

April 13, 1988

Docket No. 50-395

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Mr. D. A. Nauman
Vice President, Nuclear Operations
South Carolina Electric & Gas Company
P. O. Box 88 (Mail Code 601)
Jenkinsville, South Carolina 29065

Dear Mr. Nauman:

SUBJECT: REACTOR BUILDING COOLING UNIT TECHNICAL SPECIFICATION
AMENDMENT FOR THE V. C. SUMMER NUCLEAR STATION, UNIT NO. 1
(TAC NO. 64588)

The Commission has issued the enclosed Amendment No. 69 to Facility Operating License No. NPF-12 for the Virgil C. Summer Nuclear Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application dated January 20, 1987, as supplemented February 24, 1987.

The amendment modifies the existing reactor building cooling unit service water outlet valves to be automatic valves. This amendment is effective as of its date of issuance, and shall be implemented prior to startup after the next refueling outage.

A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

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John J. Hayes, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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Enclosure:
As stated

cc: See following page

*See previous concurrence

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| NAME | :PAnderson:ch:JHayes | : | :EAdensam | : | : |
| DATE | : 4/13/88 | : | : 4/13/88 | : | : 4/13/88 |

Mr. D. A. Nauman
South Carolina Electric & Gas Company

Virgil C. Summer Nuclear Station

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SOUTH CAROLINA ELECTRIC & GAS COMPANY

SOUTH CAROLINA PUBLIC SERVICE AUTHORITY

DOCKET NO. 50-395

VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 69
License No. NPF-12

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by South Carolina Electric & Gas Company and South Carolina Public Service Authority (the licensees), dated January 20, 1987, as supplemented February 24, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-12 is hereby amended to read as follows:

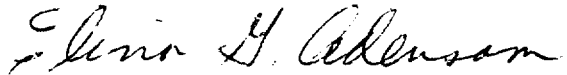
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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 69 , are hereby incorporated in the license. South Carolina Electric & Gas Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective as of its date of issuance, and shall be implemented prior to startup following the next refueling outage.

FOR THE NUCLEAR REGULATORY COMMISSION



Elinor G. Adensam, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 13, 1988

ATTACHMENT TO LICENSE

AMENDMENT NO. 69 TO FACILITY OPERATING LICENSE NO. NPF-12

DOCKET NO. 50-395

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised areas are indicated by marginal lines indicating the areas of change. The corresponding overleaf page is also provided to maintain document completeness.

Remove Pages

3/4 6-13

3/4 6-14

Insert Pages

3/4 6-13 (overleaf)

3/4 6-14

CONTAINMENT SYSTEMS

SPRAY ADDITIVE SYSTEM

LIMITING CONDITION FOR OPERATION

3.6.2.2 The spray additive system shall be OPERABLE with:

- a. A spray additive tank containing a volume of between 3140 and 3230 gallons of between 20.0 and 22.0 percent by weight NaOH solution, and
- b. A flow path capable of adding NaOH solution from the spray additive tank to the suction of each reactor building spray pump.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With the spray additive system inoperable, restore the system to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours; restore the spray additive system to OPERABLE status within the next 48 hours or be in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.2.2 The spray additive system shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each valve (manual, power operated or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position.
- b. At least once per 6 months by:
 1. Verifying the contained solution volume in the tank, and
 2. Verifying the concentration of the NaOH solution by chemical analysis.
- c. At least once per 18 months during shutdown, by verifying that each automatic valve in the flow path actuates to its correct position on a Phase 'A' signal.
- d. At least once per 5 years by verifying each solution flow rate from the following drain connections in the spray additive system:
 1. NaOH Tank to Loop A \geq 15 gpm
 2. NaOH Tank to Loop B \geq 15 gpm

CONTAINMENT SYSTEMS

REACTOR BUILDING COOLING SYSTEM

LIMITING CONDITIONS FOR OPERATION

3.6.2.3 Two independent groups of reactor building cooling units shall be OPERABLE with at least one of two cooling units OPERABLE in slow speed in each group.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

- a. With one group of the above required reactor building cooling units inoperable and both reactor building spray systems OPERABLE, restore the inoperable group of cooling units to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With two groups of the above required reactor building cooling units inoperable, and both reactor building spray systems OPERABLE, restore at least one group of cooling units to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours. Restore both above required groups of cooling units to OPERABLE status within 7 days of initial loss or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. With one group of the above required reactor building cooling units inoperable and one reactor building spray system inoperable, restore the inoperable spray system to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours. Restore the inoperable group of containment cooling units to OPERABLE status within 7 days of initial loss or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.2.3 Each group of reactor building cooling units shall be demonstrated OPERABLE:

- a. At least once per 31 days by:
 1. Starting each cooling unit group from the control room, and verifying that each cooling unit group operates for at least 15 minutes in the slow speed mode.
- b. At least once per 18 months by:
 1. Verifying that each fan group starts automatically on a safety injection test signal.
 2. Verifying a cooling water flow rate of greater than or equal to 2,000 gpm to each cooling unit group.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 69 TO FACILITY OPERATING LICENSE NO. NPF-12

SOUTH CAROLINA ELECTRIC & GAS COMPANY

SOUTH CAROLINA PUBLIC SERVICE AUTHORITY

VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-395

INTRODUCTION

By letter dated January 20, 1987, South Carolina Electric and Gas Company (the licensee) requested a change to the Virgil C. Summer Nuclear Station Reactor Building Cooling System Technical Specification Surveillance Requirement 4.6.2.3.b.2. The proposed change would lower the minimum flow rate to the Reactor Building Cooling Unit (RBCU). The basis for the change is the modification of the RBCU service water outlet valves (XVG-3109 A, B, C, and D) from non-active to active motor-operated valves, with automatic controls to open the valves on the selected post-accident RBCU and close the valves on the non-selected RBCU. This system modification is fully described in the January 20, 1987 application, as supplemented February 24, 1987. The supplemental letter provided additional information that did not change the requested amendment; therefore, this application was not renoticed in the Federal Register.

EVALUATION

The Reactor Building Cooling System is designed to have only RBCU and its associated fan, in each train, in service during a Design Basis Accident (DBA). Since the existing RBCU service water outlet valves are not assumed to be capable of operation following an accident, the service water flows through both the operating and non-operating RBCU. Thus, the present Technical Specification flow requirement is for 200% (4000 gpm) of the required cooling water flow in order to assure that the operating RBCU has sufficient cooling water during an accident situation. The modification will upgrade the RBCU service water outlet valves to environmentally qualified Class IE motor operated valves, with the automatic operation described above. Since these valves are being upgraded to be capable of operation following an accident, there is no need to assure that there is a 200% service water flow to each RBCU.

Based on the foregoing discussion, the staff finds the modification to the RBCU service water outlet valves and the prescribed change to the Virgil C. Summer Technical Specification Surveillance Requirement 4.6.2.3.b.2, to be consistent with the guidelines of Standard Review Plan Sections 6.2.2 and 9.2.1 and is, therefore, acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation of a facility component located within the restricted area as defined in 10 CFR Part 20 and a change to a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration, and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

The Commission made a proposed determination that this amendment involves no significant hazards consideration, which was published in a February 26, 1987 Federal Register Notice (52 FR 5869) and consulted with the State of South Carolina. No public comments were received, and the State of South Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: J. B. Hopkins
R. J. Giardina

Dated: April 13, 1988

TECHNICAL SPECIFICATION AMENDMENT FOR THE V. C. SUMMER NUCLEAR STATION

Docket No. 50-395

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