post Office Box 88
Jenkinsville, South Carolina 29065

SUBJECT: ISSUANCE OF AMENDMENT NO. 123 TO FACILITY OPERATING LICENSE NO. NPF-12 REGARDING DELETION OF LICENSE CONDITIONS - VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1 (TAC NO. M88172)

Dear Mr. Taylor:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 123 to Facility Operating License No. NPF-12 for the Virgil C. Summer Nuclear Station, Unit No. 1. The amendment changes Operating License NPF-12 in response to your application dated October 29, 1993, as supplemented March 11, 1994, May 18, 1994, September 20, 1994, and October 20, 1994.

The amendment deletes License Conditions 2.C.13, 2.C.14, and 2.C.32. These license conditions should have been deleted when Amendment No. 119 was issued, but were inadvertently overlooked.

A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's Bi-weekly Federal Register notice.

Sincerely,

Original signed by

Stephen Dembek, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-395

Enclosures:

1. Pages to Operating License No. NPF-12
2. Safety Evaluation

NRC FILE CENTER COPY

cc w/enclosures:
See next page

DOCUMENT NAME: G:\SUMMER\SUM88172.AMD *See Previous Concurrence

OFFICE	LA: PDII-1	PM: PDII-1	D: PDII-1	OGC*	EMCB: BC*	SRXB: BC
NAME	PAnderson	SDembek	WBateman		JStrosnider	RJones
DATE	03/29/95	03/29/95	03/29/95	03/15/95	03/12/95	03/13/95
COPY	(Yes) No	(Yes) No	Yes/No	Yes/No	Yes/No	Yes/No

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AMENDMENT NO. 123 TO FACILITY OPERATING LICENSE NO. NPF-12 - SUMMER, UNIT NO. 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SOUTH CAROLINA ELECTRIC & GAS COMPANY

SOUTH CAROLINA PUBLIC SERVICE AUTHORITY

DOCKET NO. 50-395

VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1

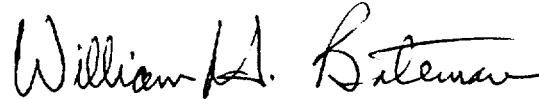
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 123
License No. NPF-12

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by South Carolina Electric & Gas Company (the licensee), dated October 29, 1993, as supplemented March 11, 1994, May 18, 1994, September 20, 1994, and October 20, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, Operating License NPF-12 is amended as indicated in the attachment to this license amendment.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



William H. Bateman, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Pages 7 and 12 to
Operating License NPF-12

Date of Issuance: March 29, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 123
TO FACILITY OPERATING LICENSE NO. NPF-12
DOCKET NO. 50-395

Replace the following pages of Operating License No. NPF-12 with the enclosed pages. The revised pages are indicated by marginal lines.

<u>Remove Pages</u>	<u>Insert Pages</u>
7	7
7a	--
12	12

- c. The reporting of the inservice inspection examination results shall be documented in a manner to define qualitatively whether, the weldment and the heat affected zone and adjacent base metal on both sides of the weld were examined by ultrasonic angle beam techniques.

(12) Design Description - Control (Section 4.3.2, SER)

SCE&G is prohibited from using part-length rods during power operation.

(13) Deleted

(14) Deleted

(15) Deleted

(16) Cable Tray Separation (Section 8.3.3, SSER 4)

Prior to startup after the first refueling outage, SCE&G shall implement the modifications to the cable trays discussed in Section 8.3.3 of Supplement No. 4 to the Safety Evaluation Report or demonstrate to the NRC staff that faults induced in non-Class 1E trays will not result in failure of cable in the adjacent Class 1E cable trays.

(17) Alternate Shutdown System (Section 9.5.1, SSER 4)

Prior to startup after the first refueling outage, SCE&G shall install a source range neutron flux monitor independent of the control complex as part of the alternate system.

- (31) Final NRC Approval of Emergency Preparedness (ASLB Supplemental Partial Initial Decision, August 4, 1982, Section VIII.9)

Prior to exceeding 5% of full power, final NRC staff approval of the state of emergency preparedness for the Virgil C. Summer Nuclear Station site shall be required.

- (32) Deleted

- (33) Emergency Preparedness Exercise (Section 13.3, SSER #5)

Prior to March 31, 1983, SCE&G shall conduct an emergency exercise similar to that conducted on May 5, 1982 but which includes full participation of the local governments and partial State participation.

