

November 20, 1987

Docket No.: 50-424

Mr. James P. O'Reilly  
Senior Vice President - Nuclear Operations  
Georgia Power Company  
P.O. Box 4545  
Atlanta, Georgia 30302

Dear Mr. O'Reilly:

Subject: Issuance of Amendment No. 3 to Facility Operating License NPF-68  
Vogtle Electric Generating Plant, Unit 1 (TAC 66127)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 3 to Facility Operating License NPF-68 for the Vogtle Electric Generating Plant, Unit 1. The amendment is being issued in response to your letter dated August 26, 1987.

The amendment modifies the Technical Specifications to revise the action requirement for inoperable Fuel Handling Building Post-Accident Ventilation System actuation instrumentation. The amendment is effective as of its date of issuance.

A copy of the related safety evaluation supporting Amendment No. 3 to Facility Operating License NPF-68 is enclosed.

Notice of issuance of the amendment and opportunity for hearing will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

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Melanie A. Miller, Project Manager  
Project Directorate II-3  
Division of Reactor Projects-I/II

Enclosures:

- 1. Amendment No. 3 to NPF-68
- 2. Safety Evaluation

cc w/encl:  
See next page

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PDR ADDCK 05000424  
PDR

PDII-3/DRPI/II  
MDuncan/mac  
09/25/87

PDII-3/DRPI/II  
MMiller  
09/25/87

*[Signature]*  
PDII-3/DRPI/II  
K. Jabour, Acting PD  
~~09/18/87~~  
L Crocker,  
11/20/87

*[Signature]*  
11-13-87

Mr. J. P. O'Reilly  
Georgia Power Company

Vogtle Electric Generating Plant

cc:

Mr. L. T. Gucwa  
Chief Nuclear Engineer  
Georgia Power Company  
P.O. Box 4545  
Atlanta, Georgia 30302

Resident Inspector  
Nuclear Regulatory Commission  
P. O. Box 572  
Waynesboro, Georgia 30830

Mr. Ruble A. Thomas  
Vice President - Licensing  
Vogtle Project  
Georgia Power Company/  
Southern Company Services, Inc.  
P.O. Box 2625  
Birmingham, Alabama 35202

Deppish Kirkland, III, Counsel  
Office of the Consumers' Utility  
Council  
Suite 225  
32 Peachtree Street, N.W.  
Atlanta, Georgia 30303

Mr. Paul D. Rice  
Vice President & Project General Manager  
Georgia Power Company  
Post Office Box 299A, Route 2  
Waynesboro, Georgia 30830

James E. Joiner  
Troutman, Sanders, Lockerman,  
& Ashmore  
Candler Building  
127 Peachtree Street, N.E.  
Atlanta, Georgia 30303

Mr. J. A. Bailey  
Project Licensing Manager  
Southern Company Services, Inc.  
P.O. Box 2625  
Birmingham, Alabama 35202

Danny Feig  
1130 Alta Avenue  
Atlanta, Georgia 30307

Ernest L. Blake, Jr.  
Bruce W. Churchill, Esq.  
Shaw, Pittman, Potts and Trowbridge  
2300 N Street, N. W.  
Washington, D. C. 20037

Carol Stangler  
Georgians Against Nuclear Energy  
425 Euclid Terrace  
Atlanta, Georgia 30307

Mr. G. Bockhold, Jr.  
Vogtle Plant Manager  
Georgia Power Company  
Route 2, Box 299-A  
Waynesboro, Georgia 30830

Regional Administrator, Region II  
U.S. Nuclear Regulatory Commission  
101 Marietta Street, N.W., Suite 2900  
Atlanta, Georgia 30323

Mr. R. E. Conway  
Senior Vice President and Project Director  
Georgia Power Company  
Rt. 2, P. O. Box 299A  
Waynesboro, Georgia 30830

November 20, 1987

AMENDMENT NO. 3 TO FACILITY OPERATING LICENSE NPF-68 -  
VOGTLE ELECTRIC GENERATING PLANT, UNIT 1

DISTRIBUTION: w/enclosures:

~~30-424~~  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-424

VOGTLE ELECTRIC GENERATING PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 3  
License No. NPF-68

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Vogtle Electric Generating Plant, Unit 1 (the facility) Facility Operating License No. NPF-68 filed by the Georgia Power Company acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensees) dated August 26, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and;
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-68 is hereby amended to read as follows:

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P PDR

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 3, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. GPC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

- 3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Lawrence P. Crocker, Acting Director  
Project Directorate II-3  
Division of Reactor Projects

