

October 31, 1983

CP1

Docket No. 50-395

Mr. O. W. Dixon, Jr.
Vice President Nuclear Operations
South Carolina Electric & Gas Company
P.O. Box 764
Columbia, South Carolina 29218

Dear Mr. Dixon:

Subject: Issuance of Amendment No. 18 to Facility Operating License NPF-12 Virgil C. Summer Nuclear Station, Unit No. 1

The Nuclear Regulatory Commission has issued Amendment No. 18 to Facility Operating License NPF-12 for the Virgil C. Summer Nuclear Station, Unit No. 1 located in Fairfield County, South Carolina. This amendment is in response to your letter dated July 22, 1983, and amended September 20, 1983.

The amendment changes the Technical Specifications to reflect changes in your nuclear organization structure. The amendment is effective as of its date of issuance.

A copy of the related safety evaluation supporting Amendment No. 18 to Facility Operating License NPF-12 is enclosed.

Sincerely,

S/

Elinor G. Adensam, Chief
Licensing Branch No. 4
Division of Licensing

Enclosures:

- 1. Amendment No. 18
- 2. Safety Evaluation

cc w/enclosure:
See next page

LA:DL:LB #4
MDuncan/hmc
10/19/83

DL:LB #4
JHopkins
10/19/83

DL:LB #4
EAdensam
10/19/83

SUMMER

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SOUTH CAROLINA ELECTRIC & GAS COMPANY

SOUTH CAROLINA PUBLIC SERVICE AUTHORITY

DOCKET NO. 50-395

VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 18
License No. NPF-12

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Virgil C. Summer Nuclear Station, Unit No. 1 (the facility) Facility Operating License No. NPF-12 filed by the South Carolina Electric & Gas Company acting for itself and South Carolina Public Service Authority (the licensees), dated July 22, 1983, and amended September 20, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this license amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C(2) of Facility Operating License No. NPF-12 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 18, are hereby incorporated into this license. South Carolina Electric & Gas Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Elinor G. Adensam, Chief
Licensing Branch No. 4
Division of Licensing

Enclosure:
Technical Specification Changes

Date of Issuance: October 31, 1983

*Immediately
before issuance
check for petitions
or comments.
If any
come back to [initials]*

LA:DL/LB #4
MDuncan/hmc
10/19/83

DL:LB #4
JHopkins
10/19/83

OELD Myung
10/20/83

FRN & amend only

DL:LB #4
EAdensam
10/21/83

AD:DL
Troyak
10/21/83

ATTACHMENT TO LICENSE AMENDMENT NO. 18

FACILITY OPERATING LICENSE NO. NPF-12

DOCKET NO. 50-395

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

<u>Amended</u> <u>Page</u>	<u>Overleaf</u> <u>Page</u>
6-1	
6-1a	
6-2	
6-3	
6-4	
6-5	
6-6	
6-7	
6-8	
6-10	6-9
6-11	
6-12	

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Director, Nuclear Plant Operations shall be responsible for overall unit operation and shall delegate in writing the succession to this responsibility during his absence. The title Director, Nuclear Plant Operations, is synonymous to the titles Manager, Virgil C. Summer Nuclear Station, Station Manager or Plant Manager.

6.1.2 The Shift Supervisor shall be responsible for unit operations. A management directive to this effect, signed by the Vice President, Nuclear Operations, shall be reissued to all station personnel on an annual basis.

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for unit management and technical support shall be as shown on Figure 6.2-1.

UNIT STAFF

6.2.2 The unit organization shall be as shown on Figure 6.2-2 and:

- a. Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Reactor Operator shall be in the control room when fuel is in the reactor. In addition, while the unit is in MODE 1, 2, 3 or 4, at least one Licensed Senior Reactor Operator shall be in the Control Room.
- c. A health physics technician[#] shall be on site when fuel is in the reactor.
- d. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- e. A site Fire Brigade of at least 5 members shall be maintained onsite at all times.[#] The Fire Brigade shall not include the Shift Supervisor and the other 2 members of the minimum shift crew necessary for safe shutdown of the unit and any personnel required for other essential functions during a fire emergency.

