

July 11, 1988

Docket No. 50-424

Mr. W. G. Hairston, III
Senior Vice President -
Nuclear Operations
Georgia Power Company
P.O. Box 4545
Atlanta, Georgia 30302

Dear Mr. Hairston:

SUBJECT: ISSUANCE OF AMENDMENT NO.6 TO FACILITY OPERATING LICENSE NPF-68
VOGTLE ELECTRIC GENERATING PLANT, UNIT 1 (TAC 66915)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 6 to Facility Operating License No. NPF-68 for the Vogtle Electric Generating Plant, Unit 1. The amendment is being issued in response to your letter dated May 6, 1988, supplemented June 1, 1988.

The amendment modified the Technical Specifications by replacing the organization charts with more general organization requirements. The amendment is effective as of its date of issuance.

A copy of the related safety evaluation supporting Amendment No. 6 to Facility Operating License NPF-68 is enclosed.

Notice of issuance of the amendment and opportunity for hearing will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

JS

Jon B. Hopkins, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II

Enclosures:

1. Amendment No. 6 to NPF-68
2. Safety Evaluation

cc w/enclosures:

See next page

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Vogle Electric Generating Plant

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DATED: July 11, 1988

AMENDMENT NO. 6 TO FACILITY OPERATING LICENSE NPF-68 - Vogtle Electric
Generating Plant, Unit 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

VOGTLE ELECTRIC GENERATING PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 6
License No. NPF-68

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Vogtle Electric Generating Plant, Unit 1 (the facility) Facility Operating License No. NPF-68 filed by the Georgia Power Company acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensees) dated May 6, 1988, supplemented June 1, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-68 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 6, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. GPC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects

Attachment:

Technical Specification Changes

Date of Issuance: July 11, 1988

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ATTACHMENT TO LICENSE AMENDMENT NO. 6

FACILITY OPERATING LICENSE NO. NPF-68

DOCKET NO. 50-424

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

<u>Amended Page</u>	<u>Overleaf Page</u>
XXII	XXI
6-1	--
6-2	--
6-3	--
6-4	--
6-6	6-5
6-7	--
6-8	--
6-9	6-10
6-12	6-11
6-13	--
6-14	--
6-24	6-23
6-25	--

INDEX

DESIGN FEATURES

<u>SECTION</u>	<u>PAGE</u>
<u>5.1 SITE</u>	
5.1.1 EXCLUSION AREA.....	5-1
5.1.2 LOW POPULATION ZONE.....	5-1
5.1.3 MAP DEFINING UNRESTRICTED AREAS AND SITE BOUNDARY FOR RADIOACTIVE GASEOUS AND LIQUID EFFLUENTS.....	5-1
FIGURE 5.1-1 EXCLUSION AREA.....	5-2
FIGURE 5.1-2 LOW POPULATION ZONE.....	5-3
<u>5.2 CONTAINMENT</u>	
5.2.1 CONFIGURATION.....	5-1
5.2.2 DESIGN PRESSURE AND TEMPERATURE.....	5-1
<u>5.3 REACTOR CORE</u>	
5.3.1 FUEL ASSEMBLIES.....	5-4
5.3.2 CONTROL ROD ASSEMBLIES.....	5-4
<u>5.4 REACTOR COOLANT SYSTEM</u>	
5.4.1 DESIGN PRESSURE AND TEMPERATURE.....	5-4
5.4.2 VOLUME.....	5-4
<u>5.5 METEOROLOGICAL TOWER LOCATION.....</u>	5-4
<u>5.6 FUEL STORAGE</u>	
5.6.1 CRITICALITY.....	5-5
5.6.2 DRAINAGE.....	5-5
5.6.3 CAPACITY.....	5-5
<u>5.7 COMPONENT CYCLIC OR TRANSIENT LIMIT.....</u>	5-5
TABLE 5.7-1 COMPONENT CYCLIC OR TRANSIENT LIMITS.....	5-6

