

Docket No. 50-395

DEC 06 1985

Mr. D. A. Nauman
Vice President - Nuclear Operations
South Carolina Electric & Gas Company
P.O. Box 764
Columbia, South Carolina 29218

Dear Mr. Nauman:

Subject: Amendment No. 47 to Facility Operating License NPF-12

Ms. Elinor Adensam's November 23, 1985, letter forwarded to you
Amendment No. 47 to Facility Operating License NPF-12 related to the
Virgil C. Summer Nuclear Station, Unit No. 1. A page was inadvertently
omitted from the enclosure to that letter. The attached revised
page B 3/4 6-1 should be inserted in Amendment No. 47.

Sincerely,

Original signed by
L. S. Rubenstein

Lester S. Rubenstein, Director
PWR Project Directorate #2
Division of PWR Licensing-A

Enclosure:
As stated

cc w/encl:
See next page

OFC	:LA:	:PM:PAD#2	:D:PAD#2	:	:	:	:
NAME	:MDuncan	:JHopkins;bg	:LSRubenstein	:	:	:	:
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P PDR

Mr. D. A. Nauman
South Carolina Electric & Gas Company Virgil C. Summer Nuclear Station

cc:

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3/4.6 CONTAINMENT SYSTEMS

BASES

3/4.6.1 PRIMARY CONTAINMENT

3/4.6.1.1 CONTAINMENT INTEGRITY

Primary CONTAINMENT INTEGRITY ensures that the release of radioactive materials from the containment atmosphere will be restricted to those leakage paths and associated leak rates assumed in the accident analyses. This restriction, in conjunction with the leakage rate limitation, will limit the site boundary radiation doses to within the limits of 10 CFR 100 during accident conditions.

3/4.6.1.2 CONTAINMENT LEAKAGE

The limitations on containment leakage rates (including those used in demonstrating a 30 day water seal) ensure that the total containment leakage volume will not exceed the value assumed in the accident analyses at the peak accident pressure, P_a . As an added conservatism, the measured overall integrated leakage rate is further limited to less than or equal to $0.75 L_a$ or $0.75 L_t$, as applicable, during performance of the periodic test to account for possible degradation of the containment leakage barriers between leakage tests.

The surveillance testing for measuring leakage rates are consistent with the requirements of Appendix "J" of 10 CFR 50.

3/4.6.1.3 REACTOR BUILDING AIR LOCKS

The limitations on closure and leak rate for the reactor building air locks are required to meet the restrictions on CONTAINMENT INTEGRITY and containment leak rate. Surveillance testing of the air lock seals provide assurance that the overall air lock leakage will not become excessive due to seal damage during the intervals between air lock leakage tests.

Distribution w/enclosure:

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