

**REFERENCES**

- \* [Accession Numbers] are provided for easy retrieval of those documents that are accessible from the NRC Nuclear Documents System Advanced Design (NUDOCS/AD) or the Agencywide Documents Access and Management Systems (ADAMS)
1. SECY-81-513, "Plan for Early Resolution of Safety Issues,' August 25, 1981. [8109140067]
  2. NUREG-0371, "Task Action Plans for Generic Activities (Category A)," U.S. Nuclear Regulatory Commission, November 1978.
  3. NUREG-0471, "Generic Task Problem Descriptions (Categories B, C, and D)," U.S. Nuclear Regulatory Commission, June 1978.
  4. NUREG-0572, "Review of Licensee Event Reports (1976-1978)," U.S. Nuclear Regulatory Commission, September 1979.
  5. IE Circular No. 77-07, "Short Period During Reactor Startup," U.S. Nuclear Regulatory Commission, April 15, 1977. [9104240445]
  6. IE Bulletin No. 79-12, "Short Period Scrams at BWR Facilities," U.S. Nuclear Regulatory Commission, May 31, 1979. [7906060168]
  7. Memorandum for D. Ross from H. Richings, "RDA Statistical Analysis," June 17, 1975. [8105050833]
  8. SECY-80-325, "Special Report to Congress Identifying New Unresolved Safety Issues," July 9, 1980. [8103180932]
  9. Federal Register Notice 54 FR 16030, "Draft Regulatory Guide; Withdrawal," April 20, 1989.
  10. NUREG/CR-3992, "Collection and Evaluation of Complete and Partial Losses of Off-Site Power at Nuclear Power Plants," U.S. Nuclear Regulatory Commission, February 1985.
  11. NUREG-0800, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants," U.S. Nuclear Regulatory Commission, (1st Edition) November 1975, (2nd Edition) March 1980, (3rd Edition) July 1981.
  12. Draft Regulatory Guide and Value/Impact Statement, Task SC 708-4, "Qualification and Acceptance Tests for Snubbers Used in Systems Important to Safety," U.S. Nuclear Regulatory Commission, February 1981. [9503290322]
  13. NUREG-0691, "Investigation and Evaluation of Cracking Incidents in Piping in Pressurized Water Reactors," U.S. Nuclear Regulatory Commission, September 1980.
  14. ASME Boiler and Pressure Vessel Code, Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," American Society of Mechanical Engineers, 1974.

15. Nuclear Safety, Vol. 14, No. 3, 'Probability of Damage to Nuclear Components Due to Turbine Failure,' S. H. Busch, 1973.
16. WASH-1400 (NUREG-75/014), "Reactor Safety Study, An Assessment of Accident Risks in U.S. Commercial Nuclear Power Plants," U.S. Atomic Energy Commission, October 1975.
17. NUREG/CR-0255, "CONTEMPT-LT/028: A Computer Code for Predicting Containment Pressure-Temperature Response to a Loss-of-Coolant Accident," U.S. Nuclear Regulatory Commission, March 1979.
18. Regulatory Guide 1.46, "Protection Against Pipe Whip Inside Containment," U.S. Atomic Energy Commission, May 1973. [7907100189]
19. NUREG-0588, "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment," U.S. Nuclear Regulatory Commission, November 1979, (Rev. 1) July 1981.
20. Memorandum for R. Fraley from R. Mattson, "ACRS PWR Question Regarding Effect of Pressurizer Heater Uncovery on Pressurizer Pressure Boundary Integrity," November 5, 1979. [8004100530]
21. Regulatory Guide 1.96, "Design of Main Steam Isolation Valve Leakage Control Systems for Boiling Water Reactor Nuclear Power Plants," U.S. Nuclear Regulatory Commission, (Rev. 1) June 1976. [7907100349]
22. Memorandum for H. Denton from C. Michelson, "BWR Jet Pump Integrity," May 23, 1980. [8006180872]
23. Memorandum for Distribution from W. Minners, "Generic Issues Screening Activity," September 30, 1981. [8110190695]
24. IE Bulletin No. 79-27, "Loss of Non-Class IE Instrumentation and Control Power System Bus During Operation," U.S. Nuclear Regulatory Commission, November 30, 1979. [7910250499]
25. Memorandum for F. Schroeder from T. Novak, "Application of SRP 15.4.6 Acceptance Criteria to Operating Reactors," December 12, 1980. [8102260305]
26. IE Information Notice 80-34, "Boron Dilution of Reactor Coolant During Steam Generator Decontamination," U.S. Nuclear Regulatory Commission, September 26, 1980. [8008220239]
27. Memorandum for R. Baer from A. Thadani, "RRAB Preliminary Assessment of the Reactor Coolant Pump Seal Failure Problem," December 12, 1980. [8103050765]
28. Memorandum for T. Novak from P. Check, "Spurious Automatic Switchover of ECCS from the Injection Mode to the Recirculation Mode," January 21, 1981. [8102280446]

29. Memorandum for T. Novak, et al., from A. Thadani, "Comparative Risk Assessment of ECCS Functional Switchover Options," April 1, 1981. [8104130436]
30. Memorandum for G. Lainas, et al., from P. Check, "BWR Scram Discharge System Safety Evaluation," December 1, 1980. [8101190514]
31. Memorandum for H. Denton from M. Ernst, "DST Evaluation of the Automatic Air Header Dump on Boiling Water Reactors," December 8, 1980. [8101230203]
32. NUREG-0138, "Staff Discussion of Fifteen Technical Issues Listed in Attachment to November 3, 1976 Memorandum from Director, NRR to NRR Staff," U.S. Nuclear Regulatory Commission, November 1976.
33. Memorandum for B. Sheron from M. Srinivasan, "Probabilities and Consequences of LOCA/Loss of Offsite Power (LOOP) Sequences," April 13, 1982. [8206300420]
34. Memorandum for the Commissioners from W. Dircks, "Resolution of Issue Concerning Steam-line Break with Small LOCA," December 23, 1980. [8101150357]
35. Memorandum for S. Hanauer from T. Murley, "Diesel Generator Loading Problems Related to SIS Reset on Loss of Offsite Power," February 25, 1981. [8110190723]
36. Memorandum for C. Michelson from H. Denton, "Combination Primary/Secondary System LOCA," December 8, 1981. [8201200049]
37. NUREG/CR-2083, "Evaluation of the Threat to PWR Vessel Integrity Posed by Pressurized Thermal Shock Events," U.S. Nuclear Regulatory Commission, October 7, 1981.
38. "Generic Issues Tracking System Report," U.S. Nuclear Regulatory Commission, December 17, 1981.
39. NUREG/CR-1707, "BWR Refill-Reflood Program, Task 4.2 - Core Spray Distribution Final Report," U.S. Nuclear Regulatory Commission, March 1981.
40. NEDO-24712, "Core Spray Design Methodology Confirmation Tests," General Electric Company, August 1979.
41. Nuclear Safety, Vol. 11, No. 4, pp. 296-308, "Tornado Considerations for Nuclear Power Plant Structures Including the Spent Fuel Storage Pool," P. L. Doan, July 1970.
42. Regulatory Guide 1.76, "Design Basis Tornado for Nuclear Power Plants," U.S. Nuclear Regulatory Commission, April 1974. [7907100297]
43. Regulatory Guide 1.117, "Tornado Design Classification," U.S. Nuclear Regulatory Commission, June 1976, (Rev. 1) April 1978 [7907110104].
44. NUREG-0705, "Identification of New Unresolved Safety Issues Relating to Nuclear Power Plant Stations," U.S. Nuclear Regulatory Commission, March 1981.

45. ANSI/ANS-58.8, "Time Response Design Criteria for Nuclear Safety Related Operator Actions," American Nuclear Society, 1984.
46. NRC Memorandum and Order CLI-80-21, May 27, 1980. [8007280084]
47. Memorandum for H. Denton from C. Michelson, "Degradation of Internal Appurtenances in LWR Piping," January 19, 1981. [8102020069]
48. NUREG-0660, "NRC Action Plan Developed as a Result of the TMI-2 Accident," U.S. Nuclear Regulatory Commission, May 1980, (Rev. 1) August 1980.
49. ISA S67.04 (ANSI N719), Draft F, "Setpoints for Nuclear Safety-Related Instrumentation Used in Nuclear Power Plants," Instrument Society of America, May 22, 1979.
50. Draft Regulatory Guide and Value/Impact Statement, TASK IC 010-5, "Proposed Revision 2 to Regulatory Guide 1.105, Instrument Setpoints," U.S. Nuclear Regulatory Commission, December 1981. [8112230003]
51. Memorandum for C. Michelson from H. Denton, "BWR Jet Pump Integrity," July 11, 1980. [8009160606]
52. IE Bulletin No. 80-07, "BWR Jet Pump Assembly Failure," U.S. Nuclear Regulatory Commission, April 4, 1980. [8002280648]
53. SIL No. 330, "Jet Pump Beam Cracks," General Electric Company/BWR Product Service, June 9, 1980.
54. NUREG/CR-1659, "Reactor Safety Study Methodology Applications Program," U.S. Nuclear Regulatory Commission, (Vol. 1) April 1981, (Vol. 2) May 1981, (Vol. 3) June 1982, (Vol. 4) November 1981.
55. Regulatory Guide 1.97, "Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant and Environs Conditions During and Following an Accident," U.S. Nuclear Regulatory Commission, December 1975, (Rev. 1) August 1977 [8001240572], (Rev. 2) December 1980 [7912310387], (Rev. 3) May 1983 [8502060303].
56. Memorandum for R. Mattson, et al., from R. DeYoung, "Draft Report of Completion of Generic Activity A-34," March 28, 1979. [7904180060]
57. NUREG-0578, "TMI-2 Lessons Learned Task Force Status Report and Short-Term Recommendations," U.S. Nuclear Regulatory Commission, July 1979.
58. Memorandum for Commissioner Ahearne from H. Denton, "Instrumentation to Follow the Course of an Accident," September 4, 1979. [8005140362]
59. NUREG-0422, "SER for McGuire Nuclear Station Units 1 and 2," U.S. Nuclear Regulatory Commission, March 1978.
60. NUREG-0606, "Unresolved Safety Issues Summary," U.S. Nuclear Regulatory Commission, Vol. 7 No. 3, August 1985.

61. Memorandum for J. Murphy from B. Sheron, "Documentation of Generic Safety Issues on Degraded Voltage Protection," July 13, 1994. [9407250133]
62. NUREG/CR-2136, "Effects of Postulated Event Devices on Normal Operation of Piping Systems in Nuclear Power Plants," U.S. Nuclear Regulatory Commission, May 1981.
63. NUREG/CR-2189, "Probability of Pipe Fracture in the Primary Coolant Loop of a PWR Plant," U.S. Nuclear Regulatory Commission, September 1981.
64. NUREG/CR-2800, "Guidelines for Nuclear Power Plant Safety Issue Prioritization Information Development," U.S. Nuclear Regulatory Commission, February 1983, (Supplement 1) May 1983, (Supplement 2) December 1983, (Supplement 3) September 1985, (Supplement 4) July 1986, (Supplement 5) July 1996.
65. Memorandum for H. Denton from R. Minogue, "Research Information Letter No. 117, 'Probability of Large LOCA Induced by Earthquakes,'" April 10, 1981. [8104220512]
66. Regulatory Guide 1.6, "Independence Between Redundant Standby (Onsite) Power Sources and Between Their Distribution Systems (Safety Guide 6)," U.S. Atomic Energy Commission, March 1971. [7907100064]
67. NRC Letter to All Power Reactor Licensees from B. Grimes, "OT Position for Review and Acceptance of Spent Fuel Storage and Handling Applications," April 14, 1978. [7910310568]
68. Memorandum for R. Fraley from K. Kniel, "Draft Task Action Plan for TASKA-45, Shutdown Decay Heat Removal Requirements," May 22, 1981. [8106010652]
69. NUREG-0880, "Safety Goals for Nuclear Power Plants: A Discussion Paper," U.S. Nuclear Regulatory Commission, February 1982, (Rev. 1) May 1983.
70. NUREG-0348, "Demographic Statistics Pertaining to Nuclear Power Reactor Sites," U.S. Nuclear Regulatory Commission, November 1979.
71. Memorandum for S. Hanauer from D. Eisenhut, "Proposed Recommendations for Improving the Reliability of Open Cycle Service Water Systems," March 19, 1982. [8204190039]
72. AEOD/C202, "Report on Service Water System Flow Blockages by Bivalve Mollusks at Arkansas Nuclear One and Brunswick," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, February 1982. [8202260124]
73. TID-14844, "Calculation of Distance Factors for Power and Test Reactor Sites," U.S. Atomic Energy Commission, March 23, 1962. [8202010067]
74. AEOD/C001, "Report on the Browns Ferry 3 Partial Failure to Scram Event on June 28, 1980," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, July 30, 1980. [8008140575]

75. Memorandum for H. Denton from C. Michelson, "Engineering Evaluation of the Salt Water System (SSWS) Flow Blockage at the Pilgrim Nuclear Power Station by Blue Mussels (*Mytilus Edilus*)," May 6, 1982. [8205130114]
76. NUREG/CR-2497, "Precursors to Potential Severe Core Damage Accidents: 1969-1979, A Status Report," U.S. Nuclear Regulatory Commission, June 1982.
77. IEEE Std 352, "IEEE Guide for General Principles of Reliability Analysis of Nuclear Power Generating Station Protection Systems," The Institute of Electrical and Electronics Engineers, Inc., 1976.
78. NUREG/CR-1496, "Nuclear Power Plant Operating Experience - 1979," U.S. Nuclear Regulatory Commission, May 1981.
79. NUREG-0109, "Occupational Radiation Exposure at Light Water Cooled Power Reactors 1969-1975," U.S. Nuclear Regulatory Commission, August 1976.
80. DOE/ET/34204-43, "Dilute Chemical Decontamination Program - Final Report," U.S. Department of Energy, August 1981.
81. IEEE Std 317, "Electrical Penetration Assemblies in Containment Structures for Nuclear Power Generating Stations," The Institute of Electrical and Electronics Engineers, Inc., 1976.
82. Regulatory Guide 1.63, "Electrical Penetration Assemblies in Containment Structures for Light-Water-Cooled Nuclear Power Plants," U.S. Nuclear Regulatory Commission, October 1973, (Rev. 1) May 1977, (Rev. 2) July 1978 [7907100240], (Rev. 3) February 1987.
83. Memorandum for K. Kniel from M. Srinivasan, "Generic Issues Tracking System (GITS) B-70; Power Grid Frequency Degradation and Effect on Primary Coolant Pumps," July 31, 1981. [8109140246]
84. Memorandum for C. Berlinger from E. Butcher, "Diagnostic Evaluation at Quad Cities Nuclear Power Station (TAC Nos. M88667/M88668)," June 8, 1994. [9406130249]
85. Regulatory Guide 1.86, "Termination of Operating Licenses for Nuclear Reactors," U.S. Nuclear Regulatory Commission, June 1974. [8404100042, 9009140263]
86. Memorandum for T. Murley from M. Ernst, "Prioritization of New Requirements for PWR Feedwater Line Cracks," June 30, 1981. [8108030041]
87. Memorandum for R. Tedesco from T. Speis, "Supplement 2 to the Safety Evaluation Report for Grand Gulf Nuclear Station, Units 1 and 2," March 25, 1982. [8204080127]
88. Memorandum for All NRR Employees from H. Denton, "Regionalization of Selected NRR Functions," June 15, 1982. [9507280052]
89. NUREG/CR-1750, "Analysis, Conclusions, and Recommendations Concerning Operator Licensing," U.S. Nuclear Regulatory Commission, January 1981.