INDEX

ADMINISTRATIVE CONTROLS

<u>SECTION</u>	<u>PAGE</u>
<u>6.1 RESPONSIBILITY</u>	6-1
<u>6.2 ORGANIZATION</u>	6-1
6.2.1 ONSITE AND OFFSITE ORGANIZATION.....	6-1
6.2.2 PLANT STAFF.....	6-1
FIGURE 6.2-1 (DELETED).....	6-3
FIGURE 6.2-2 (DELETED).....	6-4
TABLE 6.2-1 MINIMUM SHIFT CREW COMPOSITION SINGLE UNIT FACILITY.	6-5
6.2.3 INDEPENDENT SAFETY ENGINEERING GROUP (ISEG)	
Function.....	6-6
Composition.....	6-6
Responsibilities.....	6-6
Records.....	6-6
6.2.4 SHIFT TECHNICAL ADVISOR.....	6-6
<u>6.3 TRAINING</u>	6-6
<u>6.4 REVIEW AND AUDIT</u>	6-7
6.4.1 PLANT REVIEW BOARD (PRB)	
Function.....	6-7
Composition.....	6-7
Alternates.....	6-7
Meeting Frequency.....	6-7
Quorum.....	6-7
Responsibilities.....	6-7
Records.....	6-9

ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The General Manager - Nuclear Plant shall be responsible for overall plant operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The General Manager - Nuclear Plant will annually reissue a directive that emphasizes the primary management responsibility of the onshift Operations Supervisor (or during his absence from the control room, the individual designated to assume the command functions) for safe operation of the plant under all conditions on his shift and that clearly establishes his command duties.

6.2 ORGANIZATION

ONSITE AND OFFSITE ORGANIZATION

6.2.1 Onsite and offsite organizations shall be established for plant operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the FSAR.
- b. The General Manager - Nuclear Plant shall be responsible for overall plant safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Vice President - Nuclear shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

PLANT STAFF

6.2.2 The plant organization shall be subject to the following:

- a. Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1;

ADMINISTRATIVE CONTROLS

PLANT STAFF (Continued)

- b. At least one licensed Operator shall be in the control room when fuel is in the reactor. In addition, while the unit is in MODE 1, 2, 3, or 4, at least one licensed Senior Operator shall be in the control room;
- c. An individual* who has successfully completed the Initial Technician Training portion of the Health Physics Training Program or its equivalent shall be on site when fuel is in the reactor;
- d. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Operator or licensed Senior Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation;
- e. Administrative procedures shall be developed and implemented to limit the working hours of plant staff in performance of safety-related functions (e.g., licensed Senior Operators, licensed Operators, key Health Physics Technicians, key non-licensed operators, and key maintenance personnel).

Adequate shift coverage shall be maintained without routine heavy use of overtime. The objective shall be to have operating personnel work a nominal 40-hour week while the plant is operating. (This work week may consist of 12-hour shift schedules.) However, in the event that unforeseen problems require substantial amounts of overtime to be used, or during extended periods of shutdown for refueling, major maintenance, or major plant modification, on a temporary basis the following guidelines shall be followed:

1. An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time.
2. An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any 7-day period, all excluding shift turnover time.
3. A break of at least 8 hours should be allowed between work periods, including shift turnover time.
4. Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the applicable department superintendent, or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual excess overtime shall be reviewed monthly by the General Manager - Nuclear Plant or his designee to assure that excessive hours were authorized and that they do not become routine.

*This individual may be absent for a period of time not to exceed 2 hours, in order to accommodate unexpected absence, provided immediate action is taken to fill the required position.

FIGURE 6.2-1 (DELETED)

FIGURE 6.2-2 (DELETED)

TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION
SINGLE UNIT FACILITY

POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODE 1, 2, 3, or 4	MODE 5 or 6
OS	1	1
SRO	1	None
RO	2	1
NLO	2	1
STA	1*	None

- OS - Operations Supervisor with a Senior Operator license on Unit 1
- SRO - Individual with a Senior Operator license on Unit 1
- RO - Individual with an Operator license on Unit 1
- NLO - Non-Licensed Operator
- STA - Shift Technical Advisor

The shift crew composition may be one less than the minimum requirements of Table 6.2-1 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

During any absence of the Operations Supervisor from the control room while the unit is in MODE 1, 2, 3, or 4, an individual with a valid Senior Operator license shall be designated to assume the control room command function. During any absence of the Operations Supervisor from the control room while the unit is in MODE 5 or 6, an individual with a valid Senior Operator license or Operator license shall be designated to assume the control room command function.