[#]The health physics technician and Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence provided immediate action is taken to fill the required positions.

ADMINISTRATIVE CONTROLS

- f. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions; e.g., senior reactor operators, reactor operators, health physicists, auxiliary operators, and key maintenance personnel.

Adequate shift coverage shall be maintained without routine heavy use of overtime. However, in the event that unforeseen problems require substantial amounts of overtime to be used, or during extended periods of shutdown for refueling, major maintenance or major plant modifications, on a temporary basis, the following guidelines shall be followed:

- a. An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time.
- b. An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any seven day period, all excluding shift turnover time.
- c. A break of at least eight hours should be allowed between work periods, including shift turnover time.
- d. Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Director, Nuclear Plant Operations or his deputy, or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual overtime shall be reviewed monthly by the Director, Nuclear Plant Operations or his designee to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized.

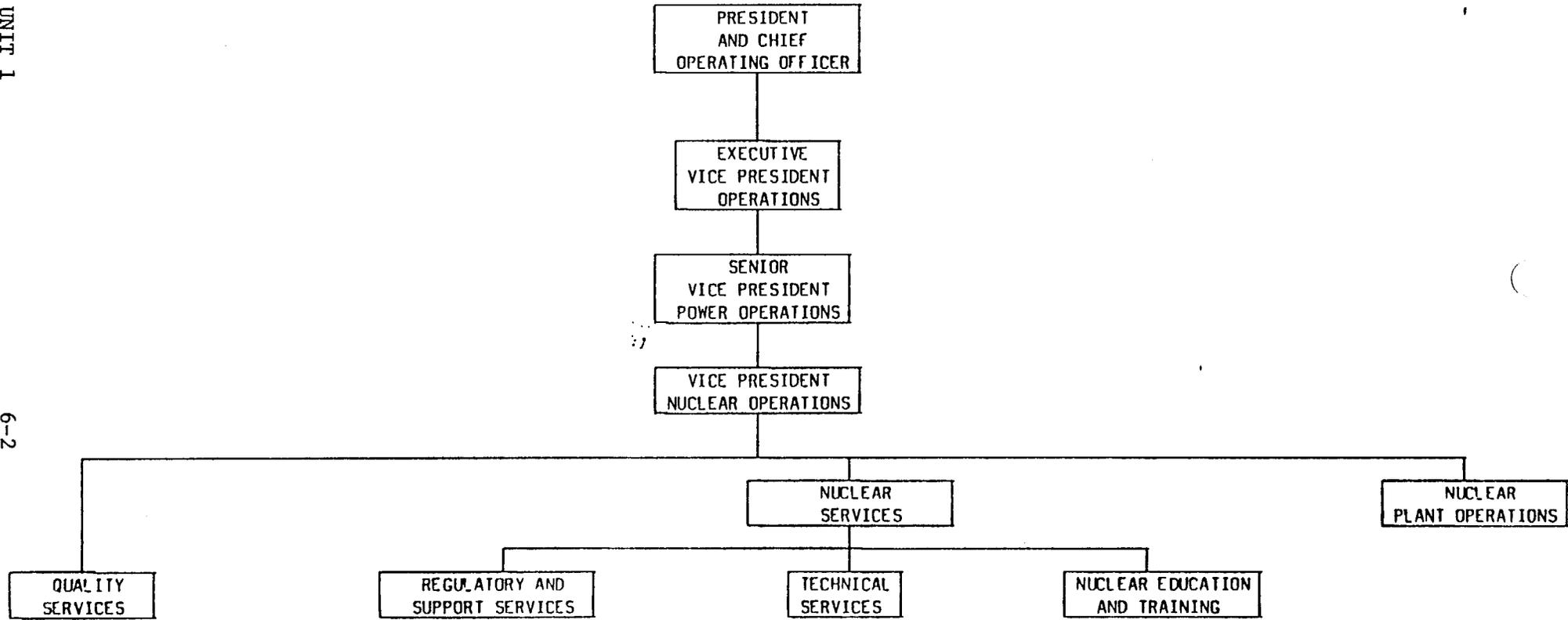
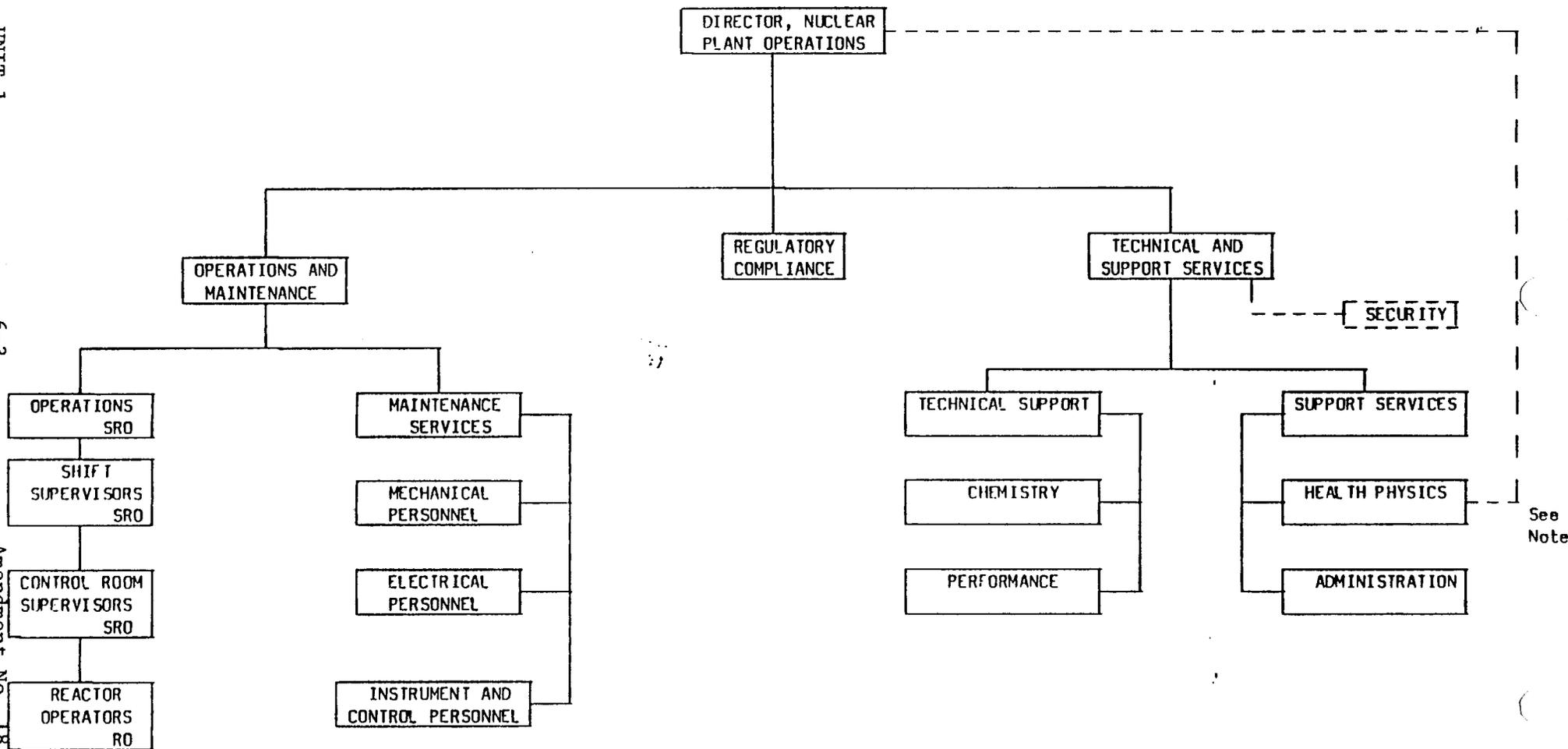


