August 3, 1988

Docket No. 50-424

Mr. W. G. Hairston, III Senior Vice President -Nuclear Operations Georgia Power Company P.O. Box 4545 Atlanta, Georgia 30302

Dear Mr. Hairston:

SUBJECT: ISSUANCE OF AMENDMENT NO. 8 TO FACILITY OPERATING LICENSE NPF-68 VOGTLE ELECTRIC GENERATING PLANT, UNIT 1 (TAC 67142)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 8 to Facility Operating License No. NPF-68 for the Vogtle Electric Generating Plant, Unit 1. The amendment is being issued in response to your letter dated February 4, 1988.

The amendment modified the Technical Specifications by deleting the control building sump effluent line radiation monitor, RE-17646. The amendment is effective as of its date of issuance.

A copy of the related safety evaluation supporting Amendment No. 8 to Facility Operating License NPF-68 is enclosed.

Notice of issuance of the amendment will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

Jon B. Hopkins, Project Manager Project Directorate II-3 Division of Reactor Projects - I/II

Enclosures: 1. Amendment No. ⁸ to NPF-68 2. Safety Evaluation

cc w/enclosures: See next page



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Mr. W. G. Hairston, III Georgia Power Company

cc: Mr. L. T. Gucwa Manager of Safety and Licensing Georgia Power Company P.O. Box 4545 Atlanta, Georgia 30302

Mr. Ruble A. Thomas Executive Consultant Southern Company Services, Inc. P.O. Box 2625 Birmingham, Alabama 35202

Mr. Paul D. Rice Vice President & Project Director Georgia Power Company Post Office Box 282 Waynesboro, Georgia 30830

Mr. J. A. Bailey Project Licensing Manager Southern Company Services, Inc. P.O. Box 2625 Birmingham, Alabama 35202

Ernest L. Blake, Jr. Bruce W. Churchill, Esq. Shaw, Pittman, Potts and Trowbridge 2300 N Street, N. W. Washington, D. C. 20037

Mr. G. Bockhold, Jr. General Manager Nuclear Operations Georgia Power Company P.O. Box 1600 Waynesboro, Georgia 30830

Regional Administrator, Region II U.S. Nuclear Regulatory Commission 101 Marietta Street, N.W., Suite 2900 Atlanta, Georgia 30323 Vogtle Electric Generating Plant

Resident Inspector Nuclear Regulatory Commission P.O. Box 572 Waynesboro, Georgia 30830 Deppish Kirkland, III, Counsel Office of the Consumers' Utility Council Suite 225 32 Peachtree Street, N.E. Atlanta, Georgia 30302 James E. Joiner Troutman, Sanders, Lockerman,

& Ashmore 1400 Candler Building 127 Peachtree Street, N.E. Atlanta, Georgia 30303

Danny Feig 1130 Alta Avenue Atlanta, Georgia 30307

Carol Stangler Georgians Against Nuclear Energy 425 Euclid Terrace Atlanta, Georgia 30307

Mr. R. P. McDonald Executive Vice President -Nuclear Operations Georgia Power Company P.O. Box 4545 Atlanta, Georgia 30302



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

VOGTLE ELECTRIC GENERATING PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 8 License No. NPF-68

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Vogtle Electric Generating Plant, Unit 1 (the facility) Facility Operating License No. NPF-68 filed by the Georgia Power Company acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensees) dated February 4, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.



- 2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-68 is hereby amended to read as follows:
 - (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. ⁸, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. GPC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By:

David B. Matthews, Director Project Directorate II-3 Division of Reactor Projects

Attachment: Technical Specification Changes

Date of Issuance: August 3, 1988

OFFICIAL RECORD COPY LA:PDII-3 MR06d 7/27/88 7/27/88 7/27/88





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ATTACHMENT TO LICENSE AMENDMENT NO. 8

FACILITY OPERATING LICENSE NO. NPF-68

DOCKET NO. 50-424

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

Amended Page	<u>Overleaf</u> Page
3/4 3-66	3/4 3-65
3/4 3-68	3/4 3-67

INSTRUMENTATION

RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.9 The radioactive liquid effluent monitoring instrumentation channels shown in Table 3.3-9 shall be OPERABLE with their Alarm/Trip Setpoints set to ensure that the limits of Specification 3.11.1.1 are not exceeded. The Alarm/ Trip Setpoints of these channels shall be determined and adjusted in accordance with the methodology and parameters in the OFFSITE DOSE CALCULATION MANUAL (ODCM).

APPLICABILITY: At all times.

ACTION:

- a. With a radioactive liquid effluent monitoring instrumentation channel Alarm/Trip Setpoint less conservative than required by the above specification, immediately suspend the release of radioactive liquid effluents monitored by the affected channel, or declare the channel inoperable.
- b. With less than the minimum number of radioactive liquid effluent monitoring instrumentation channels OPERABLE, take the ACTION shown in Table 3.3-9. Restore the inoperable instrumentation to OPERABLE status within 30 days and, if unsuccessful, explain in the next Semiannual Radioactive Effluent Release Report pursuant to Specification 6.8.1.4 why this inoperability was not corrected in a timely manner.
- c. The provisions of Specifications 3.0.3 and 3.0.4, are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.9 Each radioactive liquid effluent monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK, SOURCE CHECK, CHANNEL CALIBRATION, and ANALOG CHANNEL OPERATIONAL TEST at the frequencies shown in Table 4.3-5.

