CP 52

Docket No. 50-395

DEC 29 1982

Mr. O. W. Dixon, Jr.
Vice President Nuclear Operations
South Carolina Electric and Gas Company
P.O. Box 764
Columbia, South Carolina 29218

Dear Mr. Dixon:

Subject: Authorization of Technical Specification Change
for Virgil C. Summer Nuclear Station, Unit 1

This confirms our telephone authorization given on December 28, 1982, for a change in Technical Specifications for the Virgil C. Summer facility as requested by your telecopy dated December 28, 1982. Facility Operating License NPF-12 is hereby amended by making the following Technical Specification change:

Add footnote (3) to Table 3.6-1 for valves 9398A-SS, 9398B-SS, and 9398C-SS to read:

The provisions of Specification 3.0.4 are not applicable from December 28, 1982, until July 1, 1983.

Copies of the license amendment, our evaluation and the Federal Register Notice for this Technical Specification change will be sent to you when completed.

Sincerely,

A Schwencer

Thomas M. Novak, Assistant Director
for Licensing
Division of Licensing

cc: See next page

CP

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Summer [redacted]

Safety Evaluation Report

Background

Containment isolation valves 9398A-SS and 9398B-SS, sampling line from steam generators A and B respectively, were recently declared inoperable because they were not able to close within the 40 seconds allowed by Technical Specification Table 3.6-1. Action statement 3.6.4.c ~~requires that these~~ states that plant operation may continue with one or more containment isolation valves inoperable provided that each affected penetration is isolated by at least one manual closed valve.

The licensee ~~has~~ complied with the

action statement and continued plant operation.

On December 28, 1982, the Summer plant tripped due to a personnel error. According to specification 3.0.4, plant startup may not take place because the plant is currently relying on provisions contained in the action statement.

On December 28, 1982, the licensee requested an emergency technical specification change to allow plant startup with these two containment isolation valves inoperable.

Evaluation

The licensee's proposed basis for requesting an exemption to specification 3.0.4 is as follows:

1. the sample lines involved are small (3/8") high pressure secondary system lines that are exempt from 10 CFR 50 Appendix J, Type C testing requirements.

2. these valves are currently in their containment isolation position (closed).

3. ~~these lines are~~ equipped with manual isolation valves downstream which have also been closed.

4. continuous operation is allowed within the action statement indicating that ~~the safety concern is that the valves are~~ isolated in their safety position, not ~~that they are~~ immediately ~~restored to operation.~~

5. alternate methods for sampling exist which provide representative samples and do not increase operator dose rates, namely via the Steam Generator Blowdown System through valves 550A-BD and 550D-BD or 10512-BD and 10513-BD for steam generator A and 550B-BD and ~~550E-BD~~ or 10514-BD and 10515-BD for steam generator B and 550C-BD and 550F-BD or 10516-BD and 10517-BD for steam generator C.

Although only two of the three steam generator sample line isolation valves are inoperable, the licensee has requested that the change be applicable for all three isolation valves as a precautionary measure.

Repair of these isolation valves is currently the critical path for plant startup. Only one of the two replacement valves is currently onsite. ~~At issue~~ the issue is this condition poses an economic concern for South Carolina Electric and Gas Company (SCE&G) and the South Carolina Public Service Authority (SCPSA). Because SCE&G's current generating configuration allowed several fossil units to be shut down, ~~we are~~ now required to purchase 300 MWe from neighboring utilities and operate a pumped storage facility to replace the Virgil C. Summer Nuclear Station's output. These sources of energy are very limited. ~~Restart of the Virgil C. Summer Nuclear Station is the most feasible means of supplying our power needs.~~

We concur with the licensee that ~~de~~ restart of the Virgil C Summer Nuclear Station is the most feasible means of supplying the power needs. The proposed technical specification change states that this one time only exception will only last

through July 1, 1983. It is our understanding that the licensee plans to shutdown in late March 1983 or early April 1983 to perform steam generator notifications. The licensee states that this is when the valve replacement will take place.