FIGURE 6.2.1
SOUTH CAROLINA ELECTRIC AND GAS COMPANY
VIRGIL C. SUMMER NUCLEAR STATION
OFFSITE ORGANIZATION



See Note

FIGURE 6.2-2
 VIRGIL C. SUMMER NUCLEAR STATION
 FUNCTIONAL ORGANIZATION

SRO - Senior Reactor Operator License
 RO - Reactor Operator License

NOTE: Health Physics supervision has direct access to the Director, Nuclear Plant Operations in matters concerning any phase of radiological protection.
 NOTE: The title Director, Nuclear Plant Operations is synonymous with the title Manager, Virgil C. Summer Nuclear Station, Station Manager or Plant Manager.

TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION

SUMMER UNIT 1

POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODES 1, 2, 3, & 4	MODES 5 & 6
SS	1	1
CRS	1	None
RO	2	1
AO	2	1
STA	1	None

- SS - Shift Supervisor with a Senior Reactor Operators License on Unit 1
- CRS - Control Room Supervisor with a Senior Reactor Operators License on Unit 1
- RO - Individual with a Reactor Operators License on Unit 1
- AO - Auxiliary Operator
- STA - Shift Technical Advisor

Except for the Shift Supervisor, the Shift Crew Composition may be one less than the minimum requirements of Table 6.2-1 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the Shift Crew Composition to within the minimum requirements of Table 6.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

During any absence of the Control Room Supervisor from the Control Room while the unit is in MODE 1, 2, 3 or 4, an individual (other than the Shift Technical Advisor) with a valid SRO license shall be designated to assume the Control Room command function. During any absence of the Shift Supervisor from the Control Room while the unit is in MODE 5 or 6, an individual with a valid RO or SRO license shall be designated to assume the Control Room command function.

ADMINISTRATIVE CONTROLS

6.2.3 INDEPENDENT SAFETY ENGINEERING GROUP (ISEG)

FUNCTION

6.2.3.1 The ISEG shall function to examine plant operating characteristics, NRC issuances, industry advisories, Licensee Event Reports and other sources of plant design and operating experience information, including plants of similar design, which may indicate areas for improving plant safety.

COMPOSITION

6.2.3.2 The ISEG shall be composed of a multi-disciplined dedicated onsite group with a minimum assigned complement of five engineers or appropriate specialists.

RESPONSIBILITIES

6.2.3.3 The ISEG shall be responsible for maintaining surveillance of plant activities to provide independent verification* that these activities are performed correctly and that human errors are reduced as much as practical.

AUTHORITY

6.2.3.4 The ISEG shall make detailed recommendations for procedure revisions, equipment modifications, maintenance activities, operations activities or other means of improving plant safety to the Group Manager, Technical Services.

6.2.4 SHIFT TECHNICAL ADVISOR

The Shift Technical Advisor shall provide technical support to the Shift Supervisor in the areas of thermal hydraulics, reactor engineering and plant analysis with regard to the safe operation of the unit.

6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 letter to all licensees, except for the Associate Manager, Health Physics who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, the Shift Technical Advisor who shall meet or exceed the qualifications referred to in Section 2.2.1.b of Enclosure I of the October 30, 1979 NRC letter to all operating nuclear power plants, and the members of the Independent Safety Engineering Group, each of whom shall have a Bachelor of Science degree or registered Professional Engineer and at least two years experience in their field. At least one year experience shall be in the nuclear field.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with identified relevant industry operational experience.

*Not responsible for sign-off function.