*The STA position shall be manned in MODES 1, 2, 3, and 4 unless the Operations Supervisor or the individual with a Senior Operator license meets the qualifications for the STA as required by the NRC.

ADMINISTRATIVE CONTROLS

6.2.3 INDEPENDENT SAFETY ENGINEERING GROUP (ISEG)

FUNCTION

6.2.3.1 The ISEG shall function to examine plant operating characteristics, NRC issuances, industry advisories, Licensee Event Reports, and other sources of plant design and operating experience information, which may indicate areas for improving plant safety. The ISEG shall make detailed recommendations for revised procedures, equipment modifications, maintenance activities, operations activities, or other means of improving plant safety to the Vice President - Nuclear.

COMPOSITION

6.2.3.2 The ISEG shall be composed of at least five, dedicated, full-time engineers. Each shall have a bachelor's degree in engineering or related science and at least 2 years professional level experience in his field, at least 1 year of which experience shall be in the nuclear field.

RESPONSIBILITIES

6.2.3.3 The ISEG shall be responsible for maintaining surveillance of plant activities to provide independent verification* that these activities are performed correctly and that human errors are reduced as much as practical.

RECORDS

6.2.3.4 Records of activities performed by the ISEG shall be prepared, maintained, and forwarded each calendar month to the Vice President - Nuclear.

6.2.4 SHIFT TECHNICAL ADVISOR

6.2.4.1 The Shift Technical Advisor shall provide advisory technical support to the Shift Supervisor in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the plant. The Shift Technical Advisor shall have a bachelor's degree or equivalent in a scientific or engineering discipline and shall have received specific training in the response and analysis of the plant for transients and accidents, and in plant design and layout, including the capabilities of instrumentation and controls in the control room.

6.3 TRAINING

6.3.1 A retraining and replacement training program for the plant staff shall be maintained under the direction of the Plant Training and Emergency Preparedness Manager. Personnel will meet the minimum education and experience recommendations of ANSI N18.1-1971 and, for licensed staff, Appendix A of 10 CFR 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, before they are considered qualified to perform all duties independently. Prior to meeting the recommendations of ANSI N18.1-1971, personnel may be trained to perform specific tasks and will be qualified to perform those tasks independently.

*Not responsible for sign-off function.

ADMINISTRATIVE CONTROLS

6.4 REVIEW AND AUDIT

6.4.1 PLANT REVIEW BOARD (PRB)

FUNCTION

6.4.1.1 The PRB shall function to advise the General Manager - Nuclear Plant on all matters related to nuclear safety.

COMPOSITION

6.4.1.2 The PRB shall be composed of Department Superintendents or Managers, or supervisory personnel reporting directly to Department Superintendents or Managers from the departments listed below:

- Operations
- Maintenance
- Quality Control
- Health Physics
- Nuclear Safety and Compliance
- Engineering Support

A senior health physicist is acceptable for the Health Physics Department PRB representative. The chairman, his alternate and other members and their alternates of the PRB shall be designated by the General Manager - Nuclear Plant.

ALTERNATES

6.4.1.3 No more than two alternates shall participate as voting members in PRB activities at any one time.

MEETING FREQUENCY

6.4.1.4 The PRB shall meet at least once per calendar month and as convened by the PRB Chairman or his designated alternate.

