

JUL 24 1981

Docket Nos.: 50-424
and 50-425

Mr. W. E. Ehrensperger
Senior Vice President Power Supply
Georgia Power Company
Post Office Box 4545
Atlanta, Georgia 30302

Dear Mr. Ehrensperger:

Subject: Amendment to Construction Permits for Alvin W. Vogtle Nuclear
Plant, Unit Nos. 1 and 2

Your letter, dated December 19, 1980, transmitted an application for amendments to the Alvin W. Vogtle Nuclear Plant, Units 1 and 2 Construction Permits CPPR-108 and CPPR-109. The amendment to the construction permits was requested to remove the enclosure building and its related equipment from the plant design and replace it with an equipment building from grade to the 270 foot-level and a more restrictive containment leak rate. You had previously submitted Supplement No. 6 to the application for Construction Permit and Operating License, dated August 21, 1979, which included a description of the associated physical modifications. To further support this request for amendment, you submitted Supplement No. 8 to the application for Construction Permit and Operating License, dated December 30, 1980, which contained additional information on the more restrictive primary containment leak rate and the resultant projected offsite doses for a design basis accident if the enclosure building and its related equipment were removed.

We have reviewed your application and have concluded that the proposed modified design is acceptable and that this action does not constitute an unreasonable risk to the health and safety of the public, and is not inimical to the common defense and security. The bases for these conclusions are set forth in the enclosed Safety Evaluation.

We have also concluded that there will be no environmental impact attributable to the proposed action that was not considered in our Final Environmental Statement, and that therefore, no environmental impact statement need be prepared for the proposed action. The bases for these conclusions are set forth in the enclosed Environmental Impact Appraisal. Also enclosed is the applicable Negative Declaration.

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Mr. W. E. Ehrensperger

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Enclosed are Amendment No. 2 to CPPR-108 and Amendment No. 2 to CPPR-109 for the Alvin W. Vogtle Nuclear Plant, Units 1 and 2 which reflect the changes discussed above and a copy of a related notice which has been forwarded to the Office of the Federal Register for publication.

Sincerely,

Original signed by
Darrell G. Eisenhut
Darrell G. Eisenhut, Director
Division of Licensing
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment 2 to CPPR-108
- 2. Amendment 2 to CPPR-109
- 3. Safety Evaluation
- 4. Negative Declaration
- 5. Environmental Impact Appraisal
- 6. Federal Register Notice

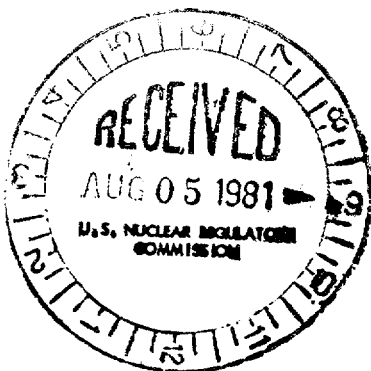
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Concurrence
based on
comment

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SURNAME	MRushbrook/1g	JGrant		BJYoung/food	RLTedesco	DEisenhut
DATE	7/7/81	7/8/81	7/9/81	7/10/81	7/12/81	7/24/81

DISTRIBUTION FOR CONSTRUCTION AMENDMENT FOR VOGTLE UNITS 1 & 2 DATED: July 24, 1981

DOCKET FILE (50-424 & 50-425)
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UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-424

GEORGIA POWER COMPANY

OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

ALVIN W. VOGTLE NUCLEAR PLANT, UNIT 1

AMENDMENT TO CONSTRUCTION PERMIT

Amendment No. 2
Construction Permit No. CPPR-108

1. The Nuclear Regulatory Commission (the Commission) having found that:
 - A. The application for amendment by Georgia Power Company, dated December 19, 1980, and supplemented by letter dated December 30, 1980, as previously discussed in a letter dated August 21, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public; and
 - C. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
 - D. Prior public notice of this amendment was published in the Federal Register on February 24, 1981 (46 FR 13865).
2. Accordingly, Construction Permit No. CPPR-108 is amended to reflect a change in the plant design as follows:

Revise paragraph 3.C. to read:

3.C. This construction permit authorizes the applicant to construct the facility described in the application and the hearing record, as amended through Amendment No. 2 in accordance with the principal architectural and engineering criteria and environmental protection commitments set forth therein.