TABLE 3.3-9

RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION

		INSTRUMENT	MINIMUM CHANNELS OPERABLE	ACTION
1.	Rad A	lioactivity Monitors Providing Alarm and utomatic Termination of Release		
	a.	Liquid Radwaste Effluent Line (RE-0018)	1	37
	b.	Steam Generator Blowdown Effluent Line (RE-0021)	1	38
	c.	Turbine Building (Floor Drains) Sumps Effluent Line	(RE-0848) 1	38
2.	Rad A	ioactivity Monitors Providing Alarm But Not Providing utomatic Termination of Release		
	a.	Nuclear Service Cooling Water System Effluent Line (RE-0020 A & B)	1	39
3.	Flo	w Rate Measurement Devices		
	a.	Liquid Radwaste Effluent Line (FT-0018)	1	40
	b.	Steam Generator Blowdown Effluent Line (FT-0021)	1	40
	c.	Flow to Blowdown Sump (AFQI-7620, FR-7620, pen 1)	1	40

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TABLE 3.3-9 (Continued)

ACTION STATEMENTS

- ACTION 37 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided that prior to initiating a release:
 - a. At least two independent samples are analyzed in accordance with Specification 4.11.1.1, and
 - b. At least two technically qualified members of the facility staff independently verify the release rate calculations and discharge line valving.

Otherwise, suspend release of radioactive effluents via this pathway.

- ACTION 38 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided grab samples are analyzed for radioactivity at a lower limit of detection of no more than 10-7 microcurie/ml:
 - a. At least once per 12 hours when the specific activity of the secondary coolant is greater than 0.01 microcurie/gram DOSE EQUIVALENT I-131, or
 - b. At least once per 24 hours when the specific activity of the secondary coolant is less than or equal to 0.01 microcurie/gram DOSE EQUIVALENT I-131.
- ACTION 39 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided that, at least once per 12 hours, grab samples are collected and analyzed for radioactivity at a lower limit of detection of no more than 10^{-7} microcurie/m].
- ACTION 40 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided the flow rate is estimated at least once per 4 hours during actual releases. Pump performance curves generated in place may be used to estimate flow.

TABLE 4.3-5

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RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

INST	RUME	<u>NT</u>	CHANNEL CHECK	SOURCE <u>CHECK</u>	CHANNEL CALIBRATION	ANALOG CHANNEL OPERATIONAL <u>TEST</u>
1.	Rad A o	ioactivity Monitors Providing larm and Automatic Termination f Release				
	a.	Liquid Radwaste Effluent Line (RE-0018)	D	Р	R(3)	Q(1)
	b.	Steam Generator Blowdown Effluent Line (RE-0021)	D	м	R(3)	Q(1)
	c.	Turbine Building (Floor Drains) Sumps Effluent Line (RE-0848)	D	м	R(3)	Q(1)
2.	Rad No	ioactivity Monitors Providing Alarm But ot Providing Automatic Termination f Release				
	N E	uclear Service Cooling Water System ffluent Line (RE-0020 A & B)	D	М	R(3)	Q(2)

VOGTLE - UNIT 1



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 8 TO FACILITY OPERATING LICENSE NPF-68

GEORGIA POWER COMPANY, ET AL.

DOCKET NO. 50-424

VOGTLE ELECTRIC GENERATING PLANT, UNIT 1

1.0 INTRODUCTION

By letter dated February 4, 1988, Georgia Power Company, et al., (the licensee) requested a change to the Technical Specifications for Vogtle Electric Generating Plant, (VEGP), Unit 1. The proposed change would change the Technical Specifications (TS) by deleting the control building sump effluent line radiation monitor, RE-17646.

2.0 EVALUATION

The proposed amendment deletes monitor RE-17646 from TS Table 3.3-9, "Radioactive Liquid Effluent Monitoring Instrumentation," and from TS Table 4.3-5, "Radioactive Liquid Effluent Monitoring Instrumentation Surveillance Requirements."

In support of this change, the licensee has provided the following information:

Radiation monitor RE-17646 is a part of the Process and Effluent Radiation Monitoring System (PERMS) and functions to minimize the release of radioactive liquid effluent from the control building sump. As a result of a recent piping modification, this function can now be performed by another PERMS monitor.

In the original piping configuration, the normally non-radioactive discharge from the control building sump was monitored by RE-17646 then tied in to the turbine building drain header at a point downstream of radiation monitor RE-0848. A high radiation signal from RE-17646 would cause isolation of control building sump discharge flow into the turbine building drain header.

The piping has been modified so that the control building sump discharge now ties in to the turbine building drain header upstream of RE-0848. Detection of high radiation by RE-0848 causes automatic diversion of drain header flow from its normal flow path (to the cily waste separator, then the waste retention basin, and then the blowdown sump) to the "dirty" turbine building drain tank.

Monitor RE-17646 is not necessary for the mitigation of any accident analyzed for Plant Vogtle. Since RE-0848 now provides for minimizing radioactive discharge from the control building sump, RE-17646 can be deleted with no safety consequences. Georgia Power Company is proposing to delete RE-17646 to reduce the manpower commitment and expense required for maintenance and surveillance.

8808110205 880803 PDR ADUCK 05000424 PNU The NRC staff has reviewed the above information and finds that monitor RE-0848 will function adequately to minimize radioactive discharge from the control building sump. Therefore, monitor RE-17646 is no longer needed, and the NRC staff concludes that this change is acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in a requirement with respect to installation of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational exposure. The NRC staff has made a determination that the amendment involves no significant hazards consideration, and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the <u>Federal Register</u> on April 20, 1988, (53 FR 13016), and consulted with the state of Georgia. No public comments were received, and the state of Georgia did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Jon B. Hopkins, PDII-3/DRP-I/II

Dated: August 3, 1988

August 3, 1988 DATED:

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EButcher

GPA/PA ARM/LFMB

DHagan BGrimes

JMiller

JCunningham

AMENDMENT NO.	8	TO FACILITY OPERATING LICENSE NPF-68 - Vogtle Electric Generating Plant, Unit 1
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