Attachment:  
Technical Specification Changes

Date of Issuance: November 20, 1987

\* SEE PREVIOUS CONCURRENCES

PDII-3/DRPI/II  
\*MDuncan/mac  
09/25/87

*mb*

PD#II-3/DRP-I/II  
LCRocker, Acting PD  
11/20/87

PDII-3/DRPI/II  
\*MMiller  
09/25/87

PD#II-3/DRP-I/II  
GLafinas  
11/10/87

SCIB  
\*JBJ  
09/28/87

OGC-Bethesda  
\*EReis, JScinto  
10/22/87

ATTACHMENT TO LICENSE AMENDMENT NO. 3

FACILITY OPERATING LICENSE NO. NPF-68

DOCKET NO. 50-424

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

<u>Amended Page</u>	<u>Overleaf Page</u>
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3/4 3-25	3/4 3-26
3/4 3-27	

TABLE 3.3-2 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
e. Trip of All Main Feedwater Pumps, Start Motor-Driven Pumps	2	2	2	1, 2	23
f. Manual Initiation	3 <sup>h</sup>	1/pump	1/pump	1, 2, 3	23
7. Semi-Automatic Switchover to Containment Emergency Sump					
a. Automatic Actuation Logic and Actuation Relays	2	1	2	1, 2, 3, 4	14
b. RWST Level--Low-Low Coincident with Safety Injection (LI-0990A&B, LI-0991A&B, LI-0992A, LI-0993A)	4	2	3	1, 2, 3, 4	17
See Functional Unit 1. above for all Safety Injection initiating functions and requirements.					
8. Loss of Power to 4.16 kV ESF Bus					
a. 4.16 kV ESF Bus Undervoltage-Loss of Voltage	4/bus	2/bus	3/bus	1, 2, 3, 4	20 <sup>d</sup>
b. 4.16 kV ESF Bus Undervoltage-Degraded Voltage	4/bus	2/bus	3/bus	1, 2, 3, 4	20 <sup>d</sup>
9. Engineered Safety Features Actuation System Interlocks					
a. Pressurizer Pressure, P11 (PI-0455A,B&C, PI-0456 & PI-0456A, PI-0457 & PI-0457A)	3	2	2	1, 2, 3	21
b. Reactor Trip, P-4	2	1	2	1, 2, 3	23

VOGTLE - UNIT 1

3/4 3-23

TABLE 3.3-2 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>MINIMUM CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
10. Control Room Ventilation Emergency Mode Actuation					
a. Manual Initiation	2	1	1	1, 2, 3, 4, 5 <sup>e</sup> , 6 <sup>e</sup>	26 <sup>d</sup>
b. Automatic Actuation Logic and Actuation Relays	2	1	2	1, 2, 3, 4, 5 <sup>e</sup> , 6 <sup>e</sup>	26 <sup>d</sup>
c. Safety Injection	See Functional Unit 1 above for all Safety Injection initiating functions and requirements.				
d. Intake Radiogas Monitor (RE-12116, RE-12117)	2	1	2	1, 2, 3, 4, 5 <sup>e</sup> , 6 <sup>e</sup>	26 <sup>d</sup>
11. Fuel Handling Building Post Accident Ventilation Actuation					
a. Manual Initiation	2	1	1	i	j
b. Fuel Handling Building Exhaust Duct Radiation Signal (ARE-2532 A&B ARE-2533 A&B)	4	1	1	i	j
c. Automatic Actuation Logic and Actuation Relays	4	1	1	i	j

VOGTLE - UNIT 1

3/4 3-24

Amendment No. 3

TABLE 3.3-2 (Continued)

TABLE NOTATIONS

- a Trip function may be blocked in this MODE below the P-11 (Pressurizer Pressure Interlock) Setpoint.
- b Trip function automatically blocked above P-11 and may be blocked below P-11 when Safety Injection on low steam line pressure is not blocked.
- c During movement of irradiated fuel or movement of loads over irradiated fuel within containment.
- d The provisions of Specification 3.0.4 are not applicable.
- e During movement of irradiated fuel or movement of loads over irradiated fuel.
- f Not applicable if one main steam isolation valve and associated bypass isolation valve per steamline is closed.
- g Containment Ventilation Radiation (RE-2565) is treated as one channel and is considered OPERABLE if the particulate (RE-2565A) and iodine monitors (RE-2565B) are OPERABLE or the noble gas monitor (RE-2565C) is OPERABLE.
- h Manual initiation of Auxiliary Feedwater is accomplished via the pump handswitches.
- i Whenever irradiated fuel is in the storage pool.
- j For actions associated with inoperable instrumentation, follow actions specified in Specification 3.9.12.