90. IEEE Std 323, "Qualifying Class 1E Equipment for Nuclear Power Generating Stations," The Institute of Electrical and Electronics Engineers, Inc., 1974.
91. Regulatory Guide 1.89, "Environmental Qualification of Certain Electric Equipment Important to Safety for Nuclear Power Plants," U.S. Nuclear Regulatory Commission, November 1974 [7907100326], (Rev. 1) June 1984 [8407110475].
92. SECY-81-504, "Equipment Qualification Program Plan," August 20, 1981. [8109220949]
93. NUREG-0611, "Generic Evaluation of Feedwater Transients and Small-Break Loss-of-Coolant Accidents in Westinghouse Designed Operating Plants," U.S. Nuclear Regulatory Commission, January 1980.
94. NUREG-0626, "Generic Evaluation of Feedwater Transients and Small Break Loss-of-Coolant Accidents in GE-Designed Operating Plants and Near-Term Operating License Applications," U.S. Nuclear Regulatory Commission, January 1980.
95. NUREG-0635, "Generic Evaluation of Feedwater Transients and Small Break Loss-of-Coolant Accidents in Combustion Engineering-Designed Operating Plants," U.S. Nuclear Regulatory Commission, February 1980.
96. NUREG-0565, "Generic Evaluation of Small Break Loss-of-Coolant Accident Behavior in Babcock & Wilcox Designed 177-FA Operating Plant," U.S. Nuclear Regulatory Commission, January 1980.
97. NUREG-0623, "Generic Assessment of Delayed Reactor Coolant Pump Trip During Small Break Loss-of-Coolant Accidents in Pressurized Water Reactors," U.S. Nuclear Regulatory Commission, November 1979.
98. NUREG-0737, "Clarification of TMI Action Plan Requirements," U.S. Nuclear Regulatory Commission, November 1980, (Supplement 1) January 1983.
99. ANSI/ANS 56.8, "Containment System Leakage Testing Requirements," American National Standards Institute, 1981.
100. Letter to General Electric Company from R. Tedesco (NRC), "Acceptance for Reference Topical Report NEDO-24712: Core Spray Design Methodology Confirmation Tests," January 30, 1981. [8103270130]
101. SECY-82-475, "Staff Resolution of the Reactor Coolant Pump Trip Issue," November 30, 1982. [8306030370]
102. NUREG/CR-0848, "Summary and Bibliography of Operating Experience with Valves in Light-Water-Reactor Nuclear Power Plants for the Period 1965-1978," U.S. Nuclear Regulatory Commission, August 1979.
103. Memorandum for M. Ernst from B. Fourest, "Review of ECCS Actuations on U.S. PWRs," June 11, 1981. [8107160006]

104. Memorandum for C. Michelson from H. Denton, "NRR Responses to AEOD Recommendations on the Arkansas Nuclear One Loss of Offsite Power Event of April 7, 1980," February 13, 1981. [8102270127]
105. Nuclear Power Experience, Volume BWR-2, Book-2, Section IX.A, Nuclear Power Experience, Inc., May 1982.
106. Memorandum for H. Denton from C. Michelson, "Lessons Learned from the Crystal River Transient of February 26, 1980 - Correcting Atmospheric Dump Valve Opening Upon Loss of Integrated Control System Power," May 23, 1980. [8009150079]
107. Memorandum for H. Denton from C. Michelson, "Potential for Unacceptable Interaction Between the Control Rod Drive System and Non-Essential Control Air System at the Browns Ferry Nuclear Plant," August 18, 1980. [8210120129]
108. Memorandum for R. Mattson from S. Hanauer, "Inadvertent Boron Dilution," March 10, 1982. [8205130278]
109. Memorandum for T. Murley from R. Mattson, "Inadvertent Boron Dilution," September 15, 1981. [8110080185]
110. NUREG/CR-2798, "Evaluation of Events Involving Unplanned Boron Dilutions in Nuclear Power Plants," U.S. Nuclear Regulatory Commission, July 1982.
111. Letter to R. Curtis (NRC) from N. DeMuth (LANL), "Analysis of Unmitigated Boron Dilution Events," November 18, 1981. [9507280099]
112. Transactions of the American Nuclear Society and the European Nuclear Society, 1984 International Conference on Nuclear Power - A Global Reality, November 11-16, 1984, Volume 47, pp. 254-256, "Analysis of Unmitigated Boron Dilution Events in Pressurized Water Reactors During Shutdown," B. Nassersharif, J. Wing (LANL).
113. EPRI NP-1194, "Operation and Design Evaluation of Main Coolant Pumps for PWR and BWR Service," Electric Power Research Institute, September 1979.
114. EPRI NP-2092, "Nuclear Unit Operating Experience, 1978 and 1979 Update," Electric Power Research Institute, October 1981.
115. NUREG/CR-3069, "Interaction of Electromagnetic Pulse with Commercial Nuclear Power Plant Systems," U.S. Nuclear Regulatory Commission, February 1983.
116. Letter to D. Switzer (Northeast Nuclear Energy Company) from G. Lear (NRC), "Millstone Nuclear Power Station Units Nos. 1 and 2," June 2, 1977. [9507280126]
117. NRC Letter to All Power Reactor Licensees (Except Humboldt Bay), "Adequacy of Station Electric Distribution Systems Voltages," August 8, 1979. [8005120354]
118. Memorandum for V. Stello from H. Denton, "Guidelines for Evaluating Qualification of Class IE Electrical Equipment in Operating Reactors," November 13, 1979. [7912190733]

119. Memorandum for K. Kniel from W. Gammill, "Need for Generic Issue B-67, Control and Monitoring of Radioactive Materials Released in Effluents and Performance of Radwaste Systems," November 20, 1981. [8201130493]
120. NUREG-0442, "Technical Report on Operating Experience with BWR Offgas Systems," U.S. Nuclear Regulatory Commission, April 1978.
121. IE Bulletin No. 78-03, "Potential Explosive Gas Mixture Accumulations Associated with BWR Offgas System Operations," U.S. Nuclear Regulatory Commission, February 8, 1978. [7909050232]
122. NRC Working Paper on Appendix J to 10 CFR Part 50, "Leak Tests for Primary and Secondary Containments of Light-Water-Cooled Nuclear Power Plants," May 17, 1982. [8401040228]
123. NRC Working Paper on Draft Regulatory Guide (MS021-5), "Containment System Leakage Testing," May 1982. [8405240527]
124. NUREG-0193, "FRANTIC - A Computer Code for Time-Dependent Unavailability Analysis," U.S. Nuclear Regulatory Commission, October 1977.
125. NRC Letter to the Northern States Power Company, "Order for Modification of License Concerning BWR Scram Discharge Systems," January 9, 1981. [8103250282]
126. Memorandum for R. Vollmer from T. Murley, "PWR Feedwater Line Cracks - New Regulatory Requirements," March 10, 1981. [8103250569]
127. Memorandum for H. Kouts from B. Rusche, "Quantification of Inherent Safety Margins in Seismic Design (SAFER-76-5)," June 7, 1976. [8003100435]
128. Memorandum for S. Levine from E. Case, "Quantification of Inherent Safety Margins in Seismic Design," June 16, 1977. [8103270937]
129. Memorandum for H. Denton from S. Levine, "RES Response to NRR User Request on Quantification of Inherent Safety Margins to Seismic Design," November 1, 1978. [8003100425]
130. Memorandum for S. Levine from H. Denton, "Seismic Safety Margins Research Program," February 23, 1979. [8003280774]
131. NUREG/CR-2015, "SSMRP Phase I Final Report," U.S. Nuclear Regulatory Commission, (Vol. 1) April 1981, (Vol. 2) July 1981, (Vol. 3) January 1983, (Vol. 4) June 1982, (Vol. 5) August 1981, (Vol. 6) October 1981, (Vol. 8) September 1984, (Vol. 9) September 1981, (Vol. 10) July 1981.
132. Memorandum for R. Minogue from H. Denton, "NRR Research Needs in Seismic Analyses Methodology," April 8, 1982. [8706040051]
133. NUREG-0784, "Long Range Research Plan FY 1984-1988," U.S. Nuclear Regulatory Commission, August 1982.

134. SECY-82-53, "Possible Relocation of Design Controlling Earthquakes in the Eastern U.S.," U.S. Nuclear Regulatory Commission, February 5, 1982. [8203050077]
135. NUREG-0484, "Methodology for Combining Dynamic Responses," U.S. Nuclear Regulatory Commission, September 1978, (Rev. 1) May 1980.
136. Memorandum for W. Minners from R. Bosnak, "Comments on Generic Issue B-6," August 26, 1982. [8209280601]
137. Memorandum for W. Minners from F. Schauer, "Generic Issue B-6," September 2, 1982. [8401170090]
138. NUREG/CR-1924, "FRANTIC II - A Computer Code for Time Dependent Unavailability Analysis," U.S. Nuclear Regulatory Commission, April 1981.
139. Letter to W. Dickhoner (The Cincinatti Gas & Electric Company) from A. Giambusso (NRC), December 18, 1972. [8709240215]
140. Memorandum for R. Frahm from R. Emrit, "Summary Report on a Risk Based Categorization of NRC Technical and Generic Issues," June 30, 1989. [9507280169]
141. Regulatory Guide 1.150, "Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice Examinations," U.S. Nuclear Regulatory Commission, June 1981 [8108040038], (Rev. 1) February 1983 [8808230046].
142. NRC Letter to Alabama Power Company, "Containment Purging During Normal Plant Operation," November 28, 1978. [7812140364]
143. NRC Letter to Nebraska Public Power District, "Containment Purging and Venting during Normal Operation," October 22, 1979. [7911190034]
144. Regulatory Guide 1.75, "Physical Independence of Electric Systems," U.S. Nuclear Regulatory Commission, February 1974, (Rev. 1) January 1975 [8605300425], (Rev. 2) September 1978 [7810050139].
145. Memorandum for D. Thatcher from R. Emrit, "Interim Criteria for Evaluating Steel Containment Buckling," June 21, 1982. [9507280196]
146. Regulatory Guide 1.133, "Loose-Part Detection Program for the Primary System of Light-Water-Cooled Reactors," U.S. Nuclear Regulatory Commission, September 1977, (Rev. 1) May 1981 [8106120320].
147. Regulatory Guide 1.20, "Comprehensive Vibration Assessment Program for Reactor Internals During Preoperational and Initial Startup Testing," U.S. Nuclear Regulatory Commission, December 1971, (Rev. 1) June 1975, (Rev. 2) May 1976 [7907100101].
148. "Memorandum of Agreement between the Institute of Nuclear Power Operations and the U.S. Nuclear Regulatory Commission," (Rev. 1) April 1, 1982. [8207010053]

149. Memorandum for J. Funches from R. Mattson, "Comments on Prioritization of Licensing Improvement Issues," February 2, 1983. [8401170099]
150. Regulatory Guide 1.47, "Bypassed and Inoperable Status Indication for Nuclear Power Plant Safety Systems," U.S. Atomic Energy Commission, May 1973. [7907100191]
151. SECY-82-111, "Requirements for Emergency Response Capability," March 11, 1982. [8203180409]
152. NUREG/CR-2417, "Identification and Analysis of Human Errors Underlying Pump and Valve Related Events Reported by Nuclear Power Plant Licensees," U.S. Nuclear Regulatory Commission, February 1982.
153. NUREG/CR-3621, "Safety System Status Monitoring," U.S. Nuclear Regulatory Commission, March 1984.
154. NRC Letter to Construction Permit Holders of B&W Designed Facilities, October 25, 1979.
155. NUREG-0667, "Transient Response of Babcock & Wilcox Designed Reactors," U.S. Nuclear Regulatory Commission, May 1980.
156. Memorandum for H. Denton from D. Eisenhut, "NUREG-0667, Transient Response of Babcock & Wilcox Designed Reactors, Implementation Plan," June 3, 1981. [8510070181]
157. Memorandum for D. Eisenhut from G. Lainas, "Status Report on Implementation of NUREG-0667 Category A Recommendations," December 15, 1981. [8201190550]
158. Memorandum for H. Denton from R. Mattson, "Review of Final Report of the B&W Reactor Transient Response Task Force (NUREG-0667)," August 8, 1980. [8010270109, 8010240413]
159. Memorandum for S. Hanauer from R. Mattson, "Design Sensitivity of B&W Reactors, Item II.E.5.1 of NUREG-0660," February 26, 1982. [8203170235]
160. Memorandum for R. Mattson from S. Hanauer, "Design Sensitivity of B&W Reactors," June 21, 1982. [8207150195]
161. NUREG/CR-1250, "Three Mile Island: A Report to the Commission and to the Public," U.S. Nuclear Regulatory Commission, January 1980.
162. NRC Letter to All Light Water Reactors, "Containment Purging and Venting During Normal Operation - Guidelines for Valve Operability," September 27, 1979. [9705190209]
163. NUREG-0305, "Technical Report on DC Power Supplies in Nuclear Power Plants," U.S. Nuclear Regulatory Commission, July 1977.
164. NUREG-0666, "A Probabilistic Safety Analysis of DC Power Supply Requirements for Nuclear Power Plants," U.S. Nuclear Regulatory Commission, April 1981.

165. Memorandum for M. Srinivasan from O. Parr and B. Sheron, "Generic Issue (GI) A-30, Adequacy of Safety Related DC Power Supplies, Development of Licensing Guidelines," March 12, 1982. [8401170018]
166. Memorandum for E. Case from R. Mattson, "Task No. D-3 Control Rod Drop Accident (BWRs)," March 6, 1978. [8001140319]
167. Federal Register Notice 44 FR 68307, "Decommissioning and Site Reclamation of Uranium and Thorium Mills," November 28, 1979.
168. NRC Letter to Arkansas Power & Light Company, "Order for Modification of License Concerning Primary Coolant System Pressure Isolation Valves," (Docket No. 50-313), April 20, 1981. [8104270071]
169. Memorandum for A. Ungaro (NRC) from F. Clark (ORNL), "Report on Standards and Requirements for Electrical Penetration Assemblies for Nuclear Reactor Containment Structures," December 13, 1978. [9507280225]
170. NUREG/CR-1345, "Nuclear Power Plant Design Concepts for Sabotage Protection," U.S. Nuclear Regulatory Commission, 1981.
171. Bulletin of the Atomic Scientists, Vol. 32, No. 8, pp. 29-36, "Nuclear Sabotage," M. Flood, October 1976.
172. Federal Register Notice 43 FR 10370, "[10 CFR Parts 30, 40, 50, and 70] Decommissioning Criteria for Nuclear Facilities, Advance Notice of Proposed Rulemaking," March 13, 1978.
173. NUREG-0586, "Final Generic Environmental Impact Statement on Decommissioning of Nuclear Facilities," U.S. Nuclear Regulatory Commission, August 1988.
174. NUREG-0585, "TMI Lessons Learned Task Force Final Report," U.S. Nuclear Regulatory Commission, October 1979.
175. ZAR-791030-01, "Report of the President's Commission on the Accident at Three Mile Island," J. G. Kemeny, et al., November 30, 1979.
176. Memorandum for J. Ahearn from M. Carbon, "Comments on the Pause in Licensing," December 11, 1979. [8001080218]
177. Memorandum for N. Moseley from J. Allan, "Operations Team Recommendations - IE/TMI Unit 2 Investigation," October 16, 1979. [8007160815]
178. EPRI NP-801, "ATWS: A Reappraisal, Part III, Frequency of Anticipated Transients," Electric Power Research Institute, July 1978.
179. NUREG-0020, "Licensed Operating Reactors, Status Summary Report," U.S. Nuclear Regulatory Commission, (Vol. 6, No. 2) February 1982.
180. NUREG-0580, "Regulatory Licensing Status Summary Report," U.S. Nuclear Regulatory Commission, (Vol. 11, No. 5) June 1982.

181. SECY-82-155, "Public Law 96-295, Section 307(B), Study of the Feasibility and Value of Licensing of Nuclear Plant Managers and Senior Licensee Officers," April 12, 1982. [8205050080]
182. NUREG/CR-0672, "Technology, Safety, and Costs of Decommissioning a Reference Boiling Water Reactor Power Station," U.S. Nuclear Regulatory Commission, June 1980.
183. NUREG-0153, "Staff Discussion of 12 Additional Technical Issues Raised by Responses to the November 3, 1976 Memorandum from Director, NRR to NRR Staff," December 1976.
184. Memorandum for R. Vollmer from D. Eisenhut, "Transmittal of Report on Threaded Fastener Experience in Nuclear Power Plants," August 25, 1982. [8209210482]
185. Memorandum for H. Denton from C. Michelson, "AEOD Report on the St. Lucie Natural Circulation Cooldown on June 11, 1980," December 24, 1980. [8101120011]
186. NUREG-0510, "Identification of Unresolved Safety Issues Relating to Nuclear Power Plants," U.S. Nuclear Regulatory Commission, January 1979.
187. NUREG/CR-2300, "PRA Procedures Guide," U.S. Nuclear Regulatory Commission, (Vols. 1 and 2) January 1983.
188. NUREG/CR-2644, "An Assessment of Offsite, Real-Time Dose Measurements for Emergency Situations," U.S. Nuclear Regulatory Commission, April 1982.
189. Memorandum for K. Goller from R. Mattson, "Proposed Changes to Regulatory Guide 1.97," July 29, 1982. [8208060339]
190. Memorandum for W. Dircks from S. Chilk, "Staff Requirements - Affirmative Session, 11:50 a.m., Friday July 16, 1982," July 20, 1982. [8208040248, 8209010068]
191. NUREG-0799, "Draft Criteria for Preparation of Emergency Operating Procedures," U.S. Nuclear Regulatory Commission, June 1981.
192. NUREG-0899, "Guidelines for Preparation of Emergency Operating Procedures - Resolution of Comments on NUREG-0799," U.S. Nuclear Regulatory Commission, September 3, 1982.
193. Memorandum for J. Martin, et al., from L. Shao, "Division Review Request: Amendments to 10 CFR Parts 30, 40, 50, 70, and 72 on Decommissioning Criteria for Nuclear Facilities," July 7, 1982. [8209140007]
194. IEEE Std 500, "IEEE Guide to the Collection and Presentation of Electrical, Electronic, and Sensing Component Reliability Data for Nuclear Power Generating Stations," The Institute of Electrical and Electronics Engineers, Inc., 1977.
195. Memorandum for E. Adensam from R. Riggs, "Status on Reactor Coolant Pump Seal Degradation Review," December 9, 1980. [8102280212]

196. Memorandum for H. Denton from S. Hanauer, "Preliminary Ranking of NRR Generic Safety Issues," March 26, 1982. [8204280036]
197. Federal Register Notice 45 FR 37011, "Decommissioning of Nuclear Facilities Regulation (10 CFR Parts 30, 40, 50, and 70)," May 30, 1980.
198. NUREG-0698, "NRC Plans for Cleanup Operations at Three Mile Island Unit 2," U.S. Nuclear Regulatory Commission, July 1980.
199. NUREG-0683, "Final Programmatic Environmental Impact Statement Related to Decontamination and Disposal of Radioactive Wastes Resulting from the March 28, 1979 Accident at Three Mile Island Nuclear Station, Unit 2," U.S. Nuclear Regulatory Commission, March 1981.
200. IEEE Std 603, "IEEE Standard Criteria for Safety Systems for Nuclear Power Generating Stations," The Institute of Electrical and Electronics Engineers, Inc., 1980.
201. IE Bulletin No. 80-24, "Prevention of Damage Due to Water Leakage Inside Containment (October 17, 1980 Indian Point 2 Event)," U.S. Nuclear Regulatory Commission, November 21, 1980. [80082220270]
202. Memorandum for G. Cunningham et al., from K. Goller, "Proposed Amendment to Part 50 on Radiation Programs, Including ALARA," September 10, 1982. [8209300046]
203. SECY-82-157A, "Status Report on the NRR Investigation of the Effects of Electromagnetic Pulse (EMP) on Nuclear Power Plants," July 16, 1982. [8205050108]
204. NUREG-0855, "Health Physics Appraisal Program," U.S. Nuclear Regulatory Commission, March 1982.
205. NUREG-0761, "Radiation Protection Plans for Nuclear Power Reactor Licensees," U.S. Nuclear Regulatory Commission, March 1981.
206. Memorandum for L. Rubenstein from M. Ernst, "Proposed Position Regarding Containment Purge/Vent Systems," April 17, 1981. [8105260251]
207. IE Bulletin No. 81-03, "Flow Blockage of Cooling Water to Safety System Components by CORBICULA SP. (Asiatic Clam) and MYTILUS SP. (Mussel)," U.S. Nuclear Regulatory Commission, April 10, 1981. [8011040289]
208. Regulatory Guide 1.52, "Design, Testing and Maintenance Criteria for Post Accident Engineered-Safety-Feature Atmosphere Cleanup System Air Filtration and Adsorption Units of Light-Water-Cooled Nuclear Power Plants," U.S. Nuclear Regulatory Commission, June 1973, (Rev. 1) July 1976, (Rev. 2) March 1978 [7907100211].
209. Regulatory Guide 1.140, "Design, Testing, and Maintenance Criteria for Normal Ventilation Exhaust System Air Filtration and Adsorption Units of Light-Water-Cooled Nuclear Power Plants," U.S. Nuclear Regulatory Commission, March 1978, (Rev. 1) October 1979 [7911090195].