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Add a new paragraph 3.D.:

3.D. The replacement of the enclosure building with an equipment building from grade to the 270 foot-level and a more restrictive containment leak rate is acceptable.

3. This amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Darrell G. Eisenhut, Director
Division of Licensing
Office of Nuclear Reactor Regulation

Date of Issuance: JUL 24 1981

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SURNAME	MRushbrook/g	JG...		BJYoungblood	RLTedesco	DEisenhut
DATE	7/7/81	7/8/81	7/9/81	7/10/81	7/12/81	7/14/81

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-425

GEORGIA POWER COMPANY

OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

ALVIN W. VOGTLE NUCLEAR PLANT, UNIT 2

AMENDMENT TO CONSTRUCTION PERMIT

Amendment No. 2
Construction Permit No. CPPR-109

1. The Nuclear Regulatory Commission (the Commission) having found that:
 - A. The application for amendment by Georgia Power Company, dated December 19, 1980, and supplemented by letter dated December 30, 1980, as previously discussed in a letter dated August 21, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public; and
 - C. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
 - D. Prior public notice of this amendment was published in the Federal Register on February 24, 1981 (46 FR 13865).

2. Accordingly, Construction Permit No. CPPR-109 is amended to reflect a change in the plant design as follows:

Revise paragraph 3.C. to read:

- 3.C. This construction permit authorizes the applicant to construct the facility described in the application and the hearing record, as amended through Amendment No. 2 in accordance with the principal architectural and engineering criteria and environmental protection commitments set forth therein.

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Add a new paragraph 3.D.:

3.D. The replacement of the enclosure building with an equipment building from grade to the 270 foot-level and a more restrictive containment leak rate is acceptable.

3. This amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by
Darrell G. Eisenhut

Darrell G. Eisenhut, Director
Division of Licensing
Office of Nuclear Reactor Regulation

Date of Issuance: JUL 24 1981

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SURNAME	MRushbrook/Lg	JGrant		BYoungblood	RLTedesco	DKEisenhut
DATE	7/7/81	7/8/81	7/7/81	7/10/81	7/12/81	7/28/81

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SAFETY EVALUATION REPORT BY THE
OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 2
TO CONSTRUCTION PERMITS CPPR-108 AND CPPR-109

Introduction

On December 19, 1980, Georgia Power Company, acting on its own behalf and agent for Oglethorpe Electric Membership Corporation, Municipal Electric Authority of Georgia, and the City of Dalton, requested an amendment to the Construction Permits CPPR-108 and CPPR-109, for the Alvin W. Vogtle Nuclear Plant, Units 1 and 2, to reflect a modification in plant design. The modification would remove the enclosure building in the application for Vogtle licenses and add as a substitute a steel-framed, metal-siding equipment building from grade to the 270-foot level, and add a more restrictive primary containment leakage rate of 0.2 weight percent (%) per day. These substitutions, together with onsite meteorological data, will ensure that offsite post-accident doses are less than doses judged to be acceptable for design and are lower than those calculated at the time construction was initially authorized.

In support of this application for amendment, Georgia Power Company had previously submitted Supplement No. 6, dated August 21, 1979, to its application for Construction Permit and Operating License, which included a description of the associated physical modifications. These modifications would entail removal of the enclosure building and its related equipment as described in Sections 1.2.6, 3.8.4.1.1, and 6.6 of the Vogtle PSAR, while retaining from grade to the 270-foot level, an equipment building described in Sections 1.2.6 and 3.8.4.1.1 of Supplement No. 6.