- 2.D An exemption to the requirements of Paragraph III.B.4 of Appendix G to 10 CFR Part 50 is described in Section 5.3.1 of Supplement No. 1 to the Office of Nuclear Reactor Regulation's Safety Evaluation Report. A limited exemption to the requirements of Section IV.F.1(b) of Appendix E to 10 CFR Part 50 is described in a letter from B. J. Youngblood, NRC to O.W. Dixon, Jr., dated November 2, 1982. These exemptions are authorized by law and will not endanger life or property or the common defense and security and are otherwise in the public interest. The facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.
- 2.E SCE&G shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Virgil C. Summer Nuclear Station Physical Security Plan," with revisions submitted through December 15, 1987; "Virgil C. Summer Nuclear Station Guard Training and Qualification Plan," with revisions submitted through March 24, 1987; and "Virgil C. Summer Nuclear Station Safeguards Contingency Plan," with revisions submitted through March 24, 1987. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 123 TO FACILITY OPERATING LICENSE NO. NPF-12

SOUTH CAROLINA ELECTRIC & GAS COMPANY

SOUTH CAROLINA PUBLIC SERVICE AUTHORITY

VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-395

1.0 INTRODUCTION

By letter dated October 29, 1993, as supplemented March 11, 1994, May 18, 1994, September 20, 1994, and October 20, 1994, South Carolina Electric & Gas Company (the licensee) submitted a request for changes to the Virgil C. Summer Nuclear Station, Unit No. 1 (Summer Station), Operating License and Technical Specifications (TS). The proposed changes supported the installation of new steam generators at Summer Station. The staff performed a Safety Evaluation and granted Operating License Amendment No. 119 on November 18, 1994. However, Operating License Amendment No. 119 inadvertently did not delete License Conditions 2.C.13, 2.C.14, and 2.C.32 as requested by the licensee. This Safety Evaluation is directed at correcting this omission.

The amendment was originally noticed on February 16, 1994 (59 FR 7698). It was renoticed on April 28, 1994 (59 FR 22012), based on the no significant hazards determination contained in the licensee's March 11, 1994, submittal. A Notice of Correction was published in the Federal Register on June 30, 1994 (59 FR 33795), correcting the amendment request date in the second notice from October 29, 1993, to March 11, 1994.

2.0 EVALUATION

The staff's November 18, 1994, Safety Evaluation addressed the TS changes requested by the licensee, but did not address the deletion of related License Conditions 2.C.13, 14, and 32. These license conditions were originally issued for the Model D3 steam generators. The Model D3 steam generators were replaced by Delta 75 steam generators during Summer Station's 1994 refueling outage. In the licensee's October 29, 1993, letter, the licensee stated that these license conditions "...concern the Model D3 Steam Generators and have been considered in the design of the Delta 75 model." The individual license conditions are discussed below:

2.C.(13) Steam Generator Inspection Ports (Section 5.4.2, SER)

Prior to startup after the first refueling outage, SCE&G shall install steam generator inspection ports at the locations specified in Section

5.4.2 of the Safety Evaluation Report unless justification is submitted to the NRC staff for not installing such ports and written approval is given by the NRC staff.

Access ports have been incorporated into the design of the Delta 75 steam generators as described in WCAP-13480, "Westinghouse Delta 75 Steam Generator Design and Fabrication Information for the Virgil C. Summer Nuclear Station," August 1992.

2.C.(14) Model D-3 Steam Generator (Section 5.4.2, SSER #5)

The licensee shall conduct the inspection, testing and monitoring program as described in O.W. Dixon's letter of May 3, 1983.

The licensee shall not make any major modifications to this program unless prior NRC approval is received.

Major modifications are defined as:

- a. Elimination of any identified testing, inspection, or monitoring,
- b. Changes in the frequency of performing the identified testing, inspection or monitoring, and
- c. Reduction in the scope of any of the identified testing, inspection, or monitoring.

The current Surveillance Requirements 4.4.5.0 through 4.4.5.5 delineate the appropriate steam generator inspection, testing, and monitoring requirements.

2.C.(32) Steam Generator Tube Vibration (ASLB Supplemental Partial Initial Decision, August 4, 1982, Section VIII.10)

Prior to operation at full power, SCE&G shall satisfy the NRC staff that appropriate surveillance measures and remedial action plans have been implemented with respect to the steam generator tube failure problem.

The current Surveillance Requirement 4.4.5.0 through 4.4.5.5 delineate the appropriate steam generator inspection, testing, and monitoring requirements.

Therefore, for the reasons stated the NRC staff has concluded that License Conditions 2.C.13, 2.C.14, and 2.C.32 are not applicable for the Delta 75 steam generators installed at Summer Station and their deletion is appropriate.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of South Carolina official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued proposed findings that the amendment involves no significant hazards consideration, and there has been no public comment on such findings (59 FR 7698 and 59 FR 22012). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Dembek

Date: March 29, 1995