ADMINISTRATIVE CONTROLS

6.5 REVIEW AND AUDIT

6.5.1 PLANT SAFETY REVIEW COMMITTEE (PSRC)

FUNCTION

6.5.1.1 The PSRC shall function to advise the Director, Nuclear Plant Operations on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Plant Safety Review Committee shall be composed of the:

Chairman:	Director, Nuclear Plant Operations or Deputy Director, Operations and Maintenance
Member:	Manager Operations
Member:	Manager Technical Support
Member:	Manager Maintenance Services
Member:	Group Manager, Technical and Support Services
Member:	Associate Manager, Health Physics or a Health Physics Supervisor

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PSRC Chairman to serve on a temporary basis; however, no more than two alternates including the Chairman's alternate, if applicable, shall participate as voting members in PSRC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PSRC shall meet at least once per calendar month and as convened by the PSRC Chairman or his designated alternate.

QUORUM

6.5.1.5 The minimum quorum of the PSRC necessary for the performance of the PSRC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and three members including alternates.

RESPONSIBILITIES

6.5.1.6 The Plant Safety Review Committee shall review:

- a. Station administrative procedures and changes thereto,
- b. The safety evaluations for 1) procedures, 2) changes to procedures, equipment or systems, and 3) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question and all programs required by Specification 6.8 and changes thereto.
- c. Proposed procedures and changes to procedures, equipment or systems which may involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed tests or experiments which may involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- e. Proposed changes to Technical Specifications or the Operating License.

ADMINISTRATIVE CONTROLS

- f. Reports of violations of codes, regulations, orders, Technical Specifications, or Operating License requirements having nuclear safety significance or reports of abnormal degradation of systems designed to contain radioactive material.
- g. Reports of significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- h. All written reports concerning events requiring 24 hour notification to the Commission.
- i. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- j. The plant Security Plan and changes thereto.
- k. The Emergency Plan and changes thereto.
- l. Items which may constitute a potential nuclear safety hazard as identified during review of facility operations.
- m. Investigations or analyses of special subjects as requested by the Chairman of the Nuclear Safety Review Committee.
- n. The unexpected offsite release of radioactive material and the report as described in 6.9.1.13(e).
- o. Changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL.

AUTHORITY

- 6.5.1.7 The Plant Safety Review Committee shall:
- a. Recommend in writing to the Director, Nuclear Plant Operations approval or disapproval of items considered under 6.5.1.6a, c, d, e, j, and k above.
 - b. Render determinations in writing to the Director, Nuclear Plant Operations with regard to whether or not each item considered under 6.5.1.6a, c, and d above constitutes an unreviewed safety question.
 - c. Make recommendations in writing to the Director, Nuclear Plant Operations that actions reviewed under 6.5.1.6(b) above did not constitute an unreviewed safety question.
 - d. Provide written notification within 24 hours to the Vice President, Nuclear Operations and the Nuclear Safety Review Committee of disagreement between the PSRC and the Director, Nuclear Plant Operations however, the Director, Nuclear Plant Operations shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

- 6.5.1.8 The Plant Safety Review Committee shall maintain written minutes of each PSRC meeting that, at a minimum, document the results of all PSRC activities performed under the responsibility and authority provisions of these technical specifications. Copies shall be provided to the Vice President Nuclear Operations and the Chairman of the Nuclear Safety Review Committee.

ADMINISTRATIVE CONTROLS

6.5.2 NUCLEAR SAFETY REVIEW COMMITTEE (NSRC)

FUNCTION

6.5.2.1 The Nuclear Safety Review Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

COMPOSITION

6.5.2.2 NSRC shall consist of a Chairman and four or more other members appointed by the Vice President, Nuclear Operations. No more than a minority of the members of the NSRC shall have line responsibility for the operation of the unit.

The NSRC members shall hold a Bachelor's degree in an engineering or physical science field or equivalent experience and a minimum of five years of technical experience of which a minimum of three years shall be in one or more of the disciplines of 6.5.2.1a through h. In the aggregate, the membership of the committee shall provide specific practical experience in the majority of the disciplines of 6.5.2.1a through h.

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the Vice President, Nuclear Operations; however, no more than two alternates shall participate as voting members in NSRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the NSRC Chairman to provide expert advice to the NSRC.