QUORUM

6.4.1.5 The quorum of the PRB necessary for the performance of the PRB responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.4.1.6 The PRB shall be responsible for:

- a. Review of 1) procedures which establish plant-wide administrative controls to implement the QA program or Technical Specifications surveillance program, 2) procedures for changing plant operating modes, 3) emergency and abnormal operating procedures, 4) procedures for effluent releases of radiological consequences, and 5) fuel handling procedures.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- b. Review of 1) programs required by Specification 6.7.4 and changes thereto, and 2) proposed procedures and changes to procedures which involve an unreviewed safety question as per 10 CFR 50.59.
 - c. Review of all proposed tests and experiments that affect nuclear safety;
 - d. Review of all proposed changes to the Technical Specifications;
 - e. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety;
 - f. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence, to the Vice President - Nuclear and to the Safety Review Board;
 - g. Review of all REPORTABLE EVENTS;
 - h. Review of plant operations to detect potential hazards to nuclear safety;
 - i. Performance of special reviews, investigations, or analyses and reports thereon as requested by the General Manager - Nuclear Plant or the Safety Review Board;
 - j. Review of the Security Plan and implementing procedures and submittal of recommended changes to the General Manager - Nuclear Plant and the Safety Review Board;
 - k. Review of the Emergency Plan and implementing procedures and submittal of recommended changes to the General Manager - Nuclear Plant and the Safety Review Board;
 - l. Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President - Nuclear and to the Safety Review Board;
 - m. Review of changes to the PROCESS CONTROL PROGRAM, the OFFSITE DOSE CALCULATION MANUAL, and the Radwaste Treatment Systems; and
 - n. Review of the Fire Protection Program and Implementing procedures and submittal of recommended changes to the General Mngager - Nuclear Plant.
- 6.4.1.7 The PRB shall:
- a. Recommend in writing to the General Manager - Nuclear Plant approval or disapproval of items considered under Specification 6.4.1.6a. through e. prior to their implementation;

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- b. Render determinations in writing with regard to whether or not each item considered under Specification 6.4.1.6a. through f. constitutes an unreviewed safety question; and
- c. Provide written notification within 24 hours to the Vice President - Nuclear and the Safety Review Board of disagreement between the PRB and the General Manager - Nuclear Plant; however, the General Manager - Nuclear Plant shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1.

RECORDS

6.4.1.8 The PRB shall maintain written minutes of each PRB meeting that, at a minimum, document the results of all PRB activities performed under the responsibility provisions of these Technical Specifications. Copies shall be provided to the Vice President - Nuclear and the Safety Review Board.

6.4.2 SAFETY REVIEW BOARD (SRB)

FUNCTION

6.4.2.1 The SRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,
- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering, and
- h. Quality assurance practices.

The SRB shall report to and advise the Vice President - Nuclear on those areas of responsibility specified in Specifications 6.4.2.7 and 6.4.2.8.

COMPOSITION

6.4.2.2 The SRB shall be organized as one board for all GPC Nuclear power plants. The SRB shall be composed of a minimum of five persons who, as a group, provide the expertise to review and audit the operation of a nuclear power plant. The chairman and other members shall be appointed by the Vice President - Nuclear or other such person as he may designate. The composition of the SRB shall meet the requirements of ANSI N18.7-1976.

ADMINISTRATIVE CONTROLS

ALTERNATES

6.4.2.3 All alternate members shall be appointed in writing by the SRB Chairman to serve on a temporary basis; however, no more than a minority of alternates shall participate as voting members in SRB activities at any one time.

CONSULTANTS

6.4.2.4 Consultants shall be utilized as determined by the SRB chairman to provide expert advice to the SRB.

MEETING FREQUENCY

6.4.2.5 The SRB shall meet at least once per calendar quarter during the initial year of plant operation following fuel loading and at least once per 6 months thereafter.

QUORUM

6.4.2.6 The quorum of the SRB necessary for the performance of the SRB review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least a majority of the SRB members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the plant.

REVIEW

6.4.2.7 The SRB shall be responsible for the review of:

- a. The safety evaluations for: (1) changes to procedures, equipment, or systems; and (2) tests or experiments completed under the provision of 10 CFR 50.59, to verify that such actions did not constitute an unreviewed safety question;
- b. Proposed changes to procedures, equipment, or systems which involve an unreviewed safety question as defined in 10 CFR 50.59;
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59;
- d. Proposed changes to Technical Specifications or this Operating License;
- e. Violations of Codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety;

ADMINISTRATIVE CONTROLS

REVIEW (continued)

- g. All REPORTABLE EVENTS;
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety; and
- i. Reports and meeting minutes of the PRB.