210. NUREG-0885, "U.S. Nuclear Regulatory Commission Policy and Planning Guidance," U.S. Nuclear Regulatory Commission, (Issue 1) January 1982, (Issue 2) January 1983, (Issue 3) January 1984, (Issue 4) February 1985, (Issue 5) February 1986, (Issue 6) September 1987.
211. Federal Register Notice 46 FR 764, "NRC Policy Statement on Cleanup of the Three Mile Island Plant," May 1, 1981.
212. NUREG-0772, "Technical Bases for Estimating Fission Product Behavior During LWR Accidents," U.S. Nuclear Regulatory Commission, June 1981.
213. Regulatory Guide 1.3, "Assumptions Used for Evaluating the Potential Radiological Consequences of a Loss-of-Coolant Accident for Boiling Water Reactors," U.S. Nuclear Regulatory Commission, November 1970, (Rev. 1) June 1973, (Rev. 2) June 1974 [7907100054].
214. Regulatory Guide 1.4, "Assumptions Used for Evaluating the Potential Radiological Consequences of a Loss-of-Coolant Accident for Pressurized Water Reactors," U.S. Nuclear Regulatory Commission, November 1970, (Rev. 1) June 1973, (Rev. 2) June 1974 [7907100058].
215. Memorandum for E. Sullivan from R. Bosnak, "Generic Issues," September 17, 1982. [8312290147]
216. Regulatory Guide 1.108, "Periodic Testing of Diesel Generator Units Used as Onsite Electric Power Systems at Nuclear Power Plants," U.S. Nuclear Regulatory Commission, August 1976, (Rev. 1) August 1977. [7907100397]
217. NUREG/CR-0660, "Enhancement of On-site Emergency Diesel Generator Reliability," U.S. Nuclear Regulatory Commission, February 1979.
218. Memorandum for D. Eisenhut, et al., from S. Hanauer, "Diesel Generator Reliability at Operating Plants," May 6, 1982. [8205280490]
219. Memorandum for S. Hanauer from R. Mattson, "Request for Prioritization of BWR Main Steam Line Isolation Valve Leakage as a Generic Issue," July 30, 1982. [8209130423]
220. IE Bulletin No. 82-23, "Main Steam Isolation Valve (MSIV) Leakage," U.S. Nuclear Regulatory Commission, July 16, 1982. [8204210393]
221. Regulatory Guide 1.48, "Design Limits and Loading Combinations for Seismic Category 1 Fluid System Components," U.S. Atomic Energy Commission, May 1973. [7907100195]
222. NUREG-0479, "Report on BWR Control Rod Drive Mechanical Failures," U.S. Nuclear Regulatory Commission, January 1979.
223. NUREG-0462, "Technical Report on Operating Experience with BWR Pressure Relief Valves," U.S. Nuclear Regulatory Commission, July 1978.

224. NUREG-0654, "Criteria for Preparation and Evaluation of Radiological Emergency Response Plans and Preparedness in Support of Nuclear Power Plants," U.S. Nuclear Regulatory Commission, February 1980, (Rev. 1) November 1980.
225. Regulatory Guide 1.33, "Quality Assurance Program Requirements (Operation)," U.S. Nuclear Regulatory Commission, November 1972, (Rev. 1) February 1977, (Rev. 2) February 1978 [7907100144].
226. Regulatory Guide 1.8, "Qualification and Training of Personnel for Nuclear Power Plants," U.S. Nuclear Regulatory Commission, March 1971, (Rev. 1) September 1975 [8801130111], (Rev. 1-R) May 1977 [7907100073], (Rev. 2) April 1987 [8907180147].
227. NUREG/CR-0130, "Technology, Safety, and Costs of Decommissioning a Reference Pressurized Water Reactor Power Station," U.S. Nuclear Regulatory Commission, June 1978.
228. SECY-81-450, "Development of a Selective Absorption System Emergency Unit," July 27, 1981. [8108140094]
229. Memorandum for T. Speis from R. Houston, "Containment Venting and Purging - Completion of TMI Action Plan Item II.E.4.4(4)," March 3, 1982. [8401170023, 8203240149]
230. Memorandum for R. Mattson from T. Speis, "Containment Purge and Venting -Completion of TMI Action Plan Item II.E.4.4(5)," April 9, 1982. [8204260021]
231. Memorandum for W. Dircks from R. Mattson, "Status Report on Containment Purge Evaluations," May 13, 1982. [8401170021]
232. SECY-81-168B, "Response to Commission Request for Information on Financial Considerations in Licensing Proceedings," July 13, 1981. [8107310227]
233. Regulatory Guide 1.26, "Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-Waste-Containing Components of Nuclear Power Plants," U.S. Nuclear Regulatory Commission, March 1972, (Rev. 1) September 1974, (Rev. 2) June 1975, (Rev. 3) February 1976.
234. Federal Register Notice 47 FR 9987, "10 CFR Part 2, General Statement of Policy and Procedure for Enforcement Actions," March 9, 1982.
235. Memorandum for H. Denton from R. DeYoung, "TMI Action Plan Items Still Pending," June 10, 1982. [8401170101]
236. Memorandum for W. Dircks from R. DeYoung, "TMI Action Plan - Completed Items," June 30, 1982. [8208110023]
237. SECY-80-366, "NRC Legislative Program for 97th Congress," August 6, 1980. [8101050634]

238. Memorandum for Chairman Hendrie, et al., from W. Dircks, "Memorandum of Agreement with INPO and NSAC on a Cooperative Relationship for the Collection and Feedback of Operational Data," June 16, 1981. [8106260511, 8106260514]
239. Memorandum for W. Dircks from V. Stello, "TMI Action Plan - Status Report," December 19, 1980. [8205260193]
240. SECY-81-153, "Nuclear Data Link," March 11, 1981. [8103240155]
241. NUREG/CR-1440, "Light Water Reactor Status Monitoring During Accident Conditions," U.S. Nuclear Regulatory Commission, May 1980.
242. NUREG/CR-2100, "Boiling Water Reactor Status Monitoring During Accident Conditions," U.S. Nuclear Regulatory Commission, May 1981.
243. NUREG/CR-2278, "Light Water Reactor Engineered Safety Features Status Monitoring," U.S. Nuclear Regulatory Commission, October 1981.
244. NUREG/CR-2147, "Nuclear Control Room Annunciators," U.S. Nuclear Regulatory Commission, October 1981.
245. Memorandum for H. Denton, et al., from R. Minogue, "Research Information Letter #RIL-124, 'Control Room Alarms and Annunciators,'" October 20, 1981. [8111130045]
246. RIL-98, "Light Water Reactor Status Monitoring During Accident Conditions," U.S. Nuclear Regulatory Commission, August 18, 1980. [8104230867]
247. NUREG/CR-5669, "Evaluation of Exposure Limits to Toxic Gases for Nuclear Reactor Control Room Operators," U.S. Nuclear Regulatory Commission, July 1991.
248. Memorandum for W. Dircks from R. DeYoung, "TMI Action Plan - Completed Items," December 28, 1981. [8205260197]
249. NUREG/CR-6210, "Computer Codes for Evaluation of Control Room Habitability (HABIT)," U.S. Nuclear Regulatory Commission, June 1996.
250. SECY-81-440, "Nuclear Power Plant Staff Working Hours," July 22, 1981. [8107290183]
251. SECY-79-330E, "Qualifications of Reactor Operators," July 30, 1979. [7910020256, 7910020279]
252. NRR-80-117, "Study of Requirements for Operator Licensing," February 4, 1982. [8203180234]
253. ANSI/ANS 3.1, "Selection, Qualification, and Training of Personnel for Nuclear Power Plants," American National Standards Institute, 1981.
254. Letter to N. Palladino (NRC) from M. Udall (Chairman, Committee on Interior and Insular Affairs, U.S. House of Representatives), June 4, 1982. [8207120246]

255. Letter to M. Udall (Chairman, Committee on Interior and Insular Affairs, U.S. House of Representatives) from N. Palladino (NRC), June 30, 1982. [8206130067]
256. Memorandum for W. Dircks from R. DeYoung, "TMI Action Plan - Completed Items," June 2, 1982. [8401170114]
257. NUREG-0728, "Report to Congress - NRC Incident Response Plan," U.S. Nuclear Regulatory Commission, September 1980.
258. NUREG-0845, "Agency Procedure for the NRC Incident Response Plan," U.S. Nuclear Regulatory Commission, March 1982.
259. Memorandum for J. Sniezek from J. Taylor, "TMI Action Plan Item II.J.1.2, Modification of Vendor Inspection Program," October 13, 1982. [8301050485]
260. SECY-81-494, "Integrated Operational Experience Reporting System," August 18, 1981. [8109110483]
261. Federal Register Notice 46 FR 53594, "NRC Regulatory Agenda," October 29, 1981.
262. BNL/NUREG-28955, "PWR Training Simulator and Evaluation of the Thermal-Hydraulic Models for Its Main Steam Supply System," Brookhaven National Laboratory, 1981.
263. BNL/NUREG-29815, "BWR Training Simulator and Evaluation of the Thermal-Hydraulic Models for Its Main Steam Supply System," Brookhaven National Laboratory, 1981.
264. BNL/NUREG-30602, "A PWR Training Simulator Comparison with RETRAN for a Reactor Trip from Full Power," Brookhaven National Laboratory, 1981.
265. Memorandum for the Commissioners from W. Dircks, "Enforcement Policy," March 18, 1980. [8005160508]
266. SECY-80-139A, "NRC Enforcement Program," August 27, 1980. [8009180277]
267. Memorandum for R. Purple from R. Minogue, "TMI Action Plan," October 24, 1980. [8011120511]
268. Memorandum for W. Dircks from V. Stello, "Assignment of Resident Inspectors to Nuclear Steam System Suppliers and Architect-Engineers," September 14, 1981. [8111030559]
269. IE Circular No. 80-15, "Loss of Reactor Coolant Pump Cooling and Natural Circulation Cooldown," U.S. Nuclear Regulatory Commission, June 20, 1980. [8005050073]
270. Memorandum for C. Michelson from H. Denton, "Report on St. Lucie Natural Circulation Cooldown," April 6, 1981. [8104150248]
271. Memorandum for J. Taylor from E. Beckjord, "Closeout of TMI Action Plan Task I.D.5(5), Research on Disturbance Analysis Systems," April 17, 1995. [9705190216]

272. Memorandum for J. Gagliardo from D. Eisenhut, "Potential Failure of Turbine Driven Auxiliary Feedwater Pump Steam Supply Line - Fort Calhoun," October 8, 1982. [8210290122]
273. Memorandum for H. Denton from C. Michelson, "Technical Review Report, Postulated Loss of Auxiliary Feedwater System Resulting from Turbine Driven Auxiliary Feedwater Pump Steam Supply Line Rupture," February 16, 1983. [8303040296]
274. Letter to G. Knighton (NRC) from K. Baskin (Southern California Edison Company), "Docket Nos. 50-361 and 50-362, San Onofre Nuclear Generating Station Units 2 and 3," October 29, 1982. [8211020483]
275. NUREG/CR-1614, "Approaches to Acceptable Risk: A Critical Guide," U.S. Nuclear Regulatory Commission, September 1980.
276. NUREG/CR-1539, "A Methodology and a Preliminary Data Base for Examining the Health Risks of Electricity Generation from Uranium and Coal Fuels," U.S. Nuclear Regulatory Commission, August 1980.
277. NUREG/CR-1930, "Index of Risk Exposure and Risk Acceptance Criteria," U.S. Nuclear Regulatory Commission, February 1981.
278. NUREG/CR-1916, "A Risk Comparison," U.S. Nuclear Regulatory Commission, February 1981.
279. NUREG/CR-2040, "A Study of the Implications of Applying Quantitative Risk Criteria in the Licensing of Nuclear Power Plants in the U.S.," U.S. Nuclear Regulatory Commission, March 1981.
280. SECY-80-331, "NRC Training Program," July 14, 1980. [8009100166]
281. Memorandum for H. Denton, et al., from C. Michelson, "Case Study Report - Failure of Class 1E Safety-Related Switchgear Circuit Breakers to Close on Demand," August 4, 1982. [8208240007]
282. Memorandum for C. Michelson from H. Denton, "AEOD Preliminary Report on Failures of Class 1E Safety-Related Switchgear Circuit Breakers to Close on Demand," September 23, 1982. [8210290150]
283. Federal Register Notice 47 FR 36099, "Executive Order 12379 of August 17, 1982, Termination of Boards, Committees, and Commissions," August 19, 1982.
284. Letter to N. Palladino (NRC) from G. Keyworth (OSTP), July 21, 1982. [9705190213]
285. Letter to G. Keyworth (OSTP) from N. Palladino (NRC), July 23, 1982. [9705190203]
286. Letter to T. Pestorius (OSTP) from R. Minogue (NRC), August 27, 1982. [9104170201]
287. SECY-81-600A, "Revised General Statement of Policy and Procedure for Enforcement Actions," December 14, 1981. [8201190600]

288. NEDO-10174, "Consequences of a Postulated Flow Blockage Incident in a Boiling Water Reactor," General Electric Company, October 1977, (Rev. 1) May 1980.
289. NUREG/CR-2075, "Standards for Psychological Assessment of Nuclear Facility Personnel," U.S. Nuclear Regulatory Commission, July 1981.
290. NUREG/CR-2076, "Behavioral Reliability Program for the Nuclear Industry," U.S. Nuclear Regulatory Commission, July 1981.
291. Memorandum for E. Jordan, et al., from R. Bernero, "Proposed Rule Review Request - 10 CFR Part 21, 'Reporting of Defects and Noncompliance,'" September 28, 1982. [8210150634]
292. Memorandum for R. Minogue from R. DeYoung, "Proposed Rule Amending 10 CFR Parts 50.55(e) and 21: RES Task Numbers RA 128-1 and RA 808-1," July 13, 1982.
293. Federal Register Notice 47 FR 18508, "NRC Regulatory Agenda," April 29, 1982.
294. Federal Register Notice 47 FR 48960, "NRC Regulatory Agenda," October 28, 1982.
295. BNL-NUREG-31940, "Postulated SRV Line Break in the Wetwell Airspace of Mark I and Mark II Containments - A Risk Assessment," Brookhaven National Laboratory, October 1982. [8212070471]
296. Letter to T. Kress from J. Taylor, "Resolution of Generic Safety Issue 83, 'Control Room Habitability,'" September 13, 1995. [9605130222, 9605150092]
297. Memorandum for W. Dircks from R. DeYoung, "TMI Action Plan - Completed Item," October 29, 1982. [8401170104]
298. Memorandum for W. Dircks from V. Stello, "TMI Action Plan - Status Report," April 17, 1981. [8205260194]
299. NUREG/CR-1482, "Nuclear Power Plant Simulators: Their Use in Operator Training and Requalification," U.S. Nuclear Regulatory Commission, August 1980.
300. NUREG/CR-2353, "Specification and Verification of Nuclear Power Plant Training Simulator Response Characteristics," U.S. Nuclear Regulatory Commission, (Vol. 1) January 1982, (Vol. 2) May 1982.
301. Memorandum for R. Emrit from P. Goldman, "Draft Report on the Prioritization of Non-NRR TMI Action Plan Items," December 29, 1982. [8312290171]
302. Memorandum for H. Denton from C. Michelson, "Operational Restrictions for Class IE 120 VAC Vital Instrument Buses," July 15, 1980. [8210120114]
303. Memorandum for C. Michelson from H. Denton, "LCO for Class IE Vital Instrument Buses in Operating Reactors," September 29, 1980. [8010220035]

