



REGULATORY COPY  
UNITED STATES NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

August 20, 1980

Dockets Nos. 50-269, 270  
✓ 287

Mr. William O. Parker, Jr.  
Vice President, Steam Production  
Duke Power Company  
P. O. Box 2178  
422 South Church Street  
Charlotte, North Carolina 28242

Dear Mr. Parker:

The Commission has issued the enclosed Amendments Nos. 86, 86, and 83 for Licenses Nos. DPR-38, DPR-47 and DPR-55 for the Oconee Nuclear Station, Units Nos. 1, 2 and 3. These amendments consist of changes to the Station's common Technical Specifications and are in response to your submittal dated July 7, 1980.

These amendments revise the Technical Specifications by incorporating the test frequency requirements of 10 CFR 50, Appendix J, into the containment leak test program.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing

Enclosures:

1. Amendment No. 86 to DPR-38
2. Amendment No. 86 to DPR-47
3. Amendment No. 83 to DPR-55
4. Safety Evaluation
5. Notice of Issuance

cc w/enclosures: See next page

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Duke Power Company

cc w/enclosure(s):

Mr. William L. Porter  
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P. O. Box 2178  
422 South Church Street  
Charlotte, North Carolina 28242

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Babcock & Wilcox  
Nuclear Power Generation Division  
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Bethesda, Maryland 20014

Oconee Public Library  
201 South Spring Street  
Walhalla, South Carolina 29691

Manager, LIS  
NUS Corporation  
2536 Countryside Boulevard  
Clearwater, Florida 33515

Honorable James M. Phinney  
County Supervisor of Oconee County  
Walhalla, South Carolina 29621

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DeBevoise & Liberman  
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Washington, D. C. 20036

Director, Technical Assessment  
Division  
Office of Radiation Programs  
(AW-459)  
U. S. Environmental Protection Agency  
Crystal Mall #2  
Arlington, Virginia 20460

cc w/enclosure(s) and incoming dtd.:  
7/7/80

Office of Intergovernmental Relations  
116 West Jones Street  
Raleigh, North Carolina 27603

U. S. Environmental Protection Agency  
Region IV Office  
ATTN: EIS COORDINATOR  
345 Courtland Street, N.E.  
Atlanta, Georgia 30308

Mr. Francis Jape  
U. S. Nuclear Regulatory Commission  
P. O. Box 7  
Seneca, South Carolina 29678



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 86  
License No. DPR-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Duke Power Company (the licensee) dated July 7, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-38 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 86 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing

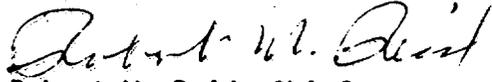
Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 20, 1980



3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 20, 1980



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-287

OCONEE NUCLEAR STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 83  
License No. DPR-55

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Duke Power Company (the licensee) dated July 7, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-55 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 83 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 20, 1980

- a. Personnel hatch
- b. Emergency hatch
- c. Equipment hatch seals
- d. Fuel transfer tube seals
- e. Reactor Building normal sump drain line
- f. Reactor coolant pump seal outlet line
- g. Reactor coolant pump seal inlet line
- h. Quench tank drain line
- i. Quench tank return line
- j. Quench tank vent line
- k. Normal makeup to Reactor Coolant System
- l. High pressure injection line
- m. Electrical penetrations
- n. Reactor Building purge inlet line
- o. Reactor Building purge outlet line
- p. Reactor Building sample lines
- q. Reactor coolant letdown line

#### 4.4.1.2.2 Conduct of Tests

- a. Local leak rate tests shall be performed at a pressure of not less than 59 psig.
- b. Acceptable methods of testing are halogen gas detection, soap bubbles, pressure decay, hydrostatic flow or equivalent.

#### 4.4.1.2.3 Acceptance Criteria

The total leakage from all penetrations and isolation valves shall not exceed 0.125 weight percent of the Reactor Building atmosphere per 24 hours.

#### 4.4.1.2.4 Corrective Action and Retest

- a. If at any time it is determined that the criterion of 4.4.1.2.3 above is exceeded, repairs shall be initiated immediately.
- b. If conformance to the criterion of 4.4.1.2.3 is not demonstrated within 48 hours following detection of excessive local leakage, the reactor shall be shutdown and depressurized until repairs are effected and the local leakage meets the acceptance criterion as demonstrated by retest.

#### 4.4.1.2.5 Test Frequency

Local leak detection tests shall be performed during each reactor shutdown for refueling but in no case at intervals greater than 2 years except that:

- a. The equipment hatch and fuel transfer tube seals shall be additionally tested after each opening.
- b. The personnel hatch and emergency hatch outer door seals shall be tested at four-month intervals, except when the hatches are not opened during that interval. In no case shall the test interval be longer than 12 months.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 86 TO FACILITY OPERATING LICENSE NO. DPR-38

AMENDMENT NO. 86 TO FACILITY OPERATING LICENSE NO. DPR-47

AMENDMENT NO. 83 TO FACILITY OPERATING LICENSE NO. DPR-55

DUKE POWER COMPANY

OCONEE NUCLEAR STATION, UNITS NOS. 1, 2 AND 3

DOCKETS NOS. 50-269, 50-270 AND 50-287

Introduction

By letter dated July 7, 1980, the Duke Power Company (the licensee) submitted proposed changes to the Technical Specifications (TSs) to incorporate the test frequency requirements of 10 CFR 50.54(o), Appendix J, into the containment leak test program.

Evaluation

The existing TSs for the Oconee Station state in Section 4.4.1.2.5 that: "Local leak detection tests shall be performed annually, ...". The licensee proposes to change Section 4.4.1.2.5 to read: "Local leak detection tests shall be performed during each reactor shutdown for refueling but in no case at intervals greater than 2 years...". The proposed change is a verbatim incorporation of the requirements of 10 CFR 50.54(o), Appendix J, Sections III.D.2 and III.D.3. These sections of the Commission's regulations pertain to test frequency of the components being tested in TS 4.4.1.2.5. Based on the above, we conclude that the change is acceptable.

The staff is currently reviewing the entire containment leak testing program for the Oconee Station to determine its compliance with Appendix J of 10 CFR 50. Changes to the program may be required as a result of this review. However, the results of the staff review are not expected to affect the change resulting from this license amendment.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: August 20, 1980

UNITED STATES NUCLEAR REGULATORY COMMISSION

7590-01

DOCKETS NOS. 50-269, 50-270 AND 50-287

DUKE POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY  
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 86, 86, and 83 to Facility Operating Licenses Nos. DPR-38, DPR-47 and DPR-55, respectively, issued to Duke Power Company, which revised the Technical Specifications for operation of the Oconee Nuclear Station, Units Nos. 1, 2 and 3, located in Oconee County, South Carolina. The amendments are effective as of the date of issuance.

These amendments revise the Station's Common Technical Specifications by incorporating the test frequency requirements of 10 CFR 50, Appendix J, into the containment leak test program.

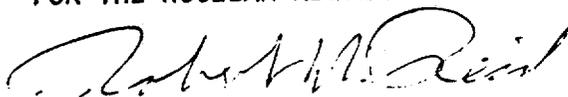
The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR Section 57.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated July 7, 1980, (2) Amendments Nos. 86, 86, and 83 to Licenses Nos. DPR-38, DPR-47 and DPR-55, respectively, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Oconee County Library, 201 South Spring Street, Walhalla, South Carolina. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 20th day of August 1980.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing