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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

June 8, 1987

<u>fosted</u> Amdt. 159 to DPR-47

DO NOT REMOVE

Docket Nos.: 50-269, 50-270 and 50-287

Mr. H. B. Tucker, Vice President Nuclear Production Department Duke Power Company 422 South Church Street Charlotte, North Carolina 28242

Dear Mr. Tucker:

Subject: Issuance of Amendment Nos. 159, 159, and 156 to Facility Operating Licenses DPR-38, DPR-47, and DPR-55 - Oconee Nuclear Station, Units 1, 2, and 3 (TAC Nos. 63544, 63545, 63546)

The Nuclear Regulatory Commission has issued the enclosed Amendment Nos. 159, 159, and 156 to Facility Operating Licenses Nos. DPR-38, DPR-47 and DPR-55 for the Oconee Nuclear Station, Units 1, 2, and 3. These amendments consist of changes to the Station's common Technical Specifications (TSs) in response to your request dated February 10, 1986, as supplemented August 20, 1986.

The amendments modify the TSs to (1) revise TS 3.1.12.1(a), (b) and (d) to indicate that three subcooling margin monitors are now available over the previous two monitors and reflect the actual plant design and (2) delete TS 3.1.12.1(c) to no longer require a 30-day report for outages of less than 4 hours of the Operational Aid Computer. Also TS 3.1.12.1(d) has been redesignated as TS 3.1.12.1(c) because TS 3.1.12.1(c) has been deleted.

A copy of our Safety Evaluation is also enclosed. Notice of issuance of the enclosed amendments will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

Helen N. Pastis, Project Manager

Project Directorate II-3

Division of Reactor Projects - I/II

Enclosures:

Amendment No. 159 to DPR-38

2. Amendment No. 159 to DPR-47

3. Amendment No. 156 to DPR-55

4. Safety Evaluation

cc w/enclosures: See next page

Mr. H. B. Tucker Duke Power Company

Oconee Nuclear Station Units Nos. 1, 2 and 3

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Honorable James M. Phinney County Supervisor of Oconee County Walhalla, South Carolina 29621



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON. D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.159 License No. DPR-38

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 1 (the facility) Facility Operating License No. DPR-38 filed by the Duke Power Company (the licensee) dated February 10, 1986, as supplemented on August 20, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission:
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter 1:
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 3.B of Facility Operating License No. DPR-38 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 159, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

B.J. Youngblood, Director Project Directorate II-3

Project Directorate II-3 Division of Reactor Projects - I/II

Attachment: Technical Specification Changes

Date of Issuance: June 8, 1987



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON. D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-270

OCONEE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 159 License No. DPR-47

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 2 (the facility) Facility Operating License No. DPR-47 filed by the Duke Power Company (the licensee) dated February 10, 1986, as supplemented on August 20, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter 1;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.
 - 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 3.B of Facility Operating License No. DPR-47 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.159, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

B.J. Youngblood, Director Project Directorate II-3

Division of Reactor Projects - I/II

Attachment: Technical Specification Changes

Date of Issuance: June 8, 1987



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-287

OCONEE NUCLEAP STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 156 License No. DPR-55

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 3 (the facility) Facility Operating License No. DPR-55 filed by the Duke Power Company (the licensee) dated February 10, 1986, as supplemented on August 20, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission:
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter 1;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 3.B of Facility Operating License No. DPR-55 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 156, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

B.J./Youngblood, Director

Project Directorate II-3 Division of Reactor Projects - I/II

Attachment: Technical Specification Changes

June 8, 1987 Date of Issuance:

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 159 TO DPR-38

AMENDMENT NO. 159 TO DPR-47

AMENDMENT NO. 156 TO DPR-55

DOCKET NOS. 50-269, 50-270, AND 50-287

Replace the following page of the Appendix "A" Technical Specifications with the attached page. The revised page is identified by amendment number and contains a vertical line indicating the area of change.

Remove Page	Insert <u>Page</u>

3.1.12 Reactor Coolant System Subcooling Margin Monitor

Specification

- 3.1.12.1
- A minimum of two of the three Reactor Coolant System subcooling monitors (Loop "A" temperature, Loop "B" temperature, and Core temperature) shall be operable when the average RCS coolant temperature is above 300°F.
- b. If only one monitor is operable, at least one other monitor shall be restored to operable status within seven days or the unit shall be in hot shutdown within the next 12 hours.
- c. If all of the subcooling monitors are inoperable, then restore at least one monitor to operable status within 48 hours or be in at least hot shutdown within the next 12 hours.