304. UCID-19469, "120 VAC Vital Instrument Buses and Inverter Technical Specifications," Lawrence Livermore National Laboratory, October 28, 1982. [8405180177]
305. Memorandum to Distribution from J. Davis, "NMSS Procedure for Review of Routine Inspection Operational Data and Licensee Event Reports," March 9, 1982. [8312290164]
306. IEEE Catalog No. TH0073-7, "Record of the Working Conference on Advanced Electrotechnology Applications to Nuclear Power Plants, January 15-17, 1980, Washington, D.C.," The Institute of Electrical and Electronics Engineers, Inc.
307. EPRI NP-2230, "ATWS: A Reappraisal, Part 3," Electric Power Research Institute, 1982.
308. SECY-82-352, "Assurance of Quality," August 20, 1982. [8209160068]
309. SECY-82-1, "Severe Accident Rulemaking and Related Matters," January 4, 1982. [8201190416]
310. Memorandum for W. Dircks from S. Chilk, "Staff Requirements - Briefing on Status and Plan for Severe Accident Rulemaking (SECY-82-1)," January 29, 1982. [8202160202]
311. SECY-82-1A, "Proposed Commission Policy Statement on Severe Accidents and Related Views on Nuclear Reactor Regulation," July 16, 1982. [8208040432]
312. NUREG/CR-0165, "A Value-Impact Assessment of Alternate Containment Concepts," U.S. Nuclear Regulatory Commission, June 1978.
313. NUREG/CR-2063, "Effects of the Accident of Three Mile Island on Property Values and Sales," U.S. Nuclear Regulatory Commission, March 1981.
314. NUREG/CR-2749, "Socioeconomic Impacts of Nuclear Generating Stations - Three Mile Island Case Study," U.S. Nuclear Regulatory Commission, (Vol. 12) July 1982.
315. Memorandum of Understanding Between the Federal Emergency Management Agency and the Nuclear Regulatory Commission, "Incident Response," October 22, 1980. [8011170793]
316. Memorandum of Understanding Between the Nuclear Regulatory Commission and the Federal Emergency Management Agency, "Radiological Emergency Planning and Preparedness," November 4, 1980. [8012110538]
317. Memorandum for G. Lainas from F. Miraglia, "CRGR Package for MPA B-71, 120 VAC Vital Instrument Buses and Inverter Technical Specifications," November 23, 1982. [8212160596]
318. Memorandum for H. Denton, et al., from C. Michelson, "An Analysis of the Abnormal Transient Operating Guidelines (ATOG) as Applied to the April 1981 Overfill Event at Arkansas Nuclear One - Unit 1," April 9, 1982. [8204220005]
319. Memorandum for C. Michelson from D. Eisenhut, "Review of Abnormal Transient Operating Guidelines (ATOG) as Applied to the April 8, 1981 Overfill Event at Arkansas Nuclear One - Unit 1," July 30, 1982. [8208180173]

320. Memorandum for C. Michelson from H. Denton, "Review of the Case Study of the Abnormal Transient Operating Guidelines (ATOG) as Applied to the April 1981 Overfill Event at Arkansas Nuclear One - Unit 1," October 7, 1982. [8211030575]
321. Memorandum for Commissioner Ahearn from W. Dircks, "AEOD Report on Arkansas Unit 1 Overfill Event," November 1, 1982. [8211190330]
322. AEOD/C201, "Report on The Safety Concern Associated with Reactor Vessel Level Instrumentation in Boiling Water Reactors," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, January 1982. [8202180432]
323. Memorandum for C. Michelson from H. Denton, "AEOD January 1982 Report on Safety Concern Associated with Reactor Vessel Level Instrumentation in Boiling Water Reactors," March 19, 1982. [8204190068]
324. NUREG-0785, "Safety Concerns Associated with Pipe Breaks in the BWR Scram System," U.S. Nuclear Regulatory Commission, April 1981.
325. NRC Letter to All BWR Licensees, "Safety Concerns Associated with Pipe Breaks in the BWR Scram System (Generic Letter 81-20)," April 10, 1981. [8112170367]
326. NEDO-24342, "GE Evaluation in Response to NRC Request Regarding BWR Scram System Pipe Breaks," General Electric Company, April 1981. [8105070251]
327. Letter to D. Eisenhut (NRC) from G. Sherwood (GE), "NRC Report, 'Safety Concerns Associated with Pipe Breaks in the BWR Scram System,'" April 30, 1981. [8105070249]
328. NUREG-0803, "Generic Safety Evaluation Report Regarding Integrity of BWR Scram System Piping," U.S. Nuclear Regulatory Commission, August 1981.
329. NRC Letter to All GE BWR Licensees (Except Humboldt Bay), "Safety Concerns Associated with Pipe Breaks in the BWR Scram System (Generic Letter 81-34)," August 31, 1981. [8110150121]
330. AEOD/C003, "Report on Loss of Offsite Power Event at Arkansas Nuclear One, Units 1 and 2, on April 7, 1980," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, October 15, 1980. [8011170099]
331. Memorandum for C. Michelson from H. Denton, "NRR Responses to AEOD Recommendations on the Arkansas Loss of Offsite Power Event of April 7, 1980," February 13, 1981. [8102270127]
332. NRC Letter to All BWR Applicants for CPs, Holders of CPs, and Applicants for OLs, "Safety Concerns Associated with Pipe Breaks in the BWR Scram System (Generic Letter 81-35)," August 31, 1981. [8112170388]
333. SECY-82-445, "Proposal to Assign Two Resident Inspectors to Each Reactor Construction Site," November 1, 1982. [8211190003]
334. SECY-82-478, "Resident Inspection Program," December 6, 1982. [8401250359]

335. Memorandum for J. Taylor from D. Morrison, "Resolution of Generic Safety Issue 83, 'Control Room Habitability,'" June 17, 1996. [9607250277]
336. NUREG-0834, "NRC Licensee Assessments," U.S. Nuclear Regulatory Commission, August 1981.
337. NUREG/CR-2672, "SBLOCA Outside Containment at Browns Ferry Unit One - Accident Sequence Analysis," U.S. Nuclear Regulatory Commission, November 1982.
338. NUREG/CR-2744, "Human Reliability Data Bank for Nuclear Power Plant Operations," U.S. Nuclear Regulatory Commission, November 1982.
339. NUREG/CR-1278, "Handbook of Human Reliability Analysis with Emphasis on Nuclear Power Plant Applications," U.S. Nuclear Regulatory Commission, October 1983.
340. Memorandum for H. Denton from J. Fouchard, "Draft Report on the Prioritization of Non-NRR TMI Action Plan Items," January 17, 1983. [8302030055]
341. NUREG/CR-2255, "Expert Estimation of Human Error Probabilities in Nuclear Power Plant Operations: A Review of Probability Assessment and Scaling," U.S. Nuclear Regulatory Commission, May 1982.
342. NUREG/CR-2743, "Procedures for Using Expert Judgment to Estimate Human Error Probabilities in Nuclear Power Plant Operations," U.S. Nuclear Regulatory Commission, February 1983.
343. NUREG/CR-2254, "Workbook for Conducting Human Reliability Analysis," U.S. Nuclear Regulatory Commission, February 1983.
344. NUREG/CR-1205, "Data Summaries of Licensee Event Reports of Pumps at U.S. Commercial Nuclear Power Plants," U.S. Nuclear Regulatory Commission, January 1980, (Rev. 1) January 1982.
345. Memorandum for R. Vollmer from T. Murley, "Prioritization of New Requirements for PWR Feedwater Line Cracks," July 21, 1981. [8108180001]
346. NUREG/CR-1363, "Data Summaries of Licensee Event Reports of Valves at U.S. Commercial Nuclear Power Plants," U.S. Nuclear Regulatory Commission, June 1980, (Rev. 1) October 1982.
347. NUREG/CR-6316, "Guidelines for the Verification and Validation of Expert System Software and Conventional Software," U.S. Nuclear Regulatory Commission, (Vols. 1, 2, 3, 4, 5, 6, 7, and 8) March 1995.
348. NUREG/CR-1362, "Data Summaries of Licensee Event Reports of Diesel Generators at U.S. Commercial Nuclear Power Plants, January 1, 1976 through December 31, 1978," U.S. Nuclear Regulatory Commission, March 1980.

349. NUREG/CR-1331, "Data Summaries of Licensee Event Reports of Control Rods and Drive Mechanisms at U.S. Commercial Nuclear Power Plants, January 1, 1972 through April 30, 1978," U.S. Nuclear Regulatory Commission, February 1980.
350. NUREG/CR-1730, "Data Summaries of Licensee Event Reports of Primary Containment Penetrations at U.S. Commercial Nuclear Power Plants, January 1, 1972 through December 31, 1978," U.S. Nuclear Regulatory Commission, September 1980.
351. NUREG/CR-1740, "Data Summaries of Licensee Event Reports of Selected Instrumentation and Control Components at U.S. Commercial Nuclear Power Plants from January 1, 1976 to December 31, 1978," U.S. Nuclear Regulatory Commission, May 1981.
352. Memorandum for C. Michelson from E. Brown, "Internal Appurtenances in LWRs," December 24, 1980. [8101150319]
353. NUREG/CR-2641, "The In-Plant Reliability Data Base for Nuclear Power Plant Components: Data Collection and Methodology Report," U.S. Nuclear Regulatory Commission, July 1982.
354. NUREG/CR-2886, "The In-Plant Reliability Data Base for Nuclear Power Plant Components: Interim Data Report - The Pump Component," U.S. Nuclear Regulatory Commission, January 1983.
355. EGG-EA-5502, "User's Guide to BFR, A Computer Code Based on the Binomial Failure Rate Common Cause Model," EG&G, Inc., July 1982.
356. EGG-EA-5623, "Common Cause Fault Rates for Instrumentation and Control Assemblies: Estimates Based on Licensee Event Reports at U.S. Commercial Nuclear Power Plants, 1976-1978," EG&G, Inc., (Rev. 1) September 1982.
357. EGG-EA-5485, "Common Cause Fault Rates for Valves: Estimates Based on Licensee Event Reports at U.S. Commercial Nuclear Power Plants, 1976-1980," EG&G, Inc., (Rev. 1) September 1982.
358. NUREG/CR-2099, "Common Cause Fault Rates for Diesel Generators: Estimates Based on Licensee Event Reports at U.S. Commercial Nuclear Power Plants, 1976-1978," U.S. Nuclear Regulatory Commission, (Rev. 1) June 1982.
359. NUREG/CR-1401, "Estimators for the Binomial Failure Rate Common Cause Model," U.S. Nuclear Regulatory Commission, April 1980.
360. EGG-EA-5289, "Common Cause Fault Rates for Pumps: Estimates Based on Licensee Event Reports at U.S. Commercial Nuclear Power Plants, January 1, 1972 through September 30, 1980," EG&G, Inc., (Rev. 1) August 1982.
361. JBFA-101-82, "Common Cause Screening Methodology Project (FY 81 Technical Progress Report)," JBF Associates, Inc., February 1982.

362. NUREG/CR-2542, "Sensitivity Study Using the FRANTIC II Code for the Unavailability of a System to the Failure Characteristics of the Components and the Operating Conditions," U.S. Nuclear Regulatory Commission, February 1982.
363. NUREG/CR-2332, "Time Dependent Unavailability of a Continuously Monitored Component," U.S. Nuclear Regulatory Commission, August 1981.
364. Memorandum for W. Dircks from S. Chilk, "Systematic Assessment of Licensee Performance," October 20, 1981. [8210080207]
365. NUREG/CR-2515, "Crystal River 3 Safety Study," U.S. Nuclear Regulatory Commission, December 1981.
366. NUREG/CR-2787, "Interim Reliability Evaluation Program: Analysis of the Arkansas Nuclear One - Unit One Nuclear Power Plant," U.S. Nuclear Regulatory Commission, June 1982.
367. NUREG/CR-2802, "Interim Reliability Evaluation Program: Analysis of the Browns Ferry Unit 1 Nuclear Plant," U.S. Nuclear Regulatory Commission, August 1982, (Appendix A) August 1982, (Appendix B) August 1982, (Appendix C) August 1982.
368. Memorandum for ACRS Members from C. Michelson, "Failure of a Feedwater Flow Straightener at San Onofre Nuclear Station, Unit 1," June 13, 1979. [7910180473]
369. SECY-82-396A, "Withdrawal of SECY-82-396 (Federal Policy Statement on Use of Potassium Iodide)," October 15, 1982. [8211040047]
370. SECY-81-676, "Delegation of Rulemaking Authority to the EDO," December 3, 1981. [8201110403]
371. SECY-82-187, "Revised Guidelines for Value-Impact Analyses," May 7, 1982. [8205130275]
372. SECY-82-447, "Draft Report of the Regulatory Reform Task Force," November 3, 1982. [8211160547]
373. NUREG-0499, "Preliminary Statement on General Policy for Rulemaking to Improve Nuclear Power Plant Licensing," U.S. Nuclear Regulatory Commission, December 1978.
374. Memorandum for J. Hendrie from L. Bickwit, "Review of Commission Delegation of Authority," October 4, 1979. [8001150518]
375. Memorandum for R. Minogue from R. Bernero, "Charter of the Regulatory Analysis Branch," October 9, 1981. [8110280720]
376. NRC Letter to All Licensees of Operating Reactors, Applicants for Operating Licenses, and Holders of Construction Permits, "Supplement 1 to NUREG-0737, Requirements for Emergency Response Capability (Generic Letter No. 82-33)," December 17, 1982. [8212060349]

377. Memorandum for W. Minners from B. Snyder, "Schedule for Resolving and Completing Generic Issues," December 16, 1982. [8312290162]
378. Memorandum for S. Boyd from M. Srinivasan, "FY 1983-FY 1984 Office of Nuclear Reactor Regulation Operating Plan," November 17, 1982. [8301100332]
379. Memorandum for H. Denton from R. DeYoung, "Draft Report on the Prioritization of Non-NRR TMI Action Plan Items," January 24, 1983. [8401160474]
380. SECY-93-049, "Implementation of 10 CFR Part 54, 'Requirements for Renewal of Operating Licenses for Nuclear Power Plants,'" March 1, 1993. [9303030337, 9303100375].
381. Memorandum for W. Minners from O. Parr, "Prioritization of Proposed Generic Issue on CRD Accumulator Check Valve Leakage," August 13, 1984. [8408280264]
382. Memorandum for W. Minners from R. Mattson, "Schedules for Resolving and Completing Generic Issues," January 21, 1983. [8301260532]
383. Memorandum for W. Dircks from R. Mattson, "Closeout of TMI Action Plan I.C.1(4), Confirmatory Analyses of Selected Transients," November 12, 1982. [8212080586]
384. Memorandum for T. Speis from R. Vollmer, "Schedules for Resolving and Completing Generic Issues," February 1, 1983. [8401170076]
385. Memorandum for T. Murley from D. Ross, "Use of Equipment Not Classified as Essential to Safety in BWR Transient Analysis," March 10, 1981. [8103240798, 9804090138]
386. Memorandum for T. Novak from R. Frahm, "Summary of Meeting with General Electric on the Use of Non-Safety Grade Equipment," March 7, 1979. [7903220463]
387. NUREG-0410, "NRC Program for the Resolution of Generic Issues Related to Nuclear Power Plants," U.S. Nuclear Regulatory Commission, January 1978.
388. NUREG-0577, "Potential for Low Fracture Toughness and Lamellar Tearing in PWR Steam Generator and Reactor Coolant Pump Supports," U.S. Nuclear Regulatory Commission, (Rev. 1) October 1983.
389. "Indian Point Probabilistic Safety Study," Power Authority of the State of New York and Consolidated Edison Company of New York, Inc., 1982.
390. NUREG-0850, "Preliminary Assessment of Core Melt Accidents at the Zion and Indian Point Nuclear Power Plants and Strategies for Mitigating Their Effects," U.S. Nuclear Regulatory Commission, November 1981.
391. Memorandum for E. Reeves from J. Knight, "Zion Liquid Pathway Analysis," August 8, 1980. [8008210647]
392. Memorandum for J. Funches from R. Mattson, "Request for Approval to Work on Low Priority Generic Safety Issues," November 5, 1982. [8211160524]

393. "TMI-2 Recovery Program Estimate," General Public Utilities Corp., (Rev. 1) July 1981.
394. Memorandum for S. Hanauer, et al., from D. Eisenhut, "Operating Reactor Event Memorandum No. 81-31: Loss of Direct Current (DC) Bus at Millstone Unit 2," March 31, 1981. [8104100493]
395. Memorandum for H. Denton from C. Michelson, "Millstone Unit 2 - Reactor Trip Following De-Energization of a 125 V DC Bus," November 5, 1981. [8112010276]
396. Memorandum for C. Michelson from H. Denton, "AEOD November 1981 Report on the Millstone Unit 2 Loss of 125 V DC Bus Event," January 4, 1982. [8202040017]
397. IEEE Std 279, "Criteria for Protection Systems for Nuclear Power Generating Stations (ANSI N42.7-1972)," The Institute of Electrical and Electronics Engineers, Inc., 1971.
398. Memorandum for R. Tedesco from T. Speis, "Identification of Protection System Instrument Sensing Lines," April 29, 1982. [8205270511]
399. Memorandum to C. Rossi from A. Thadani, "Prioritization of and Transfer of Responsibility for Generic Safety Issue 156.6.1, 'Pipe Break Effects on Systems and Components Inside Containment,'" July 16, 1999. [9908300234]
400. Memorandum for V. Stello from H. Denton, "Standard Review Plan Guidance for Identification of Protection System Instrument Lines," December 29, 1982. [8301070067]
401. Memorandum for H. Denton from V. Stello, "Proposed Standard Review Plan Guidance for Identification of Protection System Instrument Lines," January 27, 1983. [8302180526]
402. Letter to D. Eisenhut (NRC) from T. Dente (BWR Owners' Group), "Analysis of Scram Discharge Volume System Piping Integrity, NEDO-22209 (Prepublication Form)," August 23, 1982. [8208310340]
403. Letter to K. Eccleston (NRC) from T. Dente (BWR Owners' Group), "Transmittal of Supporting Information on Application of Scram Time Fraction to Scram Discharge Volume (SDV) Pipe Break Probability as Used in NEDO-22209," January 28, 1983. [8302010525]
404. Letter to S. Israel (NRC) from J. Hickman (SNL), "Review and Evaluation of the Indian Point Probabilistic Safety Study," August 25, 1982. [8209230166]
405. Memorandum for W. Minners from A. Thadani, et al., "Probability of Core Melt Due to Component Cooling Water System Failures," January 19, 1983. [8301270522]
406. Memorandum for W. Dircks from R. DeYoung, "TMI Action Plan - Status Report," March 4, 1982. [8204290601]
407. Memorandum for W. Dircks from R. DeYoung, "TMI Action Plan - Completed Item," May 11, 1982. [8401170108]
408. NUREG-1509, "Radiation Effects on Reactor Pressure Vessel Supports," U.S. Nuclear Regulatory Commission, May 1996.