To further support this request for amendment, Georgia Power Company has submitted Supplement No. 8 to the application, dated December 30, 1980, which contains additional information on the more restrictive primary containment leak rate and the resultant projected offsite doses for a design basis accident if the enclosure building and its related equipment were removed. Georgia Power Company has committed to a containment leak rate of 0.2% per day with the proposed modified enclosure building design. Employing this leakage rate, dispersion factors using NRC methodology and the latest onsite meteorological data, Georgia Power Company has submitted revised design basis accident offsite doses for the Vogtle Units.

Based on the information submitted in the Preliminary Safety Analysis Report (PSAR) and Supplement Nos. 6 and 8 to the PSAR, the NRC staff has completed its review of all safety-significant matters related to the issuance of the construction permit amendment as requested in the December 19, 1980 application. This Safety Evaluation is issued in support of Amendment No. 2 to Construction Permits CPPR-108 and CPPR-109, allowing the removal of the enclosure building and related equipment.

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The purpose of this Safety Evaluation is to examine the impact of the proposed modifications to the enclosure building for Vogtle Units 1 and 2. Specifically, the Safety Evaluation addresses the following safety significant items:

1. §3.0. Design criteria for structures, components, equipment and systems,
2. §6.0. Engineered safety features (containment systems), and
3. §15.0. Accident analyses.

Evaluation

We have reviewed this application for amendment submitted on December 19, 1980, and supplemented by letters of August 21, 1979, and December 30, 1980. Our review of safety-related matters and our conclusions concerning each item are described in the following subsections of this evaluation report.

Date: JUL 24 1981

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§3.0 DESIGN CRITERIA FOR STRUCTURES, COMPONENTS, EQUIPMENT AND SYSTEMS

We have reviewed the structural aspects pertaining to modifications of the enclosure building in relation to the primary containment integrity. We find that our prior reviews in Sections 3.3, 3.5, 3.7 and 3.8 of the Safety Evaluation Report are not affected by the removal of the enclosure building and its replacement with an equipment building from grade to the 270-foot level based on the following:

1. there is no change in the design methods and design criteria of the containment or other structures due to the enclosure building modifications. No structural credit has been taken for the enclosure building in either the original or modified design case;
2. the enclosure building modifications do not affect the plant's susceptibility to tornado missiles since it is assumed that the metal siding of the enclosure building afforded no protection from missiles;
3. the dynamic response characteristics of the containment building under seismic loads would essentially be unaltered because the enclosure building modifications result in only 2% reduction in weight and a very small change in stiffness of the overall containment building.

Based on the foregoing, we conclude that the safety margin of the original structural design or containment integrity is not significantly affected; therefore, the proposed design modification is acceptable.

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§6.0 ENGINEERED SAFETY FEATURES

6.2 CONTAINMENT SYSTEMS

6.2.1 Containment Functional Design

In Supplement No. 6 to the Vogtle 1 & 2 PSAR, dated August 21, 1979, the Georgia Power Company revised its analysis of the radiological consequences of a postulated loss-of-coolant accident (LOCA) using assumptions that did not take credit for an enclosure building. This revised analysis is associated with the deletion of the enclosure building and its replacement with an equipment building from grade to the 270-foot level and the more restrictive containment leak rate of 0.2% per day.

Because the enclosure building was designed only to treat leakages from the containment building, the functional performance of the containment building in the event of a LOCA is not affected by the presence or absence of the enclosure building. We conclude that since the removal of the enclosure building will have no effect on our prior review of the containment system, the removal of the enclosure building is acceptable with respect to containment functional design.