Bases

The operability requirements of the Reactor Coolant System subcooling margin monitors ensures that sufficient information is available to the operators to provide prompt recognition of saturated conditions in the primary coolant system and advanced warning of the approach to inadequate core cooling. Guidance for these requirements was provided by the NRC letter of July 2, 1980, and derived from the implementation of the TMI-2 lessons learned program.



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 159 TO FACILITY OPERATING LICENSE NO. DPR-38

AMENDMENT NO. 159 TO FACILITY OPERATING LICENSE NO. DPR-47

AMENDMENT NO. 156 TO FACILITY OPERATING LICENSE NO. DPR-55

DUKE POWER COMPANY

OCONEE NUCLEAR STATION, UNITS 1, 2, and 3

DOCKET NOS. 50-269, 50-270 AND 50-287

I. INTRODUCTION

By letter dated February 10, 1986 as supplemented on August 20, 1986, Duke Power Company (the licensee) proposed changes to the Technical Specifications (TS) of Facility Operating License Nos. DPR-38, DPR-47 and DPR-55 for the Oconee Nuclear Station, Units 1, 2, and 3. These amendments would consist of changes to the Station's common TSs and would authorize these proposed changes (1) to revise TS 3.1.12.1(a), (b) and (d) to indicate that three subcooling margin monitors are now available over the previous two monitors and reflect the actual plant design; and (2) to delete TS 3.1.12.1(c) to no longer require a 30-day report for outages of less than 4 hours of the Operational Aid Computer. Also TS 3.1.12.1(d) has been redesignated as TS 3.1.12.1(c) because TS 3.1.12.1(c) has been deleted.

II. EVALUATION

2.1 Deletion of TS 3.1.12.1(c)

The U.S. Nuclear Regulatory Commission revised the regulations for reactor plant reporting requirements by changes to 10 CFR Part 50.72 "Immediate Notification Requirements for Operating Nuclear Power Reactors" and added a new regulation 10 CFR Part 50.73, "Licensee Event Report System." These revisions became effective on January 1, 1984. The revisions to the regulations eliminated the necessity for TSs on reporting an event which requires shutdown of the plant because of Limiting Conditions for Operation (LCOs). This revision to the regulations makes the requirement in the Oconee TSs to report an outage of the subcooling margin monitors redundant, and therefore, TS 3.1.12.1(c) may be deleted. We find deletion of TS 3.1.12.1(c) acceptable.

2.2 Revision of TS 3.1.12.1(d)

As discussed in Section 2.1 above, TS 3.1.12.1(c) has been deleted. Because of this revision the licensee has proposed to change TS 3. -2.1(d) to TS 3.1.12.1(c). We find this editorial change acceptable.

2.3 Revision to TSs 3.1.12.1.(a), (b), and (d) to reflect actual plant design.

Other changes to TS 3.1.12.1 were proposed by the licensee to reflect the actual plant design which includes three subcooling margin monitors instead of two. The monitors consist of reactor coolant loop "A" temperature, reactor coolant loop "B" temperature, and core outlet temperature. Application of the revised TSs will still assure operability of the subcooling margin monitors by "Action Statements" requiring that if only one monitor is operable another must be made operable within seven days, or be in a hot shutdown within 12 hours. Also, if all subcooling margin monitors are inoperable, then one monitor must be restored to operable status within 40 hours, or be in a hot shutdown within 12 hours. Based upon maintaining the assurance of operability of the subcooling monitors, we find these proposed changes acceptable.

2.4 Summary

Therefore, we find acceptable the proposed amendments to delete TS 3.1.12.1(c) which allows inoperability of two subcooling margin monitors because of the Operational Aid Computer being out-of-service for less than four hours without having to prepare a report per TS 6.6.2; to rewrite TSs 3.1.12.1(a), 3.1.12.1(b), and 3.1.12.1(d) to reflect the actual plant design and include three reactor coolant subcooling margin monitors and change TS 3.1.12.1(d) to TS 3.1.12.1(c).

III. ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

IV. CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (52 FR 16942) on May 6, 1987, and consulted with the state of South Carolina. No public comments were received, and the state of South Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: June 8, 1987

Principal Contributors: A. Cappucci

H. Pastis