409. Memorandum for W. Minners from W. Mills, "Prioritization of Generic Issue III.D.3.5, Radiation Worker Data Base," February 22, 1983. [9705190229]
410. Memorandum for T. Speis from R. Browning, "Draft Report on the Prioritization of Non-NRR TMI Action Plan Items," April 1, 1983. [8304200629, 9705190233]
411. SLI-8211, "Review of BWR Reactor Vessel Water Level Measurement Systems," S. Levy, Inc., July 1982.
412. Memorandum for T. Speis from J. Funches, "Prioritization of Generic Issues - Environmental and Licensing Improvements," February 24, 1983. [8303090540]
413. Memorandum for D. Eisenhut from E. Jordan, "Main Steam Isolation Valve (MSIV) Survey," July 1, 1982. [8209240107]
414. Memorandum for W. Minners from L. Hulman, "Consequence Analyses for BWR Main Steam System Leakage Pathway Generic Issue Evaluation," December 9, 1982. [8301050058]
415. Memorandum for W. Minners from L. Hulman, "MSIV Leakage Consequences," December 23, 1982. [8312290172]
416. NUREG/CR-1908, "Criteria for Safety-Related Nuclear Power Plant Operator Actions: Initial Pressurized Water Reactor (PWR) Simulator Exercises," U.S. Nuclear Regulatory Commission, September 1981.
417. NUREG/CR-2598, "Nuclear Power Plant Control Room Task Analysis: Pilot Study for Pressurized Water Reactors," U.S. Nuclear Regulatory Commission, July 1982.
418. NUREG/CR-2534, "Criteria for Safety-Related Nuclear Power Plant Operator Actions: Initial Boiling Water Reactor (BWR) Simulated Exercises," U.S. Nuclear Regulatory Commission, November 1982.
419. NUREG/CR-3092, "Criteria for Safety-Related Nuclear Power Plant Operator Actions: Initial Simulator to Field Data Calibration," U.S. Nuclear Regulatory Commission, February 1983.
420. IE Bulletin No. 80-14, "Degradation of BWR Scram Discharge Volume Capability," U.S. Nuclear Regulatory Commission, June 12, 1980. [8005050056]
421. IE Bulletin No. 80-17, "Failure of 76 of 185 Control Rods to Fully Insert During Scram at BWR," U.S. Nuclear Regulatory Commission, July 3, 1980. [8005050076]
422. NRC Letter to All BWR Licensees, "BWR Scram Discharge System," December 9, 1980. [8102190299]
423. Memorandum for R. Mattson from D. Eisenhut, "Status of Long-Term Followup of the Indian Point Unit 2 Flooding Event," May 13, 1982. [8205240153]
424. Memorandum for F. Schroeder from T. Speis, "Designation of Inadvertent Containment Flooding as a Generic Issue," August 5, 1982. [8208120379]

425. "Zion Probabilistic Safety Study," Commonwealth Edison Company, 1981.
426. Memorandum for T. Novak from G. Lainas and V. Noonan, "NRR Input to SER on 'Indian Point Unit No. 2 Flood in Containment Due to Containment Cooler Service Water Leaks on 10/17/80,'" April 3, 1981. [8104090906]
427. Memorandum for T. Speis from R. Mattson, "Close-out of TAP-A-16, Steam Effects on BWR Core Spray Distribution (TACS-40066)," March 29, 1983. [8304130488]
428. Memorandum for W. Minners from P. Hayes, "Generic Safety Issue No. 51, Improved Reliability of Open Service Water Systems," April 5, 1983. [9705190249]
429. Memorandum for J. Knight from E. Sullivan, "Review ACRS Consultant Report," January 10, 1980. [8105150033]
430. Memorandum for K. Seyfrit from E. Imbro, "Flow Blockage in Essential Raw Cooling Water System Due to Asiatic Clam Intrusion," March 28, 1983. [8305230539]
431. EPRI NP-1138, "Limiting Factor Analysis of High-Availability Nuclear Plants," Electric Power Research Institute, September 1979.
432. SECY-82-296, "Resolution of AEOD Combination LOCA Concern," July 13, 1982. [8207230202]
433. Memorandum for C. Michelson from E. Brown, "Degradation of Internal Appurtenances and/or Loose Parts in LWRs," June 15, 1982. [8207280317]
434. Memorandum for H. Denton, et al., from C. Michelson, "Flow Blockage in Essential Equipment at ANO Caused by Corbicula sp. (Asiatic Clams)," October 21, 1980. [8011060029]
435. Letter to N. Palladino from P. Shewmon, "Control Room Habitability," August 18, 1982. [8207180073]
436. Letter to J. Ray from W. Dircks, "August 18, 1982, ACRS Letter on Control Room Habitability," January 31, 1983. [8302100196]
437. Memorandum for H. Denton from R. Minogue, "Draft Report on the Prioritization of Non-NRR TMI Action Plan Items," March 29, 1983. [8401160475]
438. Memorandum for G. Cunningham, et al., from W. Dircks, "NRC Actions Required by Enactment of the Nuclear Waste Policy Act of 1982," January 19, 1983. [8507110762]
439. Regulatory Guide 1.149, "Nuclear Power Plant Simulators for Use in Operator Training," U.S. Nuclear Regulatory Commission, April 1981 [8105220400], (Rev. 1) April 1987 [8704300503, 8601160291], (Rev. 2) April 1996 [9604170117].
440. Memorandum for W. Minners from D. Ziemann, "Schedules for Resolving and Completing Generic Issues," April 5, 1983. [8304180758]

441. Memorandum for H. Denton from R. DeYoung, "Commission Paper on the Prioritization of Generic Safety Issues," April 20, 1983. [9705190224]
442. Memorandum for R. Emrit from T. Rothschild, "Establishing Priorities for Generic Safety Issues," April 21, 1983. [8312290167]
443. Memorandum for W. Dircks from R. Mattson, "Closeout of NUREG-0660 Item II.E.5.1, Design Sensitivity of B&W Plants for Operating Plants," March 15, 1983. [8304080415]
444. "Letter to Public Service Electric and Gas Company from D. Fischer (NRC) 'Meeting Summary - Salem Unit - 1 Failure of Reactor Trip Breakers,'" March 14, 1983. [8303210160]
445. NUREG-1000, "Generic Implications of ATWS Events at the Salem Nuclear Power Plant," U.S. Nuclear Regulatory Commission, (Vol. 1) April 1983, (Vol. 2) August 1983.
446. Memorandum for Chairman Ahearne from C. Michelson, "New Unresolved Safety Issues," August 4, 1980. [8010240206]
447. NUREG-0977, "NRC Fact Finding Task Force Report on the ATWS Events at Salem Nuclear Generating Station Unit 1 on February 22 and 25, 1983," U.S. Nuclear Regulatory Commission, March 1983.
448. Memorandum for F. Rowsome from S. Bryan, "Reliability Assurance - Reactor Protection System," July 23, 1981. [8109100509]
449. Memorandum for S. Hanauer from D. Eisenhut, "Potential Generic Issue: BWR Control Rod Test Requirements Following Maintenance," November 26, 1982. [8212160790]
450. Memorandum for R. Mattson from T. Speis, "Potential Generic Issues Related to Scram Systems," April 7, 1983. [8304200351]
451. Memorandum for H. Denton from C. Heltemes, "Potential Design Deficiency in Westinghouse Reactor Protection System," March 10, 1983. [8303230335]
452. Memorandum for C. Heltemes from H. Denton, "Westinghouse Reactor Protection System Design Conformance to IEEE Standard 279," May 2, 1983. [8305180398]
453. Memorandum for H. Denton, et al., from R. Mattson, "Recommended Generic Actions," April 27, 1983. [8305250017]
454. SECY-83-98E, "Salem Restart Evaluation," April 11, 1983. [8304220308]
455. NUREG-0771, "Regulatory Impact of Nuclear Reactor Accident Source Term Assumptions," U.S. Nuclear Regulatory Commission, June 1981.
456. WASH-1248, "Environmental Survey of the Uranium Fuel Cycle," U.S. Atomic Energy Commission, April 1974.

457. NUREG-0116, "Environmental Survey of the Reprocessing and Waste Management Portions of the LWR Fuel Cycle," U.S. Nuclear Regulatory Commission, October 1976.
458. NUREG-0216, "Public Comments on the Environmental Survey of the Reprocessing and Waste Management Portions of the LWR Fuel Cycle," U.S. Nuclear Regulatory Commission, March 1977.
459. NUREG-0252, "The Environmental Effects of Using Coal for Generating Electricity," U.S. Nuclear Regulatory Commission, June 1977.
460. NUREG/CR-1060, "Activities, Effects, and Impacts of the Coal Fuel Cycle for a 1,000 MWe Electric Power Generating Plant," U.S. Nuclear Regulatory Commission, February 1980.
461. NUREG-0332, "Health Effects Attributable to Coal and Nuclear Fuel Cycle Alternatives," U.S. Nuclear Regulatory Commission, November 1977.
462. NUREG/CR-0022, "Need for Power: Determination in the State Decisionmaking Process," U.S. Nuclear Regulatory Commission, March 1978.
463. NUREG/CR-0250, "Regional Econometric Model for Forecasting Electricity Demand by Sector and State," U.S. Nuclear Regulatory Commission, October 1978.
464. NUREG-0555, "Environmental Standard Review Plans for the Environmental Review of Construction Permit Applications for Nuclear Power Plants," U.S. Nuclear Regulatory Commission, May 1979.
465. NUREG-0398, "Federal-State Cooperation in Nuclear Power Plant Licensing," U.S. Nuclear Regulatory Commission, March 1980.
466. NUREG-0942, "Conducting Need-for-Power Review for Nuclear Power Plants," U.S. Nuclear Regulatory Commission, December 1982.
467. NUREG/CR-2423, "Mathematical Simulation of Sediment and Radionuclide Transport in Estuaries," U.S. Nuclear Regulatory Commission, November 1982.
468. NUREG/CR-2823, "A Review of the Impact of Copper Released into Marine and Estuarine Environments," U.S. Nuclear Regulatory Commission, November 1982.
469. NUREG/CR-0892, "Chronic Effects of Chlorination Byproducts on Rainbow Trout, *Salmo gairdneri*," U.S. Nuclear Regulatory Commission, November 1980.
470. NUREG/CR-0893, "Acute Toxicity and Bioaccumulation of Chloroform to Four Species of Freshwater Fish," U.S. Nuclear Regulatory Commission, August 1980.
471. NUREG/CR-2750, "Socioeconomic Impacts of Nuclear Generating Stations," U.S. Nuclear Regulatory Commission, July 1982.
472. NUREG/CR-2861, "Image Analysis for Facility Siting: A Comparison of Low and High-Attitude Image Interpretability for Land Use/Land Cover Mapping," U.S. Nuclear Regulatory Commission, November 1982.

473. NUREG/CR-2550, "Charcoal Performance Under Simulated Accident Conditions," U.S. Nuclear Regulatory Commission, July 1982.
474. NUREG-0700, "Guidelines for Control Room Design Reviews," U.S. Nuclear Regulatory Commission, September 1981.
475. Memorandum for W. Minners from F. Congel, "Prioritization of Generic Issue 58, Containment Flooding," May 19, 1983. [8306080295]
476. Regulatory Guide 4.2, "Preparation of Environmental Reports for Nuclear Power Stations," U.S. Nuclear Regulatory Commission, July 1976. [7908310195]
477. NUREG/CR-2692, "An Integrated System for Forecasting Electric Energy and Load for States and Utility Service Areas," U.S. Nuclear Regulatory Commission, May 1982.
478. Regulatory Guide 1.57, "Design Limits and Loading Combinations for Metal Primary Reactor Containment System Components," U.S. Nuclear Regulatory Commission, June 1973. [7907100220]
479. Regulatory Guide 1.7, "Control of Combustible Gas Concentrations in Containment Following a Loss-of-Coolant Accident," U.S. Nuclear Regulatory Commission, November 1978. [7812270049]
480. Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I," U.S. Nuclear Regulatory Commission, October 1977. [7907100401]
481. Regulatory Guide 1.35, "Inservice Inspection of Ungouted Tendons in Prestressed Concrete Containments," U.S. Nuclear Regulatory Commission, February 1973, (Rev. 1) June 1974, (Rev. 2) January 1976 [7907100149], (Rev. 3) July 1990 [7809180004].
482. Regulatory Guide 1.90, "Inservice Inspection of Prestressed Concrete Containment Structures with Grouted Tendons," U.S. Nuclear Regulatory Commission, August 1977. [7907100329]
483. ORNL-5470, "CONCEPT-5 User's Manual," Oak Ridge National Laboratory, December 1978.
484. ORNL/TM-6467, "A Procedure for Estimating Nonfuel Operation and Maintenance Costs for Large Steam-Electric Power Plants," Oak Ridge National Laboratory, January 1979.
485. NUREG/CR-2844, "Nonfuel Operation and Maintenance Costs for Large Steam-Electric Power Plants - 1982," U.S. Nuclear Regulatory Commission, September 1982.
486. Memorandum for Z. Rosztoczy, et al., from W. Anderson, "Seismic Scram," January 20, 1983. [8302100005]
487. Memorandum for G. Arndt from G. Burdick, "Review of Seismic Scram Report, UCRL-53037," March 3, 1983. [8303160092]

488. NUREG-0610, "Draft Emergency Action Level Guidelines for Nuclear Power Plants," U.S. Nuclear Regulatory Commission, September 1979.
489. NUREG/CR-2963, "Planning Guidance for Nuclear Power Plant Decontamination," U.S. Nuclear Regulatory Commission, June 1983.
490. Memorandum for H. Denton from C. Michelson, "Potential Generator Missiles - Generator Rotor Retaining Rings," March 16, 1982. [8203300270]
491. NRC Letter to All Licensees of Operating Westinghouse and CE PWRs (Except Arkansas Nuclear One - Unit 2 and San Onofre Units 2 and 3), "Inadequate Core Cooling Instrumentation System (Generic Letter No. 82-28)," December 10, 1982. [8212140103]
492. Memorandum for C. Michelson from H. Denton, "H. B. Robinson RCS Leak on January 29, 1981," June 15, 1981. [8107010140]
493. Memorandum for C. Michelson from H. Denton, "January 19, 1981, Memorandum on Degradation of Internal Appurtenances in LWR," April 30, 1981. [8105150032]
494. Memorandum for C. Michelson from H. Denton, "AEOD Preliminary Report on Calvert Cliffs Unit 1 Loss of Service Water," August 5, 1981. [8108170221]
495. Memorandum for C. Michelson from H. Denton, "Steam Generator Overfill and Combined Primary and Secondary Blowdown," May 27, 1981. [8106100241]
496. Memorandum for H. Denton from C. Michelson, "Concerns Relating to the Integrity of a Polymer Coating for Surfaces Inside Containment (IE Draft Bulletin No. 80-21)," August 29, 1980. [8210120370, 8009110599]
497. Memorandum for H. Denton and V. Stello from C. Michelson, "Immediate Action Memo: Common Cause Failure Potential at Rancho Seco - Desiccant Contamination of Air Lines," September 15, 1981. [8109280036]
498. Memorandum for C. Michelson from H. Denton, "AEOD Immediate Action Memo on Contamination of Instrument Air Lines at Rancho Seco," October 26, 1981. [8111300391]
499. Memorandum for H. Denton, et al., from C. Michelson, "Case Study Report on San Onofre Unit 1 Loss of Salt Water Cooling Event on March 10, 1980," August 12, 1980. [8208270684]
500. Memorandum for C. Michelson from H. Denton, "NRR Comments on AEOD Final Report: Case Study Report on San Onofre Unit 1 Loss of Salt Water Cooling Event of March 10, 1980," October 8, 1982. [8211030304]
501. IE Bulletin No. 79-24, "Frozen Lines," U.S. Nuclear Regulatory Commission, September 27, 1979. [7908220114]
502. Memorandum for H. Denton and V. Stello from C. Michelson, "Inoperability of Instrumentation Due to Extreme Cold Weather," June 15, 1981. [8107010161]