AUDITS

6.4.2.8 Audits of plant activities shall be performed under the cognizance of the SRB. Each inspection or audit shall be performed within the specified time interval with:

- 1. A maximum allowable extension not to exceed 25% of the inspection audit interval.
- 2. A total maximum combined interval time for any 3 consecutive inspection or audit intervals not to exceed 3.25 times the specified inspection or audit interval.

These audits shall encompass:

- a. The conformance of plant operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months;
- b. The performance, training, and qualifications of the entire plant staff at least once per 12 months;
- c. The results of actions taken to correct deficiencies occurring in plant equipment, structures, systems, or method of operation that affect nuclear safety, at least once per 6 months;
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50, at least once per 24 months;
- e. The fire protection programmatic controls including the implementing procedures at least once per 24 months by qualified licensee QA personnel;
- f. The fire protection equipment and program implementation at least once per 12 months utilizing either a qualified offsite licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used as least every third year;

ADMINISTRATIVE CONTROLS

AUDITS (continued)

- g. The Radiological Environmental Monitoring Program and the results thereof at least once per 12 months;
- h. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months;
- i. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months;
- j. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring at least once per 12 months;
- k. The Emergency Plan and implementing procedures (at least once per 12 months);
- l. The Security Plan and implementing procedures (at least once per 12 months); and

RECORDS

6.4.2.9 Records of SRB activities shall be prepared, approved, and distributed as indicated below:

- a. Minutes of each SRB meeting shall be prepared, approved, and forwarded to the Vice President - Nuclear within 14 days following each meeting;
- b. Reports of reviews encompassed by Specification 6.4.2.7 shall be prepared, approved, and forwarded to the Vice President - Nuclear within 14 days following completion of the review; and
- c. Audit reports encompassed by Specification 6.4.2.8 shall be forwarded to the Vice President - Nuclear and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

6.5 REPORTABLE EVENT ACTION

6.5.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Section 50.72 and Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the PRB, and the results of this review shall be submitted to the SRB and the Vice President - Nuclear.

ADMINISTRATIVE CONTROLS

6.6 SAFETY LIMIT VIOLATION

6.6.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. In accordance with 10 CFR 50.72, the NRC Operations Center shall be notified by telephone as soon as practical and in all cases within one hour after the violation has been determined. The Vice President - Nuclear, the SRB, PRB, and the General Manager - Nuclear Plant shall be notified within 24 hours.
- b. A Licensee Event Report shall be prepared in accordance with 10 CFR 50.73.
- c. The Licensee Event Report shall be submitted to the Commission in accordance with 10 CFR 50.73, and to the PRB, SRB, the General Manager - Nuclear Plant and the Vice President - Nuclear within 30 days after discovery of the event.
- d. Critical operation of the unit shall not be resumed until authorized by the Nuclear Regulatory Commission.

6.7 PROCEDURES AND PROGRAMS

6.7.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978;
- b. The emergency operating procedures required to implement the requirements of NUREG-0737 and Supplement 1 to NUREG-0737 as stated in Generic Letter No. 82-33;
- c. Security Plan implementation;
- d. Emergency Plan implementation;
- e. PROCESS CONTROL PROGRAM implementation;
- f. OFFSITE DOSE CALCULATION MANUAL implementation;
- g. Quality Assurance for effluent and environmental monitoring;
- h. Fire Protection Program Implementation; and
- i. Technical Specifications Improvement Program implementation.

6.7.2 Each procedure of 6.7.1 above, and changes thereto, shall be reviewed as set forth in administrative procedures and approved by either the General Manager - Nuclear Plant or the department head of the responsible department prior to implementation with the exception of the following which shall be approved by the General Manager - Nuclear Plant:

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

- 1) procedures which establish plant-wide administrative controls (which implement the quality assurance program and the Technical Specifications surveillance program),
- 2) unit operating procedures (UOPs)
- 3) emergency operating procedures (EOPs)
- 4) abnormal operating procedures (AOPs)
- 5) procedures for implementing the security plan, emergency plan, and the fire protection program, and
- 6) fuel handling procedures.

