

APR 16 1976

Docket Nos. 50-269/270/287

Duke Power Company
ATTN: Mr. William O. Parker, Jr.
Vice President
Steam Production
Post Office Box 2178
Charlotte, North Carolina 28242

Gentlemen:

By letter dated March 22, 1976, you requested an exemption from the requirements of 10 CFR Part 50, Appendix H, Section II.C.2, to permit the operation of Oconee Unit 3 for the remainder of Cycle 1 with the reactor vessel surveillance specimens removed from the reactor vessel. You additionally requested corresponding Technical Specification changes to reflect the removal of the surveillance capsules and to establish provisions to revise the capsule withdrawal schedule prior to Cycle 2 operation.

By letter dated April 15, 1976, you additionally proposed limiting conditions for operation for Oconee 3, Cycle 1 to assure that the possibility of further degradation of the surveillance capsule holder tubes is minimized and to assure that a failed holder tube could be detected.

We have concluded that if the reactor vessel surveillance capsules are removed for the remainder of Oconee Unit 1 Cycle 1 operation, the reactor vessel surveillance program would continue to fulfill the purpose of 10 CFR Part 50, Appendix H.

An exemption to the requirements of Section II.C.2 of Appendix H is therefore granted for Oconee Unit 3 and operation with the surveillance capsules removed for the remainder of Cycle 1 is hereby authorized. In addition, the Commission has issued the enclosed Amendments No. , and , for Licenses DPR-38, DPR-47 and DPR-55, for the Oconee Nuclear Station, Units 1, 2 and 3. These amendments provide for the removal of the surveillance capsules during a portion of Unit 3 Cycle 1 operation, require that the capsule withdrawal schedule be revised prior to Cycle 2 and impose additional Limiting Conditions for Operation for operation of Unit 3 for the remainder of Cycle 1.

OFFICE >						
SURNAME >						
DATE >						

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Duke Power Company

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Copies of the Safety Evaluation and the Federal Register Notice are enclosed.

Sincerely,

Victor Stello, Jr., Director
Division of Operating Reactors
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. to DPR-38
- 2. Amendment No. to DPR-47
- 3. Amendment No. to DPR-55
- 4. Safety Evaluation
- 5. Federal Register Notice

cc w/enclosures:
See next page

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Duke Power Company

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April 16, 1976

cc w/enclosures:

Mr. William L. Porter

Duke Power Company

P. O. Box 2178

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Charlotte, North Carolina 28242

Mr. Troy B. Conner

Conner & Knotts

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Oconee Public Library

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Walhalla, South Carolina 29691

Honorable Reese A. Hubbard

County Supervisor of Oconee County

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cc w/enclosures & incoming:

Office of Intergovernmental

Relations

116 West Jones Street

Raleigh, North Carolina 27603



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.23
License No. DPR-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duke Power Company (the licensee) dated March 22, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for Charles M. Trammell
Robert A. Purple, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the
Technical Specifications

Date of Issuance: April 16, 1976



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-270

OCONEE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 23
License No. DPR-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duke Power Company (the licensee) dated March 22, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for *Charles M. Trammell*
Robert A. Purple, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the
Technical Specifications

Date of Issuance: April 16, 1976



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-287

OCONEE NUCLEAR STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 20
License No. DPR-55

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duke Power Company (the licensee) dated March 22, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for *Charles M. Trammell*
Robert A. Purple, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the
Technical Specifications

Date of Issuance: April 16, 1976

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 23 TO FACILITY LICENSE NO. DPR-38

AMENDMENT NO. 23 TO FACILITY LICENSE NO. DPR-47

AMENDMENT NO. 20 TO FACILITY LICENSE NO. DPR-55

DOCKET NOS. 50-269, 50-270, AND 50-287

Revise Appendix A as follows:

Remove page 4.2-3 and insert revised page 4.2-3.

Remove page 3.17-1 and insert revised page 3.17-1

3.17 ADDITIONAL OPERATING RESTRICTIONS FOR OCC 2 3, CYCLE 1

Applicability

Applies to the operation of Occurrence 3, Cycle 1 and is deleted after September 1, 1976.

Objective

To provide assurance that the operation of Occurrence 3, Cycle 1 is in such a manner as to minimize the stress in degraded reactor vessel surveillance specimen holder tubes and to assure the capability to detect and respond to the possible failure of the holder tubes.

