

Docket No. 50-395

Mr. O. S. Bradham
Vice President, Nuclear Operations
South Carolina Electric & Gas Company
Virgil C. Summer Nuclear Station
P.O. Box 88
Jenkinsville, South Carolina 29065

SUBJECT: ISSUANCE OF AMENDMENT NO. 85 TO FACILITY OPERATING LICENSE
NO. NPF-12 - VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1,
REGARDING ACCIDENT MONITORING INSTRUMENTS (TAC NO. 72839)

Your November 20, 1989 submittal requested a revision to Technical Specification (TS) 3/4.3.3.6, Accident Monitoring Instruments. This revision would delete Item 14, Reactor Building Level, from Tables 3.3-10 and 4.3-7 and modify Item 13 of these same tables to read "Reactor Building/RHR Sump Level." This amendment approves this change and the proposed instrumentation modification to carryout this change.

Sincerely,

Original Signed By:

John J. Hayes, Jr., Project Manager
Project Directorate II-1
Division of Reactor Projects I/II

1. Amendment No. 85 to NPF-12
2. Safety Evaluation

cc w/enclosures:
See next page

OFC :LA:PAII-1:DRPR:PM:PDII-1:DRPR:D:PDII-1:
 NAME :PAnderson :JHayes:jd :EAdensam
 DATE :1/24/90 :12/5/90 :2/12/90

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Document Name: SUMMER AMEND TAC 72839

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AMENDMENT NO. 85 TO FACILITY OPERATING LICENSE NO. NPF-12 - SUMMER, UNIT 1
DATED:

Docket File

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Virgil C. Summer Nuclear Station

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SOUTH CAROLINA ELECTRIC & GAS COMPANY

SOUTH CAROLINA PUBLIC SERVICE AUTHORITY

DOCKET NO. 50-395

VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 85
License No. NPF-12

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by South Carolina Electric & Gas Company (the licensees), dated November 20, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-12 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 85 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. South Carolina Electric & Gas Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective as of its date of issuance, and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

E.G. Tourigny/for

Elinor G. Adensam, Director
Project Directorate II-1
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 12, 1990

OFC	: LA: PD21: DRPR: PM: PD21: DRPR	DEST: SICB	OGC	: D: PD21: DRPR	:	:
NAME	: PAnderson	: J Hayes	: S Newberry	: R Bachmann	: E Adensam	:
DATE	: 1/24/90	: 1/25/90	: 1/26/90	: 1/30/90	: 2/12/90	:

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w/changes to SE, p. 2

ATTACHMENT TO LICENSE AMENDMENT NO. 85
TO FACILITY OPERATING LICENSE NO. NPF-12
DOCKET NO. 50-395

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. Corresponding overleaf pages are also provided to maintain document completeness.

Remove Pages

3/4 3-58

3/4 3-60

Insert Pages

3/4 3-58

3/4 3-60

TABLE 3.3-10 (Continued)
ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>TOTAL OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>
13. Reactor Building/RHR Sump Level	2	1
14. DELETED.		
15. Condensate Storage Tank Level	2	1
16. Reactor Building Cooling Unit Service Water Flow	2	1
17. Service Water Temperature-Reactor Building Cooling Unit (Inlet and Discharge)	2 pairs	1 pair
18. NaOH Storage Tank Level	2	1
19. Reactor Coolant System Subcooling Margin Monitor	2	1
20. Pressurizer PORV Position Indicator	2/valve*	1/valve*
21. Pressurizer PORV Block Valve Position Indicator	1/valve	1/valve
22. Pressurizer Safety Valve Position Indicator	2/valve	1/valve
23. In-Core Thermocouples	4/core quadrant	2/core quadrant
24. Reactor Vessel Level	2	1

* Not required when the associated block valve is closed per Specification 3.4.4.