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§15.0 ACCIDENT ANALYSES

15.1 General

We have reviewed additional meteorological information submitted by the applicant on December 7, 1979, to determine the potential offsite doses calculated for the postulated loss-of-coolant accident (LOCA) and the fuel handling accident. The calculated doses are presented in Table 15.1 and the assumptions used are discussed in §15.2. A quantitative comparison of LOCA doses for the original and modified designs is presented in Table 15.2. All potential doses calculated by the applicant and by the staff for both the original and modified design cases are within the 10 CFR Part 100 guideline values. We conclude that the enclosure building modifications are acceptable.

TABLE 15.1

CALCULATED DOSES DUE TO DESIGN BASIS ACCIDENTS

	<u>Exclusion Radius</u> (1098 meters) 0-2 hours		<u>Low Population Zone</u> (3220 meters) 0-30 days	
	<u>Thyroid</u>	<u>Whole Body</u>	<u>Thyroid</u>	<u>Whole Body</u>
Loss-of-coolant (rem)	98	2.6	69	1.3
Fuel handling (rem)	8	0.7	1.3	0.2

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TABLE 15.2

COMPARISON OF LOCA DOSES FOR THE VOGTLE NUCLEAR STATION

	LOCA Doses per SER (March 1974) with Enclosure Building Primary Con. Leak Rate of 0.3%/day	LOCA Doses without Enclosure Building 0.2%/day Leak Rate & X/Q Values as of 1/80
Exclusion Area Boundary		
Thyroid (rem)	122	98
Whole body (rem)	7	2.6
Low Population Zone Boundary		
Thyroid (rem)	70	69
Whole body	8	1.3

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15.2 DESIGN BASIS ACCIDENT ASSUMPTIONS

15.2.1 Loss of Coolant Accident

We have reviewed the information supplied by the applicant in Supplement 8 to the PSAR. Based upon our review of this information, we have modelled the primary containment as two control volumes, consisting of the region covered by the containment spray and the remaining unsprayed containment volume. The spray region was assumed to be 78 percent of the total containment free volume of 2.62×10^{-6} cubic feet.

The staff also assumed that a 30 second delay exists from the initiation of the accident to the time spray injection into the containment begins. Further, the sprays, enhanced by the addition of sodium hydroxide, were assumed to operate until a reduction factor (DF) of 100 was reached in the sprayed region; after which the sprays did not remove any additional radioiodine. The single containment was assumed to leak at a design leak rate of 0.2% per day for the first 24 hours and at 0.1% per day thereafter.

The meteorological conditions and other important parameters used in our analysis of the consequences of a loss of coolant accident are tabulated in Table 15.3 and 15.4, and the calculated doses are given in Table 15.1.

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TABLE 15.3

ASSUMPTIONS USED TO ESTIMATE
RADIOLOGICAL CONSEQUENCES DUE TO A
POSTULATED LOSS OF COOLANT ACCIDENT
AT VOGTLE UNITS 1 & 2

Power level, megawatts thermal	3565
Operating time, years	3
Primary containment leak rate, percent per day	0.2 to 24 hours 0.1 greater than 24 hours
Fraction of Core Inventory Available for Leakage from Containment, percent:	
Noble Gases	100
Iodine	25
Primary Containment Free Volume, cubic feet	2.75×10^6
Iodine form fractions, percent	
Elemental	91
Organic	4
Particulate	5
Spray removal rates, per hour	
Elemental	10
Particulate	0.45
Fraction of primary containment unsprayed, percent	22
Relative concentrations, second per cubic meter	
0-2 hours at 1060 meters	1.8×10^{-4}
0-8 hours at 3218 meters	3.3×10^{-5}
8-24 hours at 3218 meters	2.2×10^{-5}
24-96 hours at 3218 meters	9.2×10^{-6}
96-720 hours at 3218 meters	2.7×10^{-6}

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TABLE 15.4

Refueling Accident Calculation Input Parameters

Shutdown Time	100 hours
Total Number of Fuel Rods in the Core	50,952
Number of Fuel Rods Involved in the Refueling Accident	264
Power Peaking Factor	1.65
Iodine Fractions Released from Pool	
Elemental	75%
Organic	25%
Filter Efficiencies	
Elemental	90%
Organic	70%
<u>X/Q Values, sec/m</u>	
0-2 hours @ 1098 meters	1.8 x 10
0-2 hours @ 3220 meters	3.3 x 10