503. Memorandum for C. Michelson from H. Denton, "AEOD Memorandum on the Inoperability of Instrumentation Due to Extreme Cold Weather," August 14, 1981. [8109110138]
504. Draft Regulatory Guide and Value/Impact Statement, Task IC 126-5, "Instrument Sensing Lines," U.S. Nuclear Regulatory Commission, March 1982. [8204190028]
505. Regulatory Guide 1.151, "Instrument Sensing Lines," U.S. Nuclear Regulatory Commission, July 1983. [8808230051]
506. Federal Register Notice 48 FR 36029, "Regulatory Guide; Issuance, Availability," August 8, 1983.
507. Memorandum for C. Michelson from H. Denton, "Interlocks and LCO's for Redundant Class 1E Tie Breakers (Point Beach Nuclear Plant Units 1 and 2)," October 16, 1980. [8011050312]
508. Memorandum for F. Schroeder from L. Rubenstein, "Review of General Electric Topical Report NEDO-10174, Revision 1," August 18, 1982. [8209030003]
509. Memorandum for C. Michelson from H. Denton, "NRR Comments on AEOD Final Report: Survey of Valve Operator-Related Events Occurring During 1978, 1979, and 1980," August 19, 1982. [8404110426]
510. Memorandum for C. Michelson from H. Denton, "Effects of Fire Protection System Actuation on Safety-Related Equipment," August 27, 1982. [8506050357]
511. "Value-Impact Analysis of Recommendations Concerning Steam Generator Tube Degradations and Rupture Events," Science Applications, Inc., February 2, 1983.
512. Memorandum for D. Eisenhut from T. Speis, "DST Prioritization of Steam Generator Requirements," May 4, 1983. [8305230682]
513. SECY-82-186A, "Make-up Nozzle Cracking in Babcock and Wilcox (B&W) Plants," July 23, 1982. [8209300376]
514. Letter to J. Stolz (NRC) from G. Westafer (Florida Power Corporation), "Crystal River Unit 3, Docket No. 50-302, Operating License No. DPR-72, Safe End Task Force Action Plan," February 28, 1983. [8303040544]
515. Memorandum for W. Minners from D. Dilanni, "Proposed Generic Issue 'PORV and Block Valve Reliability,'" June 6, 1983. [8307050513]
516. Memorandum for W. Johnston and L. Rubenstein from T. Speis, "Failure of Resin Demineralizer Systems and Their Effects on Nuclear Power Plant Safety," August 6, 1982. [8208230475]
517. Memorandum for the Atomic Safety & Licensing Boards for: Callaway Plant, Unit 1; Comanche Peak Steam Electric Station, Units 1 & 2; and the Atomic Safety & Licensing Appeal Board for Virgil C. Summer Nuclear Station, Unit 1, from T. Novak, "Board

- Notification - Control Rod Drive Guide Tube Support Pin Failures at Westinghouse Plants (Board Notification No. 82-81)," August 16, 1982. [8209290318]
518. Memorandum for D. Eisenhut from J. Crews, "NRC Lead Responsibility for Possibly Detached Thermal Sleeves - Trojan Nuclear Plant - Docket No. 50-344," June 18, 1982. [8710220041]
  519. Memorandum for W. Minners from L. Hulman, "Generic Issue on Iodine Coolant Activity Limiting Conditions for Operation," June 10, 1983. [8307080562]
  520. NRC Letter to All Licensees of Operating Reactors, Applicants for Operating Licenses, and Holders of Construction Permits, "Required Actions Based on Generic Implications of Salem ATWS Events," (Generic Letter No. 83-28), July 8, 1983 [8307080169], (Supplement 1) October 7, 1992 [9210050243].
  521. SECY-83-248, "Generic Actions for Licensees and Staff in Response to the ATWS Events at Salem Unit 1," June 22, 1983. [8307110103]
  522. AEOD/P301, "Report on the Implications of the ATWS Events at the Salem Nuclear Power Plant on the NRC Program for Collection and Analysis of Operational Experience," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, July 1983. [8307140507]
  523. Memorandum for C. Heltemes from H. Denton, "AEOD Final Report on the Implications of the ATWS Events at the Salem Nuclear Power Plant on the NRC Program for Collection and Analysis of Operational Experience," July 21, 1983. [8307290045]
  524. Memorandum for R. Mattson from T. Speis, "Draft CRGR Package on A-30, DC Power," May 24, 1983. [8306030487]
  525. Memorandum for H. Denton from C. Heltemes, "Engineering Evaluation Report, Investigation of Backflow Protection in Common Equipment and Floor Drain Systems to Prevent Flooding of Vital Equipment in Safety-Related Compartments," March 11, 1983. [8303290078]
  526. NUREG/CR-3226, "Station Blackout Accident Analyses (Part of NRC Task Action Plan A-44)," U.S. Nuclear Regulatory Commission, May 1983.
  527. IE Information Notice No. 83-44, "Potential Damage to Redundant Safety Equipment as a Result of Backflow Through the Equipment and Floor Drain System," U.S. Nuclear Regulatory Commission, July 1, 1983 [8305110502], (Supplement 1) August 30, 1990 [9008240057].
  528. Memorandum for B. Liaw from H. Berkow, "OMB Clearance Renewal - Monitoring of Fatigue Transient Limits for Reactor Coolant System," May 13, 1983. [9705190223]
  529. Memorandum for H. Berkow from W. Minners, "OMB Clearance Renewal -Monitoring of Fatigue Transient Limits for Reactor Coolant System," June 1, 1983. [8306090456]

530. Letter to R. DeYoung (NRC) from J. Taylor (B&W), "Unanalyzed Reactor Vessel Thermal Stress During Cooldown," March 18, 1983. [8303250020]
531. Memorandum for R. Vollmer from W. Minners, "B&W Notification Concerning an Unanalyzed Reactor Vessel Thermal Stress During Cooldown," April 7, 1983. [8304140390]
532. Regulatory Guide 1.93, "Availability of Electric Power Sources," U.S. Nuclear Regulatory Commission, December 1974. [7907100337]
533. Memorandum for W. Minners from R. Bosnak, "B&W Notification Concerning an Unanalyzed Reactor Vessel Thermal Stress During Cooldown," April 26, 1983. [8305240235]
534. Memorandum for N. Palladino, et al., from D. Eisenhut, "Unanalyzed Reactor Vessel Thermal Stress During Cooldown (Board Notification #BN-83-42)," April 12, 1983. [8304220651]
535. CE-NPSD-154, "Natural Circulation Cooldown, Task 430 Final Report," Combustion Engineering, Inc., October 1981. [8304280091]
536. B&W Document No. 86-1140819-00, "Reactor Vessel Head Cooldown During Natural Circulation Cooldown Transients," Babcock & Wilcox Company, February 8, 1983. [8302160171]
537. Memorandum for W. Dircks from R. Fraley, August 18, 1982. [8207180092]
538. Memorandum for R. Fraley from H. Denton, "ACRS Inquiry on Pipe Break Effects on CRD Hydraulic Lines," October 29, 1982. [8211120045]
539. Letter to W. Dircks from J. Ebersole, "ACRS Comments Regarding Potential Pipe Break Effects on Control Rod Drive Hydraulic Lines in the Drywells of BWR Mark I and II Containments," March 16, 1983. [8303290428]
540. BNL-NUREG-28109, "Thermal-Hydraulic Effects on Center Rod Drop Accidents in a Boiling Water Reactor," Brookhaven National Laboratory, July 1980. [8101220642]
541. Memorandum for B. Sheron from C. Berlinger, "ACRS Request for Information Related to LOCA Effects on CRD Hydraulic Lines," October 19, 1982. [8211040446]
542. Memorandum for R. Mattson, et al., from D. Eisenhut, "Potential Safety Problems Associated With Locked Doors and Barriers in Nuclear Power Plants," May 31, 1983. [8306200435]
543. Memorandum for T. Speis from R. Mattson, "Proposed Generic Issue on Beyond Design Basis Accidents in Spent Fuel Pools," August 10, 1983. [8308180730]
544. NUREG/CR-0649, "Spent Fuel Heatup Following Loss of Water During Storage," U.S. Nuclear Regulatory Commission, May 1979.

545. Memorandum for Z. Rosztoczy from P. Williams, "Trip Report: International Meeting on Severe Fuel Damage and Visit to Power Burst Facility," April 25, 1983. [8305060661]
546. Memorandum to C. Rossi from D. Cool, "NMSS Input for Second Quarter FY-2000 Update of the Generic Issues Management Control System," April 18, 2000.
547. SECY-95-245, "Completion of the Fatigue Action Plan," September 25, 1995. [9509290040]
548. Memorandum for W. Dircks from R. DeYoung, "TMI Action Plan Completed Items," January 26, 1983. [8303090323]
549. NUREG/CR-2039, "Dynamic Combinations for Mark II Containment Structures," U.S. Nuclear Regulatory Commission, June 1982.
550. NUREG/CR-1890, "ABS, SRSS and CDF Response Combination Evaluation for Mark III Containment and Drywell Structures," U.S. Nuclear Regulatory Commission, June 1982.
551. Letter to N. Palladino from J. Ray, "Need for Rapid Depressurization Capability in Newer Combustion Engineering, Inc. Plants," October 18, 1983. [8311010118]
552. Memorandum for W. Minners from B. Siegel, "Proposed Generic Issue 'Reliability of Vacuum Breakers Connected to Steam Discharge Lines Inside BWR Containments,'" November 3, 1983. [8312140360]
553. Memorandum for D. Eisenhut from J. Olshinski, "Loss of High Head Injection Capability at McGuire Unit 1 and Reconsideration of Technical Specification 3.0.3 and 3.5.2," April 12, 1982. [8802120046]
554. Memorandum for D. Eisenhut, et al., from H. Denton, "Development of Generic Recommendations Based on the Review of the January 25, 1982 Steam Generator Tube Rupture at Ginna," May 3, 1982. [8205280089]
555. Letter to D. Eisenhut (NRC) from D. Waters (BWR Owners' Group), "BWR Owners' Group Evaluations of NUREG-0737 Requirements II.K.3.16 and II.K.3.18," March 31, 1981. [8104200300]
556. Memorandum for G. Lainas, et al., from W. Houston, "Evaluation of BWR Owners' Group Generic Response to Item II.K.3.16 of NUREG-0737, 'Reduction of Challenges and Failures of Relief Valves - Feasibility Study and System Modification,'" April 1, 1983. [8711060070]
557. Memorandum for H. Denton and V. Stello from C. Michelson, "Calvert Cliffs Unit 1 Loss of Service Water," June 19, 1981. [8107060505]
558. Memorandum for H. Denton and R. DeYoung from C. Michelson, "Calvert Cliffs Unit 1 Loss of Service Water on May 20, 1980," December 17, 1981. [8201150431]
559. Memorandum for C. Michelson from H. Denton, "NRR Comments on AEOD Final Report: Calvert Cliffs Unit 1 Loss of Service Water on May 20, 1980," September 23, 1982. [8210180230]

560. Memorandum for H. Denton from C. Heltemes, "Response to NRR Comments on AEOD Report, 'Calvert Cliffs Unit 1 Loss of Service Water on May 20, 1980,'" May 2, 1983. [8305110577]
561. Memorandum for W. Houston and L. Rubenstein from F. Miraglia, "Response to NRR Comments on AEOD Report, 'Calvert Cliffs Unit 1 Loss of Service Water on May 20, 1980,'" June 2, 1983. [8306080036]
562. Memorandum for F. Miraglia from W. Houston and L. Rubenstein, "Comments to AEOD Memo dated May 2, 1983, on Calvert Cliffs, Unit 1, Loss of Service Water on May 20, 1980," July 22, 1983. [8308030493]
563. Memorandum for C. Heltemes from H. Denton, "Response to NRR Comments on AEOD Report, 'Calvert Cliffs Unit 1 Loss of Service Water on May 20, 1980,'" September 15, 1983. [8309270470]
564. Memorandum for R. Baer from K. Seyfrit, "Case Study, 'Calvert Cliffs Unit 1 Loss of Service Water on May 29, 1980,'" August 18, 1983. [8308290487]
565. IE Information Notice No. 83-77, "Air/Gas Entrainment Events Resulting in System Failures," U.S. Nuclear Regulatory Commission, November 14, 1983. [8311010015]
566. Memorandum for G. Holahan from W. Minners, "Prioritization of Issue 36: Loss of Service Water at Calvert Cliffs Unit 1," November 10, 1983. [8311180373]
567. Letter to A. Lundvall (Baltimore Gas and Electric Company) from D. Eisenhut (NRC), Docket No. 50-317, September 15, 1983. [8309270504]
568. Memorandum for W. Houston and L. Rubenstein from F. Schroeder, "Request for Reactor Systems Branch and Auxiliary Systems Branch Support for Plant Visits on USI A-45," November 28, 1983. [8312150068]
569. AEOD/C102, "Engineering Evaluation of the H. B. Robinson Reactor Coolant System Leak on January 29, 1981," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, March 23, 1981. [8104150060]
570. Memorandum for V. Stello from H. Denton, "Issuance of Revised Section 7.1, Appendix A to this Section, Section 7.5 and Section 7.7 of the Standard Review Plan, NUREG-0800," March 9, 1984. [8404160228]
571. Memorandum for H. Denton from V. Stello, "SRP Changes Concerning Resolution of Generic Issue 45, Inoperability of Instrumentation due to Extreme Cold Weather," April 3, 1984. [8404180510]
572. Memorandum for G. Lainas from F. Rowsome, "Safety Evaluation of the Westinghouse Licensees' Responses to TMI Action Item II.K.3.2," July 22, 1983. [8308040054]
573. Memorandum for G. Lainas from F. Rowsome, "Safety Evaluation of the B&W Licensees' Responses to TMI Action Item II.K.3.2," August 24, 1983. [8308310422]

574. Memorandum for G. Lainas from F. Rowsome, "Safety Evaluation of the CE Licensees' Responses to TMI Action Item II.K.3.2," August 26, 1983. [8309060394]
575. Memorandum for W. Minners from B. Sheron, "Proposed Generic Issue on PORV and Block Valve Reliability," June 27, 1983. [8307180224]
576. Memorandum for R. Riggs from F. Cherny, "Comments on Draft Write-up of Prioritization of Generic Issue 70 'PORV and Block Valve Reliability,'" December 21, 1983. [8401030003]
577. Memorandum for H. Denton, et al., from C. Heltemes, "Case Study Report - Low Temperature Overpressure Events at Turkey Point Unit 4," September 26, 1983. [8310060171]
578. NUREG-0748, "Operating Reactors Licensing Actions Summary," U.S. Nuclear Regulatory Commission, (Vol. 5, No. 11) February 1986.
579. NUREG-0694, "TMI-Related Requirements for New Operating Licenses," U.S. Nuclear Regulatory Commission, June 1980.
580. NUREG-0645, "Report of the Bulletins & Orders Task Force of the Office of Nuclear Reactor Regulation," U.S. Nuclear Regulatory Commission, (Vol. 1) January 1980, (Vol. 2) January 1980.
581. NUREG-0909, "NRC Report on the January 25, 1982 Steam Generator Tube Rupture at R.E. Ginna Nuclear Power Plant," U.S. Nuclear Regulatory Commission, April 1982.
582. NUREG-0713, "Occupational Radiation Exposure at Commercial Nuclear Power Reactors - 1981," U.S. Nuclear Regulatory Commission, (Vol. 1) March 1981, (Vol. 2) December 1981, (Vol. 3) November 1982, (Vol. 4) November 1983, (Vol. 5) March 1985, (Vol. 6) September 1986, (Vol. 7) April 1988.
583. EPRI NP-2292, "PWR Safety and Relief Valve Test Program," Electric Power Research Institute, December 1982.
584. EPRI NP-1139, "Limiting Factor Analysis of High Availability Nuclear Plants," Electric Power Research Institute, August 1979.
585. EPRI P-2410-SR, "Technical Assessment Guide," Electric Power Research Institute, May 1982.
586. WCAP-9804, "Probabilistic Analysis and Operational Data in Response to NUREG-0737 Item II.K.3.2 for Westinghouse NSSS Plants," Westinghouse Electric Corporation, February 1981. [8103160257]
587. "Accident Sequence Evaluation Program, Phase II Workshop Report," Sandia National Laboratories, EG&G Idaho, Inc., and Science Applications, Inc., September 1982.
588. Letter to Director (NRR) from K. Cook (Louisiana Power & Light), "Waterford SES Unit 3, Docket No. 50-382, Depressurization and Decay Heat Removal," October 27, 1983. [8311010259]