Specification

- 3.17.1 The Loose Parts Monitoring System shall have as a minimum two channels on the reactor vessel head service structure and one channel on the incore guide tubes operable when any reactor coolant pumps are operating.
- 3.17.2 a. Any abnormal indication on the Loose Parts Monitoring System shall be promptly investigated and an evaluation performed considering such factors as the duration of indication, intensity of the indication, location of the indication and comparability of the indication to previously observed/reference indications. Based on this evaluation, a determination shall be made as to whether or not continued operation is acceptable.
- b. The results of the evaluations performed pursuant to 3.17.2.a shall be reported by telephone to NRC/OJE within 24 hours.
- 3.17.3 A Reactor Coolant System gross gamma analysis shall be performed daily. If Reactor Coolant System gross gamma activity exceeds 1.0 microcurie per milliliter whenever reactor coolant pumps are operating, a gross alpha analysis will be initiated within four hours and continued on a daily basis until gross gamma activity is less than 1.0 microcuries per milliliter. The Reactor Coolant System gross alpha concentration shall not exceed 5×10^{-3} microcuries per milliliter.
- 3.17.4 With the exception of startup and shutdown, operation is restricted to four reactor coolant pumps.
- 3.17.5 Operation of Occurrence 3, Cycle 1 shall be permitted only until September 1, 1976.
- 3.17.6 If the conditions of Specifications 3.17.1, 3.17.3 or 3.17.4 are not met, or if any abnormal indication of a loose part in the reactor vessel occurs, a reactor shutdown shall be initiated immediately and within 36 hours the reactor shall be in a condition in which no reactor coolant pumps are operating.

4.2.10 For Unit 1 Cycle 3 operation, the surveillance capsules will be removed from the reactor vessel and the provisions of Specification 4.2.9 will be revised prior to Cycle 4 operation. For Unit 3 Cycle 1 operation, the surveillance capsules will be removed from the reactor vessel for a portion of the cycle and the provisions of Specification 4.2.9 will be revised prior to Cycle 2 operation.

4.2.11 During the first two refueling periods, two reactor coolant system piping elbows shall be ultrasonically inspected along their longitudinal welds (4 inches beyond each side) for clad bonding and for cracks in both the clad and base metal. The elbows to be inspected are identified in B&W Report 1364 dated December 1970.

Bases

The surveillance program has been developed to comply with Section XI of the ASME Boiler and Pressure Vessel Code, Inservice Inspection of Nuclear Reactor Coolant Systems, 1970, including 1970 winter addenda, edition. The program places major emphasis on the area of highest stress concentrations and on areas where fast neutron irradiation might be sufficient to change material properties.

The reactor vessel specimen surveillance program for Unit 1 and Unit 2 is based on equivalent exposure times of 1.8, 19.5, 33.4 and 39.5 years. The contents of the different type of capsules are defined below.

A Type

Weld Material
HAZ Material
Baseline Material

B Type

HAZ Material
Baseline Material

For Unit 3, the Reactor Vessel Surveillance Program is based on equivalent exposure times of 1.8, 13.3, 26.7, and 30.0 years. The specimens have been selected and fabricated as specified in ASTM-E-185-72.

Early inspection of Reactor Coolant System piping elbows is considered desirable in order to reconfirm the integrity of the carbon steel base metal when explosively clad with sensitized stainless steel. If no degradation is observed during the two annual inspections, surveillance requirements will revert to Section XI of the ASME Boiler and Pressure Vessel Code.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 23 TO FACILITY LICENSE NO. DPR-38

SUPPORTING AMENDMENT NO. 23 TO FACILITY LICENSE NO. DPR-47

SUPPORTING AMENDMENT NO. 20 TO FACILITY LICENSE NO. DPR-55

DUKE POWER COMPANY

OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3

DOCKET NOS. 50-269, 50-270, AND 50-287

Introduction

By letter dated March 22, and as supplemented April 12 and 15, 1976, Duke Power Company (the licensee) requested an exemption from the requirements of 10 CFR Part 50, Appendix H, Section II.C.2 to permit the continued operation of Oconee Unit 3 for the remainder of Cycle 1 with the reactor vessel surveillance capsules removed from the reactor vessel. The licensee requested corresponding changes to the Technical Specifications appended to Facility Operating Licenses Nos. DPR-38, DPR-47 and DPR-55 for the Oconee Nuclear Station, Units 1, 2 and 3. These changes would reflect the removal of the reactor vessel surveillance capsules for the remainder of Cycle 1 operation and would require the submittal of a revised surveillance capsule withdrawal schedule prior to Cycle 2 operation. In addition, these changes would add Limiting Conditions for Operation (LCO's) for Oconee 3 Cycle 1 to minimize the possibility of further damage to the surveillance capsule holder tubes and to assure that a failed holder tube could be detected.