We conclude that the proposed technical specification changes does not involve an unreviewed safety question. We agree that safe operation may be conducted with these valves in the closed position. As stated above, steam generator sampling will not be interrupted as a result of this change. Therefore we find the proposed change to be acceptable.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR Section 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered, does not create the possibility of an accident of a type different from any evaluated previously and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

DATE:

Principal Contributors:
Jon Hopkins
Doug Pickett

OK DeAngelis 12/28/82
OK A. Schwencer for T. Novak 12/28/82

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	MAXIMUM ISOLATION TIME (SEC)
A. PHASE "A" ISOLATION (Continued)		
31. 9312A-SS	Sampling Line Supply Return From Radiation Monitor	40
32. 9312B-SS	Sampling Line Return Return From Radiation Monitor	40
33. 9339-SS	Sample Return Line To PRT	40
34. 9341-SS	Sample Return Line To PRT	40
35. 9356A-SS	Sampling Line From Pressurizer	40
36. 9356B-SS	Sampling Line From Pressurizer	40
37. 9357-SS	Sampling Line From Pressurizer	40
38. 9364B-SS	Sampling Lines From Reactor Coolant Loop B	40
39. 9365B-SS	Sampling Lines From Reactor Coolant Loop B	40
40. 9364C-SS	Sampling Lines From Reactor Coolant Loop C	40
41. 9365C-SS	Sampling Lines From Reactor Coolant Loop C	40
42. 9387-SS	Sampling Line From Accumulators	40
43. 9398A-SS # (3)	Sampling Line From Steam Generator A Blowdown	40
44. 9398B-SS # (3)	Sampling Line From Steam Generator B Blowdown	40
45. 9398C-SS # (3)	Sampling Line From Steam Generator C Blowdown	40
46. 7126-WL	Reactor Coolant Drain Tank Vent Header	40
47. 7150-WL	Reactor Coolant Drain Tank Vent Header	40
48. 1003-WL	Reactor Coolant Drain Tank Discharge To Waste	40
49. 7136-WL	Reactor Coolant Drain Tank Discharge To Waste	40
B. PHASE "B" ISOLATION		
1. 9568-CC	Component Cooling To R. C. Pumps Bearings	60
2. 9600-CC	Component Cooling To R. C. Pumps	60
3. 9605-C	Component Cooling From R. C. Pumps Bearings	60
4. 9606-CC	Component Cooling From R. C. Pumps Bearings	60
5. 1638A-FW #	Chemical Feed Line To Feedwater Loop A	60
6. 1638B-FW #	Chemical Feed Line To Feedwater Loop B	60
7. 1638C-FW #	Chemical Feed Line To Feedwater Loop C	60
C. REACTOR BUILDING PURGE SUPPLY AND EXHAUST ISOLATION		
1. 0001A-AH	Reactor Building Purge Supply	5
2. 0001B-AH	Reactor Building Purge Supply	5
3. 0002A-AH	Reactor Building Purge Exhaust	5

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER F. CHECK	FUNCTION	MAXIMUM ISOLATION TIME (SEC)
1. 7541-AC	CRDM Coolant Water Inlet Line	N/A
2. 7544-AC	CRDM Coolant Water Outlet Line	N/A
3. 9570-CC	Component Cooling To R. C. Pump Bearings	N/A
4. 9689-CC	Component Cooling From R. C. Pump Bearings	N/A
5. 8103-CS	Reactor Coolant Pump Seal Water Return	N/A
6. 8368A-CS	Seal Injection To R. C. Pump A	N/A
7. 8368B-CS	Seal Injection To R. C. Pump B	N/A
8. 8368C-CS	Seal Injection To R. C. Pump C	N/A
9. 8381-CS	Charging Line To Regenerative Heat Exchanger	N/A
10. 6799-FS	Fire Service Deluge To Charcoal Filters	N/A
11. 2661-IA	Instrument Air Supply To Reactor Building	N/A
12. 6588-NG	Nitrogen Supply To Steam Generators	N/A
13. 8046-RC	Pressurizer Relief Tank Makeup Water Line	N/A
14. 2913-SA	Service Air Supply To Reactor Building	N/A
15. 3009A-SP	Supply To Reactor Building Spray Nozzles	N/A
16. 3009B-SP	Supply To Reactor Building Spray Nozzles	N/A
17. 8947-SI	Accumulator Nitrogen Supply	N/A
18. 8861-SI	Fill Line To Accumulators	N/A

#Valve not subject to Type "C" leakage test.

(1) Manual valves may be opened on an intermittent basis under administrative control.

(2) Remote manual valve positions are maintained by administrative control.

(3) The provisions of Specification 3.0.4 are not applicable from December 28, 1982 until July 1, 1983.

DEC 2 9 1982

AUTHORIZATION OF TECHNICAL SPECIFICATION CHANGE FOR VIRGIL C. SUMMER NUCLEAR STATION UNIT 1

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