589. Letter to W. Dircks (NRC) from E. Van Brunt (Arizona Public Service Company), "Palo Verde Nuclear Generating Station (PVNGS), Units 1, 2, and 3, Docket Nos. STN-50-528/529/530," November 7, 1983. [8312230233]
590. ALO-75 (TR-3459-1), "Pilot Program to Identify Valve Failures Which Impact the Safety and Operation of Light Water Nuclear Power Plants," Teledyne Engineering Services, January 11, 1980.
591. IE Information Notice No. 82-45, "PWR Low Temperature Overpressure Protection," U.S. Nuclear Regulatory Commission, November 19, 1982. [8208190253]
592. IE Information Notice No. 82-17, "Overpressurization of Reactor Coolant System," U.S. Nuclear Regulatory Commission, June 10, 1982. [8204210383]
593. SECY-84-76, "Proposed Rulemaking for Operator Licensing and for Training and Qualifications of Civilian Nuclear Power Plant Personnel," February 13, 1984. [8403260357]
594. Letter to E. Wilkinson (INPO) from W. Dircks (NRC), November 23, 1983. [8312090099]
595. SECY-83-52A, "Final Rulemaking Concerning Licensed Operator Staffing at Nuclear Power Units and Draft Policy Statement on Shift Crew Qualifications," March 14, 1983. [8304010029]
596. Memorandum for W. Dircks from S. Chilk, "Staff Requirements - Affirmation/ Discussion and Vote, 3:35 p.m., Thursday, April 21, 1983, Commissioners' Conference Room (Open to Public Attendance)," April 28, 1983. [9705190263]
597. Federal Register Notice 48 FR 33850, "Licensee Event Report System," July 26, 1983.
598. Memorandum for W. Dircks from R. Mattson, "Closeout of TMI Action Plan Task III.D.2.5, 'Offsite Dose Calculation Manual,'" January 17, 1984. [8402020114]
599. NUREG/CR-3332, "Radiological Assessment - A Textbook on Environmental Dose Analysis," U.S. Nuclear Regulatory Commission, September 1983.
600. NUREG-0978, "Mark III LOCA-Related Hydrodynamic Load Definition," U.S. Nuclear Regulatory Commission, August 1984.
601. Memorandum for T. Combs from H. Denton, "Revised SRP Section 6.2.1.1.C of NUREG-0800," September 10, 1984. [8409180459]
602. Memorandum for T. Speis from R. Mattson, "Status of Generic Issues 40 and 65 Assigned to DSI," December 27, 1983. [8401170445]
603. Regulatory Guide 1.45, "Reactor Coolant Pressure Boundary Leakage Detection Systems," U.S. Nuclear Regulatory Commission, May 1973. [7907100185]
604. SECY-81-641, "Electromagnetic Pulse (EMP) - Effects on Nuclear Power Plants," November 5, 1981. [8202090418, 8111250553]

605. SECY-82-157, "Status Report on the Evaluation of the Effects of Electromagnetic Pulse (EMP) on Nuclear Power Plants," April 13, 1982. [8205050108]
606. SECY-83-367, "Staff Study of Electromagnetic Pulse (EMP) Effects on Nuclear Power Plants and Discussion of Related Petitions for Rulemaking (PRM-50-32, 32A, and 32B)," September 6, 1983. [8312210152]
607. Memorandum for W. Dircks from S. Chilk, "SECY-83-367 - Staff Study of Electromagnetic Pulse (EMP) Effects on Nuclear Power Plants and Discussion of Related Petition for Rulemaking (PRM-50-32, 32A, and 32B)," November 15, 1983. [8402270019]
608. IE Information Notice No. 82-39, "Service Degradation of Thick-Walled Stainless Steel Recirculation Systems at BWR Plants," U.S. Nuclear Regulatory Commission, September 21, 1982. [8208190229]
609. IE Bulletin No. 82-03, "Stress Corrosion Cracking in Thick-Wall Large Diameter, Stainless Steel Recirculation System Piping at BWR Plants," U.S. Nuclear Regulatory Commission, October 14, 1982 [8208190238], (Rev. 1) October 28, 1982 [8208190240].
610. IE Bulletin No. 83-02, "Stress Corrosion Cracking in Large Diameter Stainless Steel Recirculation System Piping at BWR Plants," U.S. Nuclear Regulatory Commission, March 4, 1983. [8212060368]
611. NUREG-1061, "Report of the U.S. Nuclear Regulatory Commission Piping Review Committee," U.S. Nuclear Regulatory Commission, (Vol. 1) August 1984, (Vol. 2) April 1985, (Vol. 3) November 1984, (Vol. 4) December 1984, (Vol. 5) April 1985.
612. SECY-83-267, "Status Report on Observation of Pipe Cracking at BWRs," July 1, 1983. [8307250565]
613. SECY-83-267A, "Update of Status Report on Observation of Pipe Cracking at BWRs (SECY-83-267)," July 11, 1983. [8307250578]
614. SECY-83-267B, "Update of Status Report on Observation of Pipe Cracking at BWRs (SECY-83-267 and 267A)," August 8, 1983. [8308230648]
615. SECY-83-267C, "Staff Requirements for Reinspection of BWR Piping and Repair of Cracked Piping," November 7, 1983. [8311160350]
616. SECY-84-9, "Report on the Long Term Approach for Dealing with Stress Corrosion Cracking in BWR Piping," January 10, 1984. [8402230344]
617. SECY-84-9A, "Update of Status Report on BWR Pipe Cracks and Projection of Upcoming Licensee Actions," January 27, 1984. [8402230347]
618. SECY-84-166, "Update of Status Report on BWR Pipe Cracks and Projection of Upcoming Licensee Actions," April 20, 1984. [8405180011]

619. SECY-84-301, "Staff Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping," July 30, 1984. [8408090406]
620. NRC Letter to All Licensees of Operating Reactors, Applicants for Operating License, and Holders of Construction Permits for Boiling Water Reactors, "Inspections of BWR Stainless Steel Piping," (Generic Letter 84-11), April 19, 1984. [8404230029]
621. NUREG-0992, "Report of the Committee to Review Safeguards Requirements at Power Reactors," U.S. Nuclear Regulatory Commission, May 1983.
622. Memorandum for T. Speis from R. Mattson, "Fuel Crumbling During LOCA," February 2, 1983. [8302170511]
623. Memorandum for H. Denton from D. Eisenhut, "Potential Safety Problems Associated with Locked Doors and Barriers in Nuclear Power Plants," December 22, 1983. [8401130140]
624. Memorandum for D. Eisenhut from H. Denton, "Safety-Safeguards Interface," January 16, 1984. [8402010286]
625. Memorandum for H. Thompson from D. Eisenhut, "Potential Safety Problems Associated with Locked Doors and Barriers in Nuclear Power Plants," January 30, 1984. [8402140525]
626. Memorandum for T. Speis from H. Thompson, "Submittal of Potential Generic Issue Associated with Locked Doors and Barriers," June 8, 1984. [8407060042]
627. SECY-83-311, "Proposed Insider Safeguards Rules," July 29, 1983. [8308190179]
628. IE Information Notice No. 83-36, "Impact of Security Practices on Safe Operations," U.S. Nuclear Regulatory Commission, June 9, 1983. [8305110464]
629. Memorandum for H. Thompson from D. Morrison, "Closeout of Generic Safety Issue 78, 'Monitoring of Fatigue Transient Limits for Reactor Coolant System (RCS)' and Generic Safety Issue 166, 'Adequacy of Fatigue Life of Metal Components,'" February 5, 1997. [9703050391]
630. Memorandum for W. Minners from F. Miraglia, "Proposed Generic Issue - Technical Specifications for Anticipatory Trips," February 23, 1984. [8403080271]
631. Memorandum for F. Miraglia from W. Houston, "Task Interface Agreement Task No. 83-77 (TAC 40002, PA-157)," November 29, 1983. [8401060510]
632. NUREG/CR-6117, "Neutron Spectra at Different High Flux Isotope Reactor (HFIR) Pressure Vessel Surveillance Locations," U.S. Nuclear Regulatory Commission, December 1993.
633. Memorandum for P. Check from H. Richings, "Some Notes on PWR (W) Power Distribution Probabilities for LOCA Probabilistic Analyses," July 5, 1977.
634. NUREG-0630, "Cladding Swelling and Rupture Models for LOCA Analysis," U.S. Nuclear Regulatory Commission, April 1980.

635. Memorandum for G. Holahan and W. Minners from R. Mattson, "Disposition of AEOD Engineering and Technical Evaluation Reports," April 10, 1984. [9705190219]
636. Memorandum for R. DeYoung and H. Denton from C. Heltemes, "Vapor Binding of Auxiliary Feedwater Pumps," November 21, 1983. [8312070028]
637. AEOD/C404, "Steam Binding of Auxiliary Feedwater Pumps," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, July 1984. [8408060083]
638. Memorandum for H. Denton from C. Michelson, "Tie Breaker Between Redundant Class 1E Buses - Point Beach Nuclear Plant, Units 1 and 2," August 27, 1980. [8009150214, 8009160668]
639. Letter to J. Keppler (NRC) from C. Fay (Wisconsin Electric Power Company), "Docket No. 50-301, Point Beach Nuclear Plant Unit 2 Licensee Event Report No. 80-005/03L-0," June 27, 1980. [8007080381]
640. Memorandum for H. Denton from C. Heltemes, "Special Study Report - Human Error in Events Involving Wrong Unit or Wrong Train," January 13, 1984. [8401310079]
641. IE Information Notice No. 84-51, "Independent Verification," U.S. Nuclear Regulatory Commission, June 26, 1984. [8406250214]
642. IE Information Notice No. 84-58, "Inadvertent Defeat of Safety Function Caused by Human Error Involving Wrong Unit, Wrong Train, or Wrong System," U.S. Nuclear Regulatory Commission, July 25, 1984. [8407230079]
643. Memorandum for H. Denton from C. Heltemes, "Human Error in Events Involving Wrong Unit or Wrong Train," August 8, 1984. [9705190238]
644. Memorandum for D. Eisenhut, et al., from H. Thompson, "Maintenance and Surveillance Program Implementation Plan," July 7, 1984. [8407160259]
645. Memorandum for C. Heltemes from H. Denton, "Special Study Report - Human Errors in Events Involving Wrong Unit or Wrong Train," May 2, 1984. [8405170027]
646. Memorandum for C. Heltemes from H. Denton, "Human Error in Events Involving Wrong Unit or Wrong Train," September 17, 1984. [8410040282]
647. Memorandum for T. Speis from H. Denton, "Resolution of Generic Issue B-26, 'Structural Integrity of Containment Penetrations,'" September 27, 1984. [8410120090]
648. Memorandum for T. Speis from H. Denton, "Closeout of Generic Issue B-54, 'Ice Condenser Containments,'" October 22, 1984. [8411050142]
649. NUREG/CR-3716, "CONTEMPT 4/MOD 4," U.S. Nuclear Regulatory Commission, March 1984.

650. NUREG/CR-4001, "CONTEMPT 4/MOD 5," U.S. Nuclear Regulatory Commission, September 1984.
651. NUREG-0985, "U.S. Nuclear Regulatory Commission Human Factors Program Plan," U.S. Nuclear Regulatory Commission, August 1983, (Rev. 1) September 1984, (Rev. 2) April 1986.
652. Memorandum for W. Dircks from R. DeYoung, "Elimination of Duplicative Tracking Requirements for Revision of Regulatory Guide 1.33," July 26, 1984. [9705190264]
653. NUREG/CR-3123, "Criteria for Safety-Related Nuclear Power Plant Operator Actions: 1982 Pressurized Water Reactor (PWR) Simulator Exercises," U.S. Nuclear Regulatory Commission, June 1983.
654. Memorandum for W. Dircks from H. Thompson, "Closeout of TMI Action Plan Task I.G.2, 'Scope of Test Program,'" October 5, 1984. [8410160524]
655. Memorandum for W. Dircks from H. Denton, "Generic Issue II.A.1, 'Siting Policy Reformulation,'" September 17, 1984. [8410090175]
656. Memorandum for W. Dircks from H. Denton, "Closeout of TMI Action Plan Task II.E.5.2, Transient Response of B&W Designed Reactors," September 28, 1984. [8410110596]
657. Memorandum for D. Crutchfield from D. Eisenhut, "TMI Action Plan Task II.E.5.2," November 6, 1984. [8411270129]
658. NUREG-1054, "Simplified Analysis for Liquid Pathway Studies," U.S. Nuclear Regulatory Commission, August 1984.
659. Memorandum for H. Denton from R. Vollmer, "ESRP 7.1.1 'Environmental Impacts of Postulated Accidents Involving Radioactive Materials - Releases to Groundwater,'" September 25, 1984. [8410100758]
660. Memorandum for W. Dircks from H. Denton, "Generic Issue III.D.2.3 'Liquid Pathway Radiological Control,'" October 29, 1984. [8411190057]
661. Memorandum for H. Denton from C. Heltemes, "Failures of Class 1E Safety-Related Switchgear Circuit Breakers to Close on Demand," April 29, 1983. [8305230511]
662. Memorandum for C. Heltemes from H. Denton, "AEOD April 1983 Report on Failures of Class 1E Safety-Related Switch Gear Circuit Breakers to Close on Demand," June 17, 1983. [8306280125]
663. IE Information Notice No. 83-50, "Failures of Class 1E Safety-Related Switchgear Circuit Breakers to Close on Demand," August 1, 1983. [8306270418]
664. Memorandum for D. Eisenhut from R. Spessard, "Unmonitored Failures of Class 1E Safety-Related Switchgear Circuit Breakers and Power Supplies (AITS-F03052383)," June 1, 1984. [8408230490]

665. NUREG/CR-2989, "Reliability of Emergency AC Power System at Nuclear Power Plants," U.S. Nuclear Regulatory Commission, July 1983.
666. Memorandum for T. Speis from H. Denton, "Resolution of Generic Issue B-12: BWR Jet Pump Integrity," September 25, 1984. [8410030458]
667. Memorandum for T. Speis from H. Denton, "Resolution of Generic Issue 69: Make-up Nozzle Cracking in B&W Plants," September 27, 1984. [8410150536]
668. Memorandum for H. Denton from R. Minogue, "Comments on Generic Issue 79, 'Unanalyzed Reactor Vessel Thermal Stress During Natural Convection Cooldown,'" October 5, 1983. [8310260398]
669. Letter to P. Kadambi (NRC) from F. Miller (B&W Owners Group Analysis Committee), "Transmittal of RV Head Stress Evaluation Program Results," October 15, 1984. [8410190186]
670. Memorandum for H. Denton from R. Mattson, "Generic Issue B-60, Loose Parts Monitoring Systems for Operating Reactors (TACS 52325)," January 10, 1984. [8401180046]
671. Letter to N. Palladino from P. Shewmon, "Control Room Habitability," August 18, 1982. [8207180073]
672. Memorandum for J. Larkins from J. Murphy, "Proposed Resolution of GSI-15, 'Radiation Effects on Reactor Pressure Vessel Supports,'" June 22, 1994. [9407140032]
673. Letter to W. Dircks from J. Ebersole, "ACRS Subcommittee Report on Control Room Habitability," May 17, 1983. [8305260104]
674. Memorandum for W. Dircks from H. Denton, "Control Room Habitability," July 27, 1983. [8308180433]
675. Memorandum for H. Denton from W. Dircks, "Control Room Habitability," August 15, 1983. [8309160034]
676. Memorandum for T. Murley, et al., from H. Denton, "Control Room Habitability," September 19, 1983. [8310120463]
677. Letter to W. Milstead (NRC) from T. Powers (PNL), "A Probabilistic Examination of Nuclear Power Plant Control Room Habitability During Various Accident Scenarios," December 3, 1984. [8412050472]
678. Memorandum for W. Dircks from H. Denton, "Control Room Habitability," June 29, 1984. [8407100196]
679. Memorandum for T. Speis from R. Bernero, "Revised Schedule for Generic Issue 83, Control Room Habitability," September 28, 1984. [8410110484]
680. NUREG/CR-2258, "Fire Risk Analysis for Nuclear Power Plants," U.S. Nuclear Regulatory Commission, September 1981.