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.2.5 The NSRC shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per six months thereafter.

QUORUM

6.5.2.6 A quorum of the NSRC necessary for the performance of the NSRC review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least 3 NSRC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit.

REVIEW

6.5.2.7 The NSRC shall review:

- a. The safety evaluations for 1) changes to procedures, equipment or systems, and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications or this Operating License.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
- g. Events requiring 24 hour written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- i. Reports and meetings minutes of the Plant Safety Review Committee.

ADMINISTRATIVE CONTROLS

AUDITS

6.5.2.8 The NSRC shall have cognizance of the audits listed below. Audits may be performed by using established SCE&G groups such as the ISEG and QA or by outside groups as determined by the NSRC. Audit reports or summaries will be the basis for NSRC action:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire unit staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Emergency Plan and implementing procedures at least once per 12 months.
- f. The Security Plan and implementing procedures at least once per 12 months.
- g. Any other area of unit operation considered appropriate by the NSRC or the Vice President, Nuclear Operations.
- h. The Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or a qualified outside firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years.
- k. The radiological environmental monitoring program and the results thereof, including the performance of activities required by the quality assurance program per R.G. 4.15 Rev. 1, February 1979, at least once per 12 months.
- l. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months.
- m. The PROCESS CONTROL PROGRAM and implementing procedures for solidification of radioactive wastes at least once per 24 months.

AUTHORITY

6.5.2.9 The NSRC shall report to and advise the Vice President, Nuclear Operations on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

ADMINISTRATIVE CONTROLS

RECORDS

6.5.2.10 Records of NSRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NSRC meeting shall be prepared, approved and forwarded to the Vice President, Nuclear Operations within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Vice President, Nuclear Operations within 14 days following completion of the review.
- c. Audit summary reports encompassed by Section 6.5.2.8 above, shall be forwarded to the NSRC and to the Vice President, Nuclear Operations. Full audits shall be forwarded to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

6.5.3 TECHNICAL REVIEW AND CONTROL

ACTIVITIES

6.5.3.1 Activities which affect nuclear safety shall be conducted as follows:

- a. Procedures required by Technical Specification 6.8 and other procedures which affect plant nuclear safety, and changes thereto, shall be prepared, reviewed and approved. Each such procedure or procedure change shall be reviewed by an individual/group other than the individual/ group which prepared the procedure or procedure change, but who may be from the same organization as the individual/ group which prepared the procedure or procedure change. Procedures other than Administrative Procedures will be approved as delineated in writing by the Director, Nuclear Plant Operations. The Director, Nuclear Plant Operations, will approve administrative procedures, security implementing procedures and emergency plan implementing procedures. Temporary approval to procedures which clearly do not change the intent of the approved procedures can be made by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License. For changes to procedures which may involve a change in intent of the approved procedures, the person authorized above to approve the procedure shall approve the change.
- b. Proposed changes or modifications to plant nuclear safety-related structures, systems and components shall be reviewed as designated by the Director, Nuclear Plant Operations. Each such modification shall be designed as authorized by Nuclear Engineering and shall be reviewed by an individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modifications. Implementation of modifications to plant nuclear safety-related structures, systems and components shall be concurred in by the Director, Nuclear Plant Operations.

ADMINISTRATIVE CONTROLS

- c. Proposed tests and experiments which affect plant nuclear safety and are not addressed in the Final Safety Analysis Report shall be reviewed by an individual/group other than the individual/group which prepared the proposed test or experiment.
- d. Occurrences reportable pursuant to the Technical Specification 6.9 and violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. Such report shall be approved by the Director, Nuclear Plant Operations and forwarded to the Chairman of the Nuclear Safety Review Committee.
- e. Individuals responsible for reviews performed in accordance with 6.5.3.1.a, 6.5.3.1.b, 6.5.3.1.c and 6.5.3.1.d shall be members of the plant staff that meet or exceed the qualification requirements of Section 4.4 of ANSI 18.1, 1971, as previously designated by the Director, Nuclear Plant Operations. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the review personnel of the appropriate discipline.
- f. Each review will include a determination of whether or not an unreviewed safety question is involved.