PRB responsibilities for procedures are delineated in 6.4.1.

6.7.3 Temporary changes to procedures of Specification 6.7.1 may be made provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Operator license; and
- c. The change is documented, reviewed in accordance with Specification 6.7.2 and approved by the General manager - Nuclear Plant or department head of the responsible department within 14 days of implementation.

6.7.4 The following programs shall be established, implemented, and maintained:

a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the following:

- 1) Residual Heat Removal System
- 2) Containment Spray System (excluding NaOH Subsystem)
- 3) Safety Injection (excluding Boron Injection & Accumulators)
- 4) Chemical and Volume Control System (Letdown, Boron Recycle, and Charging Pumps)
- 5) Post Accident Processing System
- 6) Gaseous Waste Processing System
- 7) Nuclear Sampling System (Pressurizer steam and liquid sample lines, Reactor Coolant sample lines, RHR sample lines, CVCS Demineralizer and Letdown Heat Exchanger sample lines only)

ADMINISTRATIVE CONTROLS

6.11 HIGH RADIATION AREA (Continued)

- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them; or
- c. An individual qualified in radiation protection procedures with a radiation dose rate monitoring device, who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the Health Physics Superintendent in the RWP.

~~6.11.2~~ In addition to the requirements of Specification 6.11.1, areas accessible to personnel with radiation levels greater than 1000 mR/h at 45 cm (18 in.) from the radiation source or from any surface which the radiation penetrates shall be provided with locked doors to prevent unauthorized entry, and the keys shall be maintained under the administrative control of the shift Foreman on duty and/or health physics supervision. Doors shall remain locked except during periods of access by personnel under an approved RWP which shall specify the dose rate levels in the immediate work areas and the maximum allowable stay time for individuals in that area. In lieu of the stay time specification of the RWP, direct or remote (such as closed circuit TV cameras) continuous surveillance may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities being performed within the area.

For individual high radiation areas accessible to personnel with radiation levels of greater than 1000 mR/h that are located within large areas, such as PWR containment, where no enclosure exists for purposes of locking, and where no enclosure can be reasonably constructed around the individual area, that individual area shall be barricaded, conspicuously posted, and a flashing light shall be activated as a warning device.

6.12 PROCESS CONTROL PROGRAM (PCP)

6.12.1 The PCP shall be approved by the Commission prior to implementation.

6.12.2 Licensee-initiated changes to the PCP:

- a. Shall be submitted to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the change(s) was made. This submittal shall contain:
 - 1) Sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information;
 - 2) A determination that the change did not reduce the overall conformance of the solidified waste product to existing criteria for solid wastes; and

ADMINISTRATIVE CONTROLS

6.12 PROCESS CONTROL PROGRAM (PCP) (Continued)

- 3) Documentation of the fact that the change has been reviewed and found acceptable by the PRB.
- b. Shall become effective upon approval by the General Manager - Nuclear Plant.

6.13 OFFSITE DOSE CALCULATION MANUAL (ODCM)

6.13.1 The ODCM shall be approved by the Commission prior to implementation.

6.13.2 Licensee-initiated changes to the ODCM:

- a. Shall be submitted to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the change(s) was made effective. This submittal shall contain:
 - 1) Sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information. Information submitted should consist of a package of those pages of the ODCM to be changed with each page numbered, dated and containing the revision number, together with appropriate analyses or evaluations justifying the change(s);
 - 2) A determination that the change will not reduce the accuracy or reliability of dose calculations or Setpoint determinations; and
 - 3) Documentation of the fact that the change has been reviewed and found acceptable by the PRB.
- b. Shall become effective upon approval by the General Manager - Nuclear Plant.