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Conclusion

We find the enclosure building modifications acceptable for the following reasons:

- 1) the safety margin of the original structural design or containment integrity is not significantly affected;
- 2) the functional performance of the containment building in the event of a LOCA is not affected by the presence or absence of the enclosure building; and
- 3) the calculated dose consequences are less than the guideline values of 10 CFR Part 100 in both the original and modified design cases.

We have concluded, based on the considerations discussed above, that with respect to the facility design changes authorized by Amendment Nos. 2 to Construction Permits CPPR-108 and CPPR-109, (1) there is reasonable assurance, taking into consideration the criteria contained in 10 CFR Part 100, that the proposed facilities can be constructed and operated at the proposed location without undue risk to the health and safety of the public, and (2) the issuance of these Amendments will not be inimical to the common defense and security or to the health and safety of the public.

Approval of this action does not preclude future changes as deemed necessary by accident analyses on the mitigation of more severe accident sequences than postulated design basis accidents which will be performed during the operating license review for the entire Vogtle plant.

NEGATIVE DECLARATION

SUPPORTING AMENDMENT NO. 2 TO CPPR-108 and AMENDMENT NO. 2 TO CPPR-109

RELATING TO THE ENCLOSURE BUILDING MODIFICATIONS

ALVIN W. VOTGLE NUCLEAR PLANT, UNIT NOS. 1 AND 2

GEORGIA POWER COMPANY

OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NOS. 50-424 AND 50-425

The U. S. Nuclear Regulatory Commission (the Commission) has reviewed the amendments to Construction Permits CPPR-108 and CPPR-109 relating to the enclosure building modifications at the Alvin W. Vogtle Nuclear Plant, Unit Nos. 1 and 2. The amendments would delete the enclosure building and its related equipment and replace it with an equipment building from grade to the 270-foot level and commit Georgia Power Company to a more restrictive containment leak rate. In accordance with 10 CFR Part 51, the Commission's Division of Licensing has prepared an Environmental Impact Appraisal (EIA) for the amendment. Based on the EIA the Commission has concluded that an environmental impact statement for this action is not warranted because there will be no adverse environmental impacts affecting the quality of the human environment, attributable to the proposed action, that would be in addition to those impacts evaluated in the Commission's Final Environmental Statement for Alvin W. Vogtle Nuclear Plant, Unit 1 and 2, issued in March 1974. A negative declaration is, therefore, appropriate.

The EIA is available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., 20555 and at the local public document room located at the Burke County Public Library, Fourth Street,

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Waynesboro, Georgia 30830. A copy of the EIA may be obtained upon request, addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C., 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 24th day of July, 1981.

FOR THE NUCLEAR REAGULATORY COMMISSION

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B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing

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JUL 24 1981

ENVIRONMENTAL IMPACT APPRAISAL

SUPPORTING AMENDMENT NO. 2 TO CPPR-108 AND AMENDMENT NO. 2 TO CPPR-109

RELATING TO THE ENCLOSURE BUILDING MODIFICATIONS

ALVIN W. VOGTLE NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-424 AND 50-425

Description of the Proposed Action

By letter dated December 19, 1980, Georgia Power Company filed a request with the Nuclear Regulatory Commission (the Commission) to reflect a modification in plant design. The action proposed by the permittee is the issuance of amendments to Construction Permits CPPR-108 and CPPR-109 that would substitute for the enclosure building an equipment building from grade to the 270-foot level as well as a more restrictive primary containment leakage commitment which, together with onsite meteorological data, would ensure that offsite post-accident radiological doses are less than doses judged to be acceptable for design now and at the time construction was initially authorized.