ACTION STATEMENTS

- ACTION 14 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours; however, one channel may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.2.1, provided the other channel is OPERABLE.
- ACTION 15 - With the number of OPERABLE channels one less than the Total Number of Channels, operation may proceed until performance of the next required ANALOG CHANNEL OPERATIONAL TEST provided the inoperable channel is placed in the tripped condition within 1 hour.
- ACTION 16 - With the number of OPERABLE channels less than the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.9.9 (Mode 6).

TABLE 3.3-2 (Continued)

ACTION STATEMENTS (Continued)

- ACTION 17 - With the number of OPERABLE channels one less than the Total Number of Channels, operation may proceed provided the inoperable channel is placed in the bypassed condition and the Minimum Channels OPERABLE requirement is met. One additional channel may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.2.1.
- ACTION 18 - With one less than the Minimum Channels OPERABLE requirement, operation may continue provided the containment purge supply and exhaust valves are closed within 24 hours.
- ACTION 19 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- ACTION 20 - With the number of OPERABLE channels one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided the following conditions are satisfied:
- a. The inoperable channel is placed in the tripped condition within 1 hour, and
  - b. The Minimum Channels OPERABLE requirement is met; however, one additional channel may be bypassed for up to 2 hours for surveillance testing of other channels per Specification 4.3.2.1.
- ACTION 21 - With the number of OPERABLE Channels less than the Minimum Channels OPERABLE requirement, within 1 hour determine by observation of the associated permissive status light(s) that the interlock is in its required state for the existing plant condition, or apply Specification 3.0.3.
- ACTION 22 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, be in at least HOT STANDBY within 6 hours and in at least HOT SHUTDOWN within the following 6 hours; however, one channel may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.2.1 provided the other channel is OPERABLE.
- ACTION 23 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within 6 hours and in at least HOT SHUTDOWN within the following 6 hours.

TABLE 3.3-2 (Continued)

ACTION STATEMENTS (Continued)

- ACTION 24 - With the number of OPERABLE channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or declare the associated valve inoperable and take the ACTION required by Specification 3.7.1.5.
- ACTION 25 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, be in at least HOT STANDBY within 6 hours; however, one channel may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.2.1 provided the other channel is OPERABLE.
- ACTION 26 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 7 days or within the next 6 hours initiate and maintain operation of the Control Room Emergency Ventilation System in the Emergency mode. With two channels inoperable, within 1 hour initiate and maintain operation of the Control Room Emergency Ventilation System in the Emergency mode.
- ACTION 27 - Not Used.
- ACTION 28 - Not Used.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 3 TO FACILITY OPERATING LICENSE NPF-68

GEORGIA POWER COMPANY, ET AL

DOCKET NO. 50-424

VOGTLE ELECTRIC GENERATING PLANT, UNIT 1

INTRODUCTION

By letter dated August 26, 1987, Georgia Power Company, et al., (the licensee) requested a change to the Technical Specifications for Vogtle Electric Generating Plant, Unit 1. The proposed change would revise the action requirement for inoperable Fuel Handling Building Post-Accident Ventilation System (FHBPAVS) actuation instrumentation (Functional Unit 11 of Table 3.3-2).

EVALUATION

The proposed change achieves consistency within the Technical Specifications between Table 3.3-2 and specification 3.9.12 and does so in a conservative manner by deleting the option of potentially performing operations involving movement of fuel within the storage pool or crane operations with loads over the storage pool for up to 7 days with either no manual or automatic actuation of the FHBPAVS. The proposed change modifies the action statement to directly invoke specification 3.9.12 thereby prohibiting such operations until at least one FHBPAVS is operable. This change provides added conservatism to the Vogtle Technical Specifications by replacing an action statement which allows all FHBPAVS instrumentation to be inoperable for up to 7 days with one requiring immediate, appropriate corrective action if any FHBPAVS instrumentation is inoperable and, in turn, renders a corresponding train of the FHBPAVS inoperable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the use of facility components located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types of any effluents that may be released off-site and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there have been no public comments on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

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CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (52 FR 37546) on October 7, 1987, and consulted with the state of Georgia. No public comments were received and the state of Georgia did not have any comments.

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: Melanie Miller, DRPI/II/PDII-3  
Fred Burrows, SICB

Dated: November 20, 1987