6.14 MAJOR CHANGES TO LIQUID, GASEOUS, AND SOLID RADWASTE TREATMENT SYSTEMS*

6.14.1 Licensee-initiated major changes to the Radwaste Treatment Systems (liquid, gaseous, and solid):

- a. Shall be reported to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the evaluation was reviewed by the PRB. The discussion of each change shall contain:
 - 1) A summary of the evaluation that led to the determination that the change could be made in accordance with 10 CFR 50.59;
 - 2) Sufficient detailed information to totally support the reason for the change without benefit of additional or supplemental information;

*Licensees may choose to submit the information called for in this Specification as part of the annual FSAR update.

ADMINISTRATIVE CONTROLS

6.14 MAJOR CHANGES TO LIQUID, GASEOUS, AND SOLID RADWASTE TREATMENT SYSTEMS (Continued)

- 3) A detailed description of the equipment, components, and processes involved and the interfaces with other plant systems;
 - 4) An evaluation of the change, which shows the predicted releases of radioactive materials in liquid and gaseous effluents and/or quantity of solid waste that differ from those previously predicted in the License application and amendments thereto;
 - 5) An evaluation of the change, which shows the expected maximum exposures to a MEMBER OF THE PUBLIC in the UNRESTRICTED AREA and to the general population that differ from those previously estimated in the License application and amendments thereto;
 - 6) A comparison of the predicted releases of radioactive materials, in liquid and gaseous effluents and in solid waste, to the actual releases for the period prior to when the change is to be made;
 - 7) An estimate of the exposure to plant operating personnel as a result of the change; and
 - 8) Documentation of the fact that the change was reviewed and found acceptable by the PRB.
- b. Shall become effective upon approval by the General Manager - Nuclear Plant.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 6 TO FACILITY OPERATING LICENSE NPF-68

GEORGIA POWER COMPANY, ET AL

DOCKET NO. 50-424

VOGTLE ELECTRIC GENERATING PLANT, UNIT 1

1.0 INTRODUCTION

By letter dated May 6, 1988, supplemented June 1, 1988, Georgia Power Company (GPC) proposed to replace Technical Specification (TS) Figure 6.2-1, "Offsite Organization," and TS Figure 6.2-2, "Plant Organization," with a narrative description of the offsite and onsite organizations' functional requirements. Guidance for these proposed changes to the TS was provided to licensees and applicants by Generic Letter (GL) 88-06, "Removal of Organization Charts from Technical Specification Administrative Control Requirements" dated March 22, 1988.

2.0 BACKGROUND

Consistent with the guidance provided in the Standard Technical Specifications, Sections 6.2.1 and 6.2.2 of the administrative control requirements have referenced offsite and unit (onsite) organization charts that are provided as figures to these sections. On a plant specific basis, these organization charts have been provided by applicants and included in the TS issued with the operating license. Subsequent restructuring of either the offsite or unit organizations, following the issuance of an operating license, has required licensees to submit a license amendment for NRC approval to reflect the desired changes in these organizations. As a consequence, organizational changes have necessitated the need to request an amendment of the operating license.

Because of these limitations on organization structure, the nuclear industry has highlighted this as an area for improvement in the TS. The Shearon Harris licensee proposed changes to remove organization charts from its TS under the lead-plant concept that included the endorsement of the proposed changes by the Westinghouse Owners Group. In its review of the Shearon Harris (a lead-plant) proposal, the staff concluded that most of the essential elements of offsite and onsite organization charts are captured by other regulatory requirements, notably Appendix B to 10 CFR Part 50.

However, there were aspects of the organizational structure that are important to ensure that the administrative control requirements of 10 CFR 50.36 would be met and that would not be retained with the removal of the organization charts. The applicable regulatory requirements are those administrative controls that are necessary to ensure safe operation of the

facility. Therefore, those aspects of organization charts for Shearon Harris that were essential for conformance with regulatory requirements were added (1) to TS 6.2.1 to define functional requirements for the offsite and onsite organizations and (2) to TS 6.2.2 to define qualification requirements of their unit staff.

By letter dated January 27, 1988, the staff issued Amendment No. 3 to Facility Operating License NPF-63 for the Shearon Harris Nuclear Power Plant that incorporated these changes to their TS. Subsequently, the staff developed guidance on an acceptable format for license amendment requests to remove the organization charts from TS. GL 88-06 provided this guidance to all power reactors.