TABLE 4.3-7 (continued)

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
12. Reactor Building Temperature	M	R
13. Reactor Building/RHR Sump Level	M	R
14. DELETED.		
15. Condensate Storage Tank Level	M	R
16. Reactor Building Cooling Unit Service Water Flow	M	R
17. Service Water Temperature - Reactor Building Cooling Unit (Inlet and Discharge)	M	R
18. NaOH Storage Tank Level	M	R
19. Reactor Coolant System Subcooling Margin Monitor	M	R
20. Pressurizer PORV Position Indicator	M*	R*
21. Pressurizer PORV Block Valve Position Indicator	M	R
22. Pressurizer Safety Valve Position Indicator	M	R
23. In-Core Thermocouples	M	R
24. Reactor Vessel Level	M	R

* Not required when the associated block valve is closed per Specification 3.4.4.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 85 TO FACILITY OPERATING LICENSE NO. NPF-12

SOUTH CAROLINA ELECTRIC & GAS COMPANY

SOUTH CAROLINA PUBLIC SERVICE AUTHORITY

VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-395

1.0 INTRODUCTION

In a letter dated November 20, 1989, South Carolina Electric & Gas Company (the licensee) submitted a proposed change to Technical Specification (TS) 3/4.3.3.6, Accident Monitoring Instruments, to delete Item 14, "Reactor Building Level," from Tables 3.3-10 and 4.3-7 and to modify Item 13 of these same tables to read "Reactor Building/RHR Sump Level." Currently, Item 13 of these tables reads, "Reactor Building RHR Sump Level."

The proposed change in the TS was requested by the licensee as a part of a planned modification. This modification integrates the function performed by the reactor building RHR sump level instrument and the reactor building water level instrument into a function which is now performed by a new single instrument. Presently, Tables 3.3-10 and 4.3-7 require two channels of reactor building RHR sump level indication and two channels of the reactor building level indication. The proposed modification would involve one instrument reading both the reactor building RHR sump level and the reactor building level. Two channels would still be available for this purpose.

2.0 EVALUATION

Accident monitoring instruments are required by the TS for the purpose of providing sufficient information from selected parameters to monitor and assess variables following an accident and to ensure that the recommendations of Regulatory Guide 1.97 are followed. The monitoring requirements contained in Regulatory Guide 1.97 were instituted as a result of the accident at Three Mile Island, Unit 2.

The licensee proposes to modify their existing accident instrumentation by replacing the existing instruments associated with the reactor building level and the reactor building RHR sump level indication with a new instrument which would combine the functions performed by the existing instruments. The new instrument would provide indication over the range provided by existing instrumentation. The licensee states that one benefit of the proposed modification would be that an instrument would be removed from inside containment thereby decreasing the potential

impact of environmental conditions on the instrument. The licensee also states that the proposed modification would not change the capability of monitoring the reactor building level during accident conditions, nor would it change the ability to determine net positive suction head requirements for RHR during switching to the sumps nor the ability to monitor reactor building level for consideration of non-safety related equipment protection. The licensee states that with this modification the requirements of Regulatory Guide 1.97 are still met.

The staff has reviewed the licensee's submittal against the recommendations of Regulatory Guide 1.97 and agrees that the licensee's proposal does not deviate from the recommendations of the Regulatory Guide. The proposed change does not involve a change in the information currently available from the present accident monitoring instrument since the present instruments are "indication only" instruments and the same function will be performed by the instrument proposed in the modification. In addition, a potentially more reliable situation is created by the location of the instrument outside the reactor building.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in the 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The Commission has issued a "Notice of Consideration of Issuance of Amendment to Facility Operating License and Propose No Significant Hazards Consideration Determination and Opportunity for Hearing" which was published in the FEDERAL REGISTER on January 10, 1990 (55 FR 941) and consulted with the State of South Carolina. No public comments or request for hearing were received, and the State of South Carolina did not have comments.

The staff has concluded, based upon the considerations discussed above, that:

(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Hayes

Dated: February 12, 1990