Discussion

The Oconee Unit 3 design includes three reactor vessel surveillance capsule holder tubes located adjacent to the reactor vessel inside wall. Each holder tube contains two surveillance capsules which hold the specimens to be irradiated in accordance with the requirements of the reactor vessel material surveillance program as described in Appendix H to 10 CFR Part 50. The purpose of the surveillance program is to monitor changes in the fracture toughness properties of ferritic materials in the reactor vessel beltline region resulting from their exposure to neutron irradiation and the thermal environment.

In a recent inspection of the surveillance capsule holder tubes, evidence of wear was observed at several locations within and on the exterior surface of the holder tubes. The damage was evidently caused by flow-induced relative motion between the holder tubes and components of the surveillance capsule train which positions and holds the surveillance capsules in place during reactor operation. In addition excessive clearance between the shroud tube and the journal bearing indicates that flow-induced relative motion exists between the shroud tube and the journal bearing. In order to minimize the possibility of further wear damage to the Oconee Unit 3 reactor vessel surveillance capsule holder tubes, the licensee is proposing that 1) the surveillance capsules and push rod assemblies be removed for the remainder of Cycle 1 operation; and 2) the Technical Specifications be revised to reflect the removal of the surveillance capsules with the provision that a revised withdrawal schedule be established prior to Cycle 2 operation and to add LCO's for Oconee Unit 3 Cycle 1 operation.

Evaluation

As required by Paragraph II.C.2 of Appendix H to 10 CFR Part 50, the surveillance capsules of Oconee Unit 3 are positioned during reactor operation such that the neutron flux received by the specimens is at least as high as, but not more than three times as high as, that received by the vessel inner surface. More specifically, as reported in Babcock and Wilcox Topical Report BAW-10100A, February 1975, the specimen capsule locations in the Unit 3 reactor vessel provide a neutron flux 2.4 times greater than the inside $\frac{1}{4}$ wall thickness ($\frac{1}{4}t$) location of the reactor vessel beltline. The lead factor between the center of the specimens and the $\frac{1}{4}t$ vessel wall location is considered when determining the relative fracture toughness properties of the beltline region materials. To date, Cycle 1 has accumulated 0.96 effective full power years (EFPY) of actual exposure for an equivalent capsule irradiation of 2.30 EFPY. Total Cycle 1 operation is anticipated to be approximately 1.33 EFPY and, therefore, we agree that there would be considerable margin between the present capsule irradiation of 2.30 EFPY and the maximum achievable exposure at the $\frac{1}{4}t$ reactor vessel beltline irradiation at the end of Cycle 1. The irradiation effects accumulated by the specimens to this point in Cycle 1 operation will not be altered and appropriate allowances can be made to revise the capsule withdrawal schedule and thus insure that the required data is obtained. Based on the above we conclude that the licensee's proposed action to remove the Unit 3 reactor vessel surveillance capsules for the remainder of Cycle 1 operation will not adversely affect the Unit 3 surveillance program and present no danger to the public health and safety. In addition, a type B capsule removed from Unit 3 during the present outage will be analyzed as part of the reactor vessel surveillance program and will provide data for establishing the revised withdrawal schedule.

Should the exemption request be denied operation of the plant would be prohibited until a redesigned surveillance capsule holder assembly is available. Best information presently available indicates that replacement holder assemblies will not be available prior to September 1976. The licensee has verbally advised the staff that the shutdown of Unit 3 until September would incur substantial additional generating costs that would be reflected in increased customer rates. From this, we conclude that granting of the exemption request would be in the public interest.

In summary, we have concluded that the licensee's request for exemption from the requirements of 10 CFR 50, Appendix H, is authorized by law; will not endanger life or property or the common defense and security and is otherwise in the public interest.

In a meeting held on April 14, 1976 with representatives from Duke Power Company and Babcock and Wilcox, we reviewed the results of the inspection conducted on the Unit 3 holder tubes. Areas discussed included the mechanical integrity of the holder tubes, which would remain in the core, and the possibility of further damage occurring to the holder tubes. We agree with the licensee that by removing the surveillance capsules and push rod assemblies, the major source of internal wear would be removed. However, the inspection results also indicated evidence of wear at the journal bearing area located at the bottom of the shroud tube. A review of this information suggests that this wear may be the result of flow forces on the exterior of the shroud tube. To remedy the effects of this wear, the licensee has expanded each holder tube in the journal bearing area to restore adequate journal bearing support. In summary, based on the information provided, which included data of known stress levels recorded on the holder tubes during Hot Functional Testing, and analyses of the structural strength of the holder tubes in their present condition, we agree that there is reasonable assurance that the holder tubes can remain in the core for the remainder of Cycle 1 operation without experiencing significant additional damage.

