

December 22, 1986

Docket No. 50-395

Mr. D. A. Nauman  
Vice President, Nuclear Operations  
South Carolina Electric & Gas Company  
P.O. Box 764 (Mail Code 167)  
Columbia, South Carolina 29218

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Dear Mr. Nauman:

The Commission has issued the enclosed Amendment No. 57 to Facility Operating License No. NPF-12 for the Virgil C. Summer Nuclear Station, Unit No. 1. The amendment consists of a change to the Technical Specifications in response to your application dated August 2, 1985, as supplemented September 11, 1986.

The amendment revises the qualification requirements for individuals performing certain safety reviews required by Technical Specification Section 6.5.3.1.

The amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Jon B. Hopkins, Project Manager  
PWR Project Directorate #2  
Division of PWR Licensing-A  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 57 to NPF-12
2. Safety Evaluation

cc w/enclosures:

See next page

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12/4/86

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M. [unclear]  
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PD:PAD#2  
LRubenstein  
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Mr. D. A. Nauman  
South Carolina Electric & Gas Company

Virgil C. Summer Nuclear Station

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SOUTH CAROLINA ELECTRIC & GAS COMPANY

SOUTH CAROLINA PUBLIC SERVICE AUTHORITY

DOCKET NO. 50-395

VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 57  
License No. NPF-12

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by South Carolina Electric & Gas Company and South Carolina Public Service Authority (the licensees) dated August 2, 1985, as supplemented September 11, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-12 is hereby amended to read as follows:

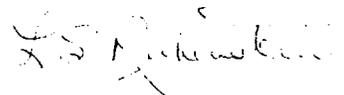
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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 57, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This amendment is effective as of its date of issuance, and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Lester S. Rubenstein, Director  
PWR Project Directorate #2  
Division of PWR Licensing-A  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: December 22, 1986

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 57 TO FACILITY OPERATING LICENSE NO. NPF-12

DOCKET NO. 50-395

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page.

Remove Pages

6-12

Insert Pages

6-12

## ADMINISTRATIVE CONTROLS

- c. Proposed tests and experiments which affect plant nuclear safety and are not addressed in the Final Safety Analysis Report shall be reviewed by an individual/group other than the individual/group which prepared the proposed test or experiment.
- d. Events reportable pursuant to the Technical Specification 6.9 and violations of Technical Specifications shall be investigated and a report prepared which evaluates the event and which provides recommendations to prevent recurrence. Such report shall be approved by the Director, Nuclear Plant Operations and forwarded to the Chairman of the Nuclear Safety Review Committee.
- e. Individuals responsible for reviews performed in accordance with 6.5.3.1.a, 6.5.3.1.b, 6.5.3.1.c and 6.5.3.1.d shall be members of the plant staff that meet or exceed the qualification requirements of Section 4 of ANSI 18.1, 1971, as previously designated by the Director, Nuclear Plant Operations. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the review personnel of the appropriate discipline.
- f. Each review will include a determination of whether or not an unreviewed safety question is involved.

## RECORDS

6.5.3.2 Records of the above activities shall be provided to the Director, Nuclear Plant Operations, PSRC and/or NSRC as necessary for required reviews.

## 6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the PSRC and the results of this review shall be submitted to the NSRC and the Vice President, Nuclear Operations.

## 6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within one hour. The Vice President, Nuclear Operations and the NSRC shall be notified within 24 hours.
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PSRC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- c. The Safety Limit Violation Report shall be submitted to the Commission, the NSRC and the Vice President, Nuclear Operations within 14 days of the violation.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 57 TO FACILITY OPERATING LICENSE NO. NPF-12

SOUTH CAROLINA ELECTRIC & GAS COMPANY

SOUTH CAROLINA PUBLIC SERVICE AUTHORITY

VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-395

INTRODUCTION

By letter dated August 2, 1985, as supplemented September 11, 1986, South Carolina Electric and Gas Company (the licensee) requested a change to Section 6.5.3.1(e) of the Technical Specifications for the Virgil C. Summer Nuclear Station. This proposed change would revise the qualification requirements for individuals performing certain safety reviews required by Section 6.5.3.1 of the Technical Specifications.

EVALUATION

Section 6.5.3.1 of the Technical Specifications now requires that individuals performing the safety reviews meet or exceed the qualification requirements of Section 4.4 of ANSI N18.1-1971, "Selection and Training of Nuclear Power Plant Personnel." Section 4.4 of the standard requires individuals to have a minimum of five years of experience in their fields, of which a minimum of two years should be related technical training and a maximum of four years may be related technical or academic training. Individuals filling positions in reactor engineering or physics must have a Bachelor's Degree in Engineering or the Physical Sciences plus two years of experience in their field.

The licensee's request for the change was prompted by an open item in Inspection Report 50-395/83-15. The inspector questioned the limited applicability of Section 4.4 of ANSI N18.1-1971, which specifically applies to Professional-Technical personnel in the fields of reactor engineering and physics, instrumentation and control, radiochemistry, and radiation protection. The inspector was of the opinion that the qualification requirements should be broadened to also include operations personnel, managers, supervisors or other positions at the facility.

The staff's review of the licensee's initial request revealed that while it eliminated the reference to Section 4.4 of the ANSI Standard, thereby accommodating the inspector's concern, it also reduced the required experience level of the reviewers to four years instead of the minimum of five years specified in the Standard.

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This preliminary staff finding was communicated to the licensee and, by letter dated September 11, 1986, the licensee amended its request. The licensee now proposes to change the wording of Technical Specification Section 6.5.3.1.e to read that, "Individuals responsible for reviews . . . shall . . . meet or exceed the qualification requirements of Section 4 of ANSI 18.1, 1971. . . ."

Section 4 of the ANSI Standard addresses the qualifications of all nuclear power plant personnel, thereby accommodating the concern of the inspector. The qualifications required by Section 4 of the standard are those endorsed by Regulatory Guide 1.8. Persons meeting the qualification requirements of the standard, therefore, meet the acceptance criteria of Section 13.4 of NUREG-0800, the Standard Review Plan. The proposed change is, therefore, acceptable.

#### ENVIRONMENTAL CONSIDERATION

This amendment involves a change to administrative procedures or requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### CONCLUSION

We have concluded, based on the considerations discussed above, that:  
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and  
(2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 22, 1986

#### Principal Contributors:

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