The staff's Final Environmental Statement (FES) relating to the Construction of Vogtle Units 1 and 2 was published in March 1974.

Environmental Impact of the Proposed Action

The Commission has reviewed the proposed enclosure building modifications for Alvin W. Vogtle Nuclear Plant, Units 1 and 2, to determine the possible radiological and nonradiological consequences. Evaluation of the possible consequences follows:

The applicant has submitted by a letter dated December 7, 1979, additional meteorological data. Employing these latest data and a minimum leak rate of 0.2% per day, we have found that the radiological dose consequences for the modified design are less than the results for the original plant design. The calculated dose consequences (Tables 15.1 and 15.2 of the Safety Evaluation Report) are less than the guidelines of 10 CFR Part 100 in both the original and modified design cases. Therefore, we conclude that the Final Environmental Statement issued March 1974, remains valid with regard to radiological environmental impacts.

In a letter dated December 19, 1980, the applicant has submitted further information concerning nonradiological impacts on both plant construction and operation due to the enclosure building modifications. According to our evaluation, the only impacts on construction would be a savings in construction man-hours and resource commitments. The physical area affected by the construction activity would be the same for either the original or modified plant design. For plant operation, there would be no change from the original design in types or amounts of nonradiological effluents.

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Therefore, we conclude that the Final Environmental Statement, issued March 1974, remains valid with regard to nonradiological environmental impacts of plant construction and operation.

Approval of this action does not preclude future changes as deemed necessary by accident analyses on the mitigation of more severe accident sequences than postulated design basis accidents which will be performed during the operating license review for the entire Vogtle plant.

Conclusion and Basis for Negative Declaration

On the basis of the foregoing information, the NRC concludes that there will be no environmental impacts resulting from the proposed action in addition to those impacts already predicted and impacted in the Commission's Final Environmental Statement, issued in March 1974. Having reached this conclusion, the staff has further concluded that no environmental impact statement for the proposed action need be prepared, and that a negative declaration to this effect is appropriate.

Dated: JUL 24 1981

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UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-424 AND 50-425

GEORGIA POWER COMPANY

OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

NOTICE OF ISSUANCE OF AMENDMENTS TO CONSTRUCTION PERMITS

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 2 to Construction Permit No. CPPR-108 and Amendment No. 2 to Construction Permit No. CPPR-109. The amendment reflects the change in plant design - the enclosure building modifications. Georgia Power Company has sole responsibility for the design construction, and operation of the facilities, which are located in Burke County, Georgia. The amendments are effective as of the date of issuance.

The amendment permits the replacement of the enclosure building with an equipment building. Notice of Proposed Issuance of Amendments to Construction Permits CPPR-108 and CPPR-109 was published in the Federal Register on February 24, 1981 (46 FR 13865). No request for a hearing or petition for leave to intervene was filed following notice of the proposed action.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the amendments.

In connection with the issuance of these amendments, the Commission has issued a Negative Declaration and Environmental Impact Appraisal.

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For further details with respect to the action, see (1) the application for amendment dated December 19, 1980, and supplementary information dated August 21, 1979, and December 30, 1980, (2) Amendment No. 2 to Construction Permit Nos. CPPR-108 and CPPR-109, (3) the Commission's related Safety Evaluation, (4) the Environmental Impact Appraisal and (5) the Negative Declaration supporting the amendments to the construction permits. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. 20555, and at the Burke County Public Library, Fourth Street, Waynesboro, Georgia 30830. In addition, a copy of items (2), (3), (4), and (5) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing, Office of Nuclear Reactor Regulation.

Dated at Bethesda, Maryland this *24th* day of July 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

15/

B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing

McGovern
*Notice of amendment
concerns based
on comment (attached)*
noted on Notice

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SURNAME	MRushbrook/1	gJGrant		BJYoungblood			
DATE	7/7/81	7/8/81	7/9/81	7/10/81			