In the remote possibility that the holder tubes would experience sufficient vibration to cause complete severance of the holder tubes at any of the wear locations, it is highly unlikely that significant core damage would result or that any accident would be involved. The sections of the holder tubes would fall into the lower core plenum and be constrained from reaching the core by the core flow distributor. For the pieces to break up into pieces small enough to reach fuel assemblies, several days of operation would be necessary. It is unlikely that this could occur without being detected by the Loose Parts Monitoring (LPM) system. The reliability of the LPM system has been demonstrated. For example, a guide pin of the dimensions 3/4" X 4" was determined to be missing from a Low Pressure Injection pump on Oconee Unit 2 in July 1974. Subsequent Monitoring on the LPM system detected the presence of a metallic noise which was later confirmed to be the missing pin when the reactor vessel was inspected. Even if some small fragments reached the region of the fuel assemblies, the most significant hazard would be the localized blockage of coolant flow which could lead to overheating of some fuel elements. If the overheating led to clad damage,

it would be promptly detected by an increase in the primary coolant system activity level. Clad damage from this occurrence is very unlikely (except in a very small area) because of the open lattice design of the core which permits redistribution of coolant flow to cool the affected assembly. In addition to the above, we have considered what possible effects small fragments of the holder tubes might have on the operation of the control rods. We have concluded that it is extremely unlikely that the control rods could be affected such that their normal or emergency functions would be jeopardized. Finally, we have reviewed the effects that fragments of the holder tubes might have during a hypothetical Loss-Of-Coolant accident. We have concluded that the core flow would not be affected to any significant degree and that the bases for such an accident remain valid. In summary, the breaking up of the holder tubes is a low probability event but, should it occur, there is a very low probability of it leading to any significant consequences with respect to public health and safety. We therefore conclude that the surveillance capsule holder tubes can remain in the Unit 3 core for the remainder of Cycle 1 operation (approximately 130 days).

In order to minimize the possibility of further damage occurring to the surveillance capsule holder tubes, the licensee has proposed additional LCO's for the operation of Oconee Unit 3 for the remainder of Cycle 1 operation. The LCO's would minimize the stress the holder tubes would be subjected to and would assure the capability to detect and respond to the possible failure of the holder tubes. The additional LCO's proposed are as follows:

- 1) The Loose Parts Monitoring (LPM) must be in operation when any reactor coolant pumps are operating and shall have as a minimum two channels on the reactor vessel head service structure and one channel on the incore guide tubes.
- 2) Any abnormal indication on the LPM system must be promptly investigated and evaluated.
- 3) A reactor coolant system gross gamma analysis must be performed daily and if it exceeds 1.0 microcurie per millimeter whenever reactor coolant pumps are operating, a gross alpha analysis must be initiated within four hours and continued daily until the gross gamma activity is less than 1.0 microcuries per millimeter. Alpha concentration shall not exceed 5×10^{-5} microcuries per millimeter.
- 4) With the exception of startup and shutdown, operation is restricted to four primary coolant pumps.
- 5) Operation of Oconee 3 Cycle 1 shall be permitted only until September 1, 1976.

- 6) If the conditions of Specifications 1), 3) or 4) above are not met or if any abnormal indication of a loose part in the reactor vessel occurs, a reactor shutdown shall be initiated immediately and within 36 hours the reactor shall be in a condition in which no reactor coolant pumps are operating.

We have reviewed the proposed additional LCO's for the operation of Oconee Unit 3 and find them to be acceptable.

We have determined that these amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that these amendments involve an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental statement, negative declaration, or environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: April 16, 1976

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-269, 50-270, AND 50-287

DUKE POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Amendments No. 23, 23., and 20 to Facility Operating Licenses No. DPR-38, DPR-47, and DPR-55, respectively, issued to Duke Power Company which revised Technical Specifications for operation of the Oconee Nuclear Station, Units 1, 2, and 3, located in Oconee County, South Carolina. The amendments are effective as of the date of issuance.

These amendments allow the removal of the reactor vessel surveillance capsules from the Oconee Unit 3 reactor for a portion of Cycle 1 operation.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments is not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to the action, see (1) the application for amendment dated March 22, 1976, (2) Amendments No. 23, 23 , and 20 to Licenses No. DPR-38, DPR-47, and DPR-55, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D.C. 20555, and at the Oconee County Library, 201 South Spring Street, Walhalla, South Carolina 29691.

A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 16th day of April 1976.

FOR THE NUCLEAR REGULATORY COMMISSION


Robert A. Purple, Chief
Operating Reactors Branch #1
Division of Operating Reactors