3.0 EVALUATION

By letter dated May 6, 1988, supplemented June 1, 1988, GPC states that the proposed changes to the TS are in accordance with the guidance provided by GL 88-06. The NRC staff evaluation follows:

1. GPC requested the deletion of Figures 6.2-1 (Offsite Organization) and 6.2-2 (Plant Organization) from the Technical Specifications. In conjunction with this change, Section 6.2-1 will be revised to delete references to the figures and to add general requirements that capture the essential elements of the organizational structure. The licensee has stated that equivalent information to that contained in the organization charts will be documented in the Final Safety Analysis Report.

We find this requested change acceptable as it conforms to the guidance of Generic Letter 88-06, "Removal of Organization Charts from Technical Specification Administrative Control Requirements."

2. GPC requested the revision of Sections 6.2.3, 6.4.1, 6.4.2, 6.5.1 and 6.6.1 to change the reference in each of these sections from Senior Vice President-Nuclear Operations to the new title of Vice President-Nuclear.

We find these requested changes acceptable as the position of Vice President-Nuclear is a sufficiently high position to implement the responsibilities designated within each of these sections.

3. GPC requested the revision of Sections 6.1.1, 6.1.2, 6.2.2, 6.4.1, 6.6.1, 6.7.3, 6.12, 6.13, and 6.14 to change the reference in each of these sections to the new title of General Manager-Nuclear Plant from General Manager-Vogtle Nuclear Operations.

We find the requested changes acceptable as the title General Manager-Nuclear Plant is synonymous with the title General Manager - Vogtle Nuclear Operations.

4. GPC requested the deletion of the requirement in Section 6.2.3.4 that records of activities of the Independent Safety Engineering Group (ISEG) be forwarded through the Manager-Nuclear Performance and Analysis to the Vice President-Nuclear.

We find this requested change acceptable as there is no requirement that ISEG records be forwarded through the Manager-Nuclear Performance and Analysis.

On the basis of its review as discussed above, the NRC staff concludes that the licensee has provided an acceptable response to these items as addressed in the NRC guidance on removing organization charts from the administrative control requirements of the TS. Furthermore, the NRC staff finds that these changes are consistent with the staff's generic finding on the acceptability of such changes as noted in GL 88-06. Accordingly, the NRC staff finds the TS changes to be acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments relate to changes in recordkeeping, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration as published on June 3, 1988 in the Federal Register (53 FR 20398), and consulted with the State of Georgia. No public comments were received, and the State of Georgia did not have any comments.

On the basis of the considerations discussed above, the staff concludes that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: F. Allenspach, LPEB/DLPQ
J. Hopkins, PDII-3/DRP-I/II

Dated: July 11, 1988

July 11, 1988

Docket No. 50-424

DISTRIBUTION:

MEMORANDUM FOR: Sholly Coordinator

Docket File
PDII-3 Reading
Vogle Reading

FROM: Jon B. Hopkins, Project Manager
Project Directorate II-3
Division of Reactor Projects-I/II

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SUBJECT: REQUEST FOR PUBLICATION IN BIWEEKLY FR NOTICE - NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE (TAC 66915)

Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, City of Dalton, Georgia, Docket No. 50-424, Vogtle Electric Generating Plant, Unit 1, Burke County, Georgia

Date of application for amendment: May 6, 1988, supplemented June 1, 1988

Brief description of amendment: The amendment modified the Technical Specifications by replacing the organization charts with more general organization requirements.

Date of issuance: July 11, 1988

Effective date: July 11, 1988

Amendment No.: 6

Facility Operating License No. NPF-68: Amendment revised the Technical Specifications.

Date of initial notice in FEDERAL REGISTER: June 3, 1988 (53 FR 20398)

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated July 11, 1988.

No significant hazards consideration comments received: No.

Local Public Document Room location: Burke County Library, 412 Fourth Street, Waynesboro, Georgia 30830

151

Jon B. Hopkins, Project Manager
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