RECORDS

6.5.3.2 Records of the above activities shall be provided to the Director, Nuclear Plant Operations, PSRC and/or NSRC as necessary for required reviews.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the PSRC and submitted to the NSRC and the Vice President, Nuclear Operations.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within one hour. The Vice President, Nuclear Operations and the NSRC shall be notified within 24 hours.
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PSRC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- c. The Safety Limit Violation Report shall be submitted to the Commission, the NSRC and the Vice President, Nuclear Operations within 14 days of the violation.

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 18 TO FACILITY OPERATING LICENSE NPF-12

SOUTH CAROLINA ELECTRIC & GAS COMPANY

SOUTH CAROLINA PUBLIC SERVICE AUTHORITY

I. INTRODUCTION

Below is a brief description and our evaluation of the proposed changes to the off-site and plant staff organization for the South Carolina Electric and Gas Company (SCE&G) Virgil C. Summer Nuclear Station as requested by letter dated July 22, 1983, and amended by letter dated September 20, 1983.

The September 20, 1983, letter changed the titles of two of the positions listed in the July 22, 1983, letter. These title changes are from Principal Engineer, Nuclear Safety and Deputy Manager, Operations and Maintenance to Senior Engineer, Nuclear Safety and Deputy Director, Operations and Maintenance respectively. The functions, responsibility, and authority of these two positions did not change from that of the original amendment request of July 22, 1983. This request was noticed (48 FR 39192) on August 29, 1983, with a proposed determination of no significant hazards consideration. These title changes did not substantively affect the proposed determination of no significant hazards consideration and, therefore, this amendment was not renoticed.

II. DISCUSSION AND EVALUATION

A. Off-site Organization

SCE&G has instituted changes to centralize the management of their nuclear departments other than Quality Assurance and Plant Operation and to establish a direct communications channel to the Vice President, Nuclear Operations from quality assurance and plant operations. The revised organization, shown in Technical Specification Figure 6.2-1, establishes the functional entity of Quality Services (responsible for operational quality assurance) which reports directly to the Vice President, Nuclear Operations; establishes the new position of Director, Nuclear Services that centralizes the management of Regulatory and Administrative Services, Technical Services, Support Services, and Nuclear Education and Training into that position; and provides for the Director, Nuclear Plant Operations to report directly to the Vice President, Nuclear Operations. The reporting chain of the Nuclear Safety Review Committee has been changed from the Senior Vice President, Power Operations to the Vice President, Nuclear Operations.

We have reviewed these changes and find them acceptable as they meet the acceptance criteria of Section 13.1.1 and 13.4 of the Standard Review Plan, NUREG-0800.

B. On-site Organization

SCE&G has instituted changes under the Director, Nuclear Plant Operations (new title) to centralize plant activities under the new positions of Deputy Director, Operations and Maintenance, and Group Manager, Technical and Support Services. Additionally the position of Associate Manager, Regulatory Compliance has been established. These changes are shown in Technical Specification Figure 6.2.-2. With the creation of these new positions, the position of Deputy Manager, Summer Nuclear Plant has been deleted.

We have reviewed these changes and found them acceptable as they meet the requirements of Section 13.1.2 of the Standard Review Plan, NUREG-0800, and the requirements of Section 3.4 of ANSI-N18.7, 1976, "Administrative Controls and Quality Assurance for the Operational Phase of Nuclear Power Plants", as endorsed by Regulatory Guide 1.33.

III. ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

IV. CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (48 FR 39192) on August 29, 1983, and consulted with the state of South Carolina. No public comments were received, and the state of South Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

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Dated: October 31, 1983

October 31, 1983

AMENDMENT NO. 18 TO FACILITY OPERATING LICENSE NPF-12 - Virgil C. Summer Unit 1

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