681. NUREG-0844, "NRC Integrated Program for the Resolution of Unresolved Safety Issues A-3, A-4, A-5 Regarding Steam Generator Tube Integrity," U.S. Nuclear Regulatory Commission, September 1988.
682. Note to W. Kane from G. Holahan, "Background Information Relating to the Assessment of the Offsite Consequences of Non-Core Melt, Steam Generator Tube Rupture Events," October 24, 1983. [9705190255]
683. Memorandum for W. Johnston from R. Ballard, "Disputed Procedures for Estimating Probable Maximum Precipitation," January 13, 1984. [8401260466]
684. Hydrometeorological Report No. 52, "Application of Probable Maximum Precipitation Estimates - United States East of the 105th Meridian," U.S. Department of Commerce, National Oceanic and Atmospheric Administration, August 1982.
685. Hydrometeorological Report No. 51, "Probable Maximum Precipitation Estimates, United States East of the 105th Meridian," U.S. Department of Commerce, National Oceanic and Atmospheric Administration, June 1978.
686. Hydrometeorological Report No. 33, "Seasonal Variation of the Probable Maximum Precipitation East of the 105th Meridian for Areas from 10 to 1,000 Square Miles and Durations of 6, 12, 24 and 48 Hours," U.S. Department of Commerce, April 1956.
687. Regulatory Guide 1.59, "Design Basis Floods for Nuclear Power Plants," U.S. Nuclear Regulatory Commission, (Rev. 2) August 1977. [7907100225]
688. Regulatory Guide 1.102, "Flood Protection for Nuclear Power Plants," U.S. Nuclear Regulatory Commission, (Rev. 1) September 1976. [7907100372]
689. Memorandum for V. Stello from H. Denton, "Potential Generic Requirement Concerning Design for Probable Maximum Precipitation," June 25, 1984. [8407100105]
690. Memorandum for V. Stello from H. Denton, "Generic Requirements Regarding Design for Probable Maximum Precipitation," October 10, 1984. [8503140522, 8410190029]
691. Memorandum for H. Denton from V. Stello, "Generic Requirements Regarding Design for Probable Maximum Precipitation," August 8, 1984. [8408160442]
692. Memorandum for T. Speis from H. Denton, "Generic Issue A-41; 'Long Term Seismic Program,'" October 10, 1984. [9705200066]
693. Memorandum for H. Denton from R. Bernero, "Resolution of Generic Issue No. 22, Inadvertent Boron Dilution Events (BDES)," September 17, 1984. [8410020424]
694. Memorandum for T. Speis from H. Denton, "Closeout of Generic Issue No. 22, 'Inadvertent Boron Dilution Events (BDE),' " October 15, 1984. [8410310592]
695. Memorandum for T. Speis from H. Denton, "Closeout of Generic Issue 50, 'Reactor Vessel Level Instrumentation in BWRs,'" October 17, 1984. [8411030745]

696. NRC Letter to All Boiling Water Reactor (BWR) Licensees of Operating Reactors (Except LaCrosse, Big Rock Point, Humboldt Bay and Dresden-1), "Reactor Vessel Water Level Instrumentation in BWRs (Generic Letter No. 84-23," October 26, 1984. [8410290050]
697. Memorandum for D. Eisenhut from R. Bernero, "Resolution of Generic Issue 50, Reactor Vessel Level Instrumentation in BWRs," September 6, 1984. [8410010093]
698. NUREG-0927, "Evaluation of Water Hammer Occurrence in Nuclear Power Plants," U.S. Nuclear Regulatory Commission, (Rev. 1) March 1984.
699. NUREG-0609, "Asymmetric Blowdown Loads on PWR Primary Systems," U.S. Nuclear Regulatory Commission, January 1981.
700. NRC Letter to All Operating PWR Licenses, Construction Permit Holders, and Applicants for Construction Permits, "Safety Evaluation of Westinghouse Topical Reports Dealing with Elimination of Postulated Pipe Breaks in PWR Primary Main Loops (Generic Letter 84-04)," February 1, 1984. [8402010410]
701. NUREG-0408, "Mark I Containment Short-Term Program Safety Evaluation Report," U.S. Nuclear Regulatory Commission, December 1977.
702. NUREG-0661, "Mark I Containment Long Term Program Safety Evaluation Report, Resolution of Generic Technical Activity A-7," U.S. Nuclear Regulatory Commission, July 1980, (Supplement 1) August 1982.
703. NUREG-0808, "Mark II Containment Program Evaluation and Acceptance Criteria," U.S. Nuclear Regulatory Commission, August 1981.
704. NUREG-0460, "Anticipated Transients Without Scram for Light Water Reactors," U.S. Nuclear Regulatory Commission, (Vol. 1) April 1978, (Vol. 2) April 1978, (Vol. 3) December 1978, (Vol. 4) March 1980.
705. Memorandum for C. Thomas from O. Parr, "CRD Accumulators - Proposed Improved Technical Specification," August 13, 1984. [8408270516]
706. NUREG-0123, "Standard Technical Specifications for General Electric Boiling Water Reactors (BWR/5)," U.S. Nuclear Regulatory Commission, (Rev. 3) December 1980.
707. Memorandum for H. Denton, et. al., from C. Michelson, "Survey of Valve Operator-Related Events Occurring During 1978, 1979, and 1980," December 23, 1981. [8202040039]
708. Memorandum for C. Michelson from H. Denton, "NRR Comments on AEOD Draft Report: Survey of Valve Operator-Related Events Occurring During 1978, 1979 and 1980," March 5, 1982. [8203240048]
709. AEOD/C203, "Survey of Valve Operator-Related Events Occurring During 1978, 1979, and 1980," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, May 1982. [8206180032]

710. Memorandum for C. Michelson from E. Brown and F. Ashe, "AEOD Assessment of Program Office Responses to the Report AEOD/C203, 'Survey of Valve Operator-Related Events Occurring During 1978, 1979, and 1980,'" December 23, 1982. [8301250189, 8301120496]
711. Memorandum for H. Denton from C. Michelson, "AEOD Assessment of Program Office Responses to AEOD Case Study (C-203), 'Survey of Valve Operator Related Events Occurring During 1978, 1979, and 1980,'" January 12, 1983. [8301250183]
712. Memorandum for C. Michelson from H. Denton, "AEOD Assessment of Program Office Responses to AEOD Case Study (C203), 'Survey of Valve Operator Related Events Occurring During 1978, 1979, and 1980,'" February 23, 1983. [8303100567]
713. Memorandum for K. Seyfrit from E. Brown and F. Ashe, "Engineering Evaluation Report AEOD/E305 Inoperable Motor Operated Valve Assemblies Due to Premature Degradation of Motors and/or Improper Limit Switch/Torque Switch Adjustment," April 13, 1983. [8305050353]
714. Memorandum for W. Minners from R. Bosnak, "Status of Potential Generic Issue 54, 'Valve Operator Related Events Occurring During 1978, 1979, and 1980,'" March 26, 1984. [8404110417]
715. Memorandum for R. Vollmer from R. Bosnak, "MEB Task Action Plan for Resolution of Generic Issue II.E.6.1, 'In Situ Testing of Valves,'" July 30, 1984. [8408070139]
716. Memorandum for D. Eisenhut from D. Muller, "PWR Reactor Cavity Uncontrolled Exposures, Generic Letter Implementing a Generic Technical Specification," July 12, 1984. [8407230356]
717. Memorandum for A. Thadani from W. Minners, "CRAC2 Computer Runs in Support of USI A-43," February 1, 1983. [8302090275]
718. Memorandum for W. Minners from F. Congel, "Prioritization of Generic Issue 97: PWR Reactor Cavity Uncontrolled Exposures," February 8, 1985. [8502250136]
719. Memorandum for H. Denton from R. Bernero, "PWR Reactor Cavity Uncontrolled Exposures," November 28, 1984. [8412180620]
720. Memorandum for T. Speis from R. Bernero, "Request for Prioritization of Generic Safety Issue - Break Plus Single Failure in BWR Water Level Instrumentation," October 10, 1984. [8410290282]
721. Memorandum for H. Denton and V. Stello from C. Michelson, "Case Study Report - Safety Concern Associated with Reactor Vessel Instrumentation in Boiling Water Reactors," September 2, 1981. [8109220940]
722. Memorandum for B. Sheron from A. Thadani, "Reactor Vessel Level Instrumentation in BWR's (Generic Issue 50)," August 2, 1984. [8408090089]

723. Memorandum for H. Denton from T. Speis, "Reactor Vessel Level Instrumentation in BWRs (Generic Issue 50)," August 2, 1984. [8408090386, 8408090094]
724. Memorandum for W. Dircks, et al., from S. Chilk, "Staff Requirements - Affirmation/Discussion and Vote, 11:30 a.m., Friday, June 1, 1984, Commissioners' Conference Room, D.C. Office (Open to Public Attendance)," June 1, 1984.
725. Federal Register Notice 49 FR 26036, "10 CFR Part 50, Reduction of Risk from Anticipated Transients Without Scram (ATWS) Events for Light-Water-Cooled Nuclear Power Plants," June 26, 1984.
726. NEDO-21506, "Stability and Dynamic Performance of the General Electric Boiling Water Reactor," General Electric Company, January 1977.
727. Memorandum for D. Crutchfield from L. Rubenstein, "Staff Evaluation of GE Topical Report NEDE-24011 (GESTAR) Amendment 8," April 17, 1985. [8504290470]
728. XN-NF-691(P)(A) & Supplement 1, "Stability Evaluation of Boiling Water Reactor Cores Sensitivity Analyses & Benchmark Analysis," Exxon Nuclear Company, Inc., August 22, 1984.
729. Memorandum for D. Eisenhut from R. Mattson, "Board Notification - BWR Core Thermal Hydraulic Stability," February 27, 1984. [8403020299]
730. Memorandum for T. Novak from L. Rubenstein, "Susquehanna 1 and 2 - Thermal Hydraulic Stability Technical Specification Change (TACS 55021 and 55022)," July 11, 1984. [8407170149]
731. Memorandum for G. Lainas from L. Rubenstein, "SER Input for Peach Bottom-3 Technical Specification Changes for Cycle 6 Operation with Increased Core Flows and Decreased Feedwater Temperatures (TACS #55123)," October 23, 1984. [8411010312]
732. NEDO-21078, "Test Results Employed by GE for BWR Containment and Vertical Vent Loads," General Electric Company, October 1975.
733. NUREG-0487, "Mark II Containment Lead Plant Program Load Evaluation and Acceptance Criteria," U.S. Nuclear Regulatory Commission, November 1978, (Supplement 1) September 1980.
734. NUREG-0783, "Suppression Pool Temperature Limits for BWR Containments," U.S. Nuclear Regulatory Commission, November 1981.
735. Letter to T. Novak (NRC) from T. Pickens (BWR Owners' Group), "Agreements from BWROG/NRC Meeting on Suppression Pool Temperature Limit," October 16, 1984. [8410220072]
736. Memorandum for T. Speis from R. Bernero, "Proposed Generic Issue 'BWR Suppression Pool Temperature Limits,'" November 21, 1984. [8412030526]

737. Memorandum for W. Minners from W. Butler, "Comments on Prioritization of Generic Issue 108, 'BWR Suppression Pool Temperature Limits,'" January 10, 1985. [8501160095]
738. NUREG-1044, "Evaluation of the Need for a Rapid Depressurization Capability for CE Plant," U.S. Nuclear Regulatory Commission, December 1984.
739. SECY-84-134, "Power Operated Relief Valves for Combustion Engineering Plants," March 23, 1984. [8404180339]
740. "Draft Maintenance Program Plan," U.S. Nuclear Regulatory Commission, May 8, 1984.
741. NUREG/CR-3543, "Survey of Operating Experience from LERs to Identify Aging Trend," U.S. Nuclear Regulatory Commission, January 1984.
742. NUREG-0619, "BWR Feedwater Nozzle and Control Rod Drive Return Line Nozzle Cracking," U.S. Nuclear Regulatory Commission, November 1980.
743. NUREG-0744, "Resolution of the Task A-11 Reactor Vessel Materials Toughness Safety Issue," U.S. Nuclear Regulatory Commission, (Rev. 1) October 1982.
744. NRC Letter to All Power Reactor Licensees (Except Ft. St. Vrain), "NUREG-0744 Rev. 1; Generic Letter No. 82-26) - Pressure Vessel Material Fracture Toughness," November 12, 1982. [8211160047]
745. EPRI NP-3967, "Classification and Analysis of Reactor Operating Experience Involving Dependent Events," Electric Power Research Institute, June 1985.
746. NUREG-0224, "Final Report on Reactor Vessel Pressure Transient Protection for Pressurized Water Reactors," U.S. Nuclear Regulatory Commission, September 1978.
747. NUREG-0612, "Control of Heavy Loads at Nuclear Power Plants Resolution of Generic Technical Activity A-36," U.S. Nuclear Regulatory Commission, July 1980.
748. NUREG-0763, "Guidelines for Confirmatory Inplant Tests of Safety Relief Valve Discharges for BWR Plants," U.S. Nuclear Regulatory Commission, May 1981.
749. NUREG-0802, "Safety/Relief Valve Quencher Loads: Evaluation for BWR Mark II and III Containments," U.S. Nuclear Regulatory Commission, October 1982.
750. NUREG-0313, "Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping," U.S. Nuclear Regulatory Commission, July 1977, (Rev. 1) July 1980, (Rev. 2) January 1988.
751. WASH-1270, "Anticipated Transients Without Scram for Water-Cooled Reactors," U.S. Atomic Energy Commission, September 1973.
752. Memorandum for S. Hanauer from D. Eisenhut, "Value/Impact Assessment of Proposed Steam Generator Generic Requirements," October 12, 1982. [8211110465]

753. SECY-84-13, "NRC Integrated Program for the Resolution of Steam Generator USI's," January 11, 1984. [8401310036]
754. NUREG-0916, "Safety Evaluation Report Related to Restart of R.E. Ginna Nuclear Power Plant," U.S. Nuclear Regulatory Commission, May 1982.
755. NUREG-0651, "Evaluation of Steam Generator Tube Rupture Events," U.S. Nuclear Regulatory Commission, March 1980.
756. Memorandum for D. Eisenhut from T. Speis, "Prioritization of Staff Actions Concerning S.G. Tube Degradation and Rupture Events," February 23, 1983. [8303090047]
757. SECY-84-13A, "NRC Integrated Program for the Resolution of Steam Generator USIs," September 7, 1984. [8409140060]
758. SECY-84-13B, "NRC Integrated Program for the Resolution of Steam Generator USI's - Response to Commissioner Comments (Memo from Chilk to Dircks dated September 13, 1984)," November 5, 1984. [8411210357]
759. AEOD/C005, "AEOD Observations and Recommendations Concerning the Problem of Steam Generator Overfill and Combined Primary and Secondary Blowdown," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, December 17, 1980. [8101150366]
760. NUREG/CR-2883, "Study of the Value and Impact of Alternative Decay Heat Removal Concepts for Light Water Reactors," U.S. Nuclear Regulatory Commission, (Vol. 1) June 1983, (Vol. 2) June 1983, (Vol. 3) June 1983.
761. AEOD/E414, "Stuck Open Isolation Check Valve on the Residual Heat Removal System at Hatch Unit 2," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, May 31, 1984. [8406190101]
762. Memorandum for W. Minners from G. Holahan, "Prioritization of Interfacing System LOCA at Boiling Water Reactors," October 25, 1984. [8411050292]
763. NUREG-0677, "The Probability of Intersystem LOCA: Impact Due to Leak Testing and Operational Changes," U.S. Nuclear Regulatory Commission, May 1980.
764. SECY-85-129, "Maintenance and Surveillance Program Plan," April 12, 1985. [8509190696]
765. SECY-85-62, "NRC Integrated Program for the Resolution of Steam Generator USI's - Response to Commissioner Comments (Memo from Chilk to Dircks Dated January 23, 1985)," February 22, 1985. [8504080388]
766. Memorandum for W. Dircks from S. Chilk, "SECY-85-62 - NRC Integrated Program for the Resolution of Steam Generator USIs - Response to Commissioners Comments (Memo from Chilk and Dircks Dated January 23, 1985)," March 15, 1985.

767. Memorandum for W. Dircks from H. Denton, "Final Rule - Applicability of License Conditions and Technical Specifications in an Emergency," February 17, 1983. [8303300333]
768. Memorandum for T. Speis from H. Denton, "Formation of a Technical Specification Improvement Project Group," December 31, 1984. [8501150417]
769. Memorandum for V. Stello from H. Denton, "Close Out Generic Issue #B-19 -Thermal-Hydraulic Stability," May 21, 1985. [8506040556]
770. Letter from P. Crane (Pacific Gas and Electric Company) to Director, Division of Licensing, U.S. Nuclear Regulatory Commission, "Report on June 7, 1975 Ferndale Earthquake," August 4, 1975. [8602070315, 993280104, ML993280111]
771. Memorandum for W. Minners from L. Reiter, "Generic Issue No. B-50 Post Operating Basis Earthquake Inspection," June 7, 1985.
772. Letter to A. Schwencer (NRC) from C. Dunn (Duquesne Light Company), "Beaver Valley Power Station, Unit No. 1, Docket No. 50-334, Request for Amendment to the Operating License - No. 35," October 27, 1978. [7811030107]
773. Letter to J. Carey (Duquesne Light Company) from S. Varga (NRC), "Beaver Valley Unit No. 1 - Operation With Two Out of Three Reactor Coolant Loops - Safety Evaluation," July 20, 1984. [8408010218]
774. Memorandum for D. Eisenhut from D. Wigginton, "Closeout of MPA E-05; Westinghouse N-1 Loop Operation," January 11, 1985. [8501300565]
775. Memorandum for R. Emrit from A. Murphy, "Generic Issue Management Control System, Issue No. 119.3, Decouple OBE from SSE," February 21, 1992. [9803260147]
776. Memorandum for R. Bernero from D. Eisenhut, "BWR Thermal-Hydraulic Stability Technical Specifications," November 16, 1984. [8411290326]
777. Memorandum for W. Dircks from H. Denton, "Closeout of TMI Action Plan Items I.A.2.2 and I.A.2.7 Training and Qualifications of Operating Personnel," June 24, 1985. [8507020587]
778. Memorandum for W. Dircks from H. Denton, "TMI Action Item I.A.3.4," February 12, 1985. [8502260084]
779. Memorandum for W. Dircks from J. Taylor, "TMI Action Plan - Completed Item," June 26, 1985. [8507080034]
780. IE Information Notice No. 83-58, "Transamerica DeLaval Diesel Generator Crankshaft Failure," U.S. Nuclear Regulatory Commission, August 30, 1983. [8308040044]
781. IE Information Notice No. 83-51, "Diesel Generator Events," U.S. Nuclear Regulatory Commission, August 5, 1983. [8306270425]

782. Memorandum for C. Berlinger from H. Denton, "Detail Assignment to DOL, Transamerica DeLaval Emergency Diesel Generator Project Group (TDI Project Group)," January 25, 1984. [8505130221]
783. SECY-84-34, "Emergency Diesel Generators Manufactured by Transamerica DeLaval, Inc.," January 25, 1984. [8403010451]
784. Letter to D. Bixby (TDI) from D. Eisenhut (NRC), February 14, 1984. [8402290333]
785. TDI Diesel Generators Owners' Group Program Plan, March 2, 1984.
786. SECY-84-155, "Section 208 Report to the Congress on Abnormal Occurrences for October-December, 1983," April 11, 1984. [8405140043]
787. Letter to J. George (Transamerica Delaval, Inc., Owners' Group) from D. Eisenhut (NRC), "Safety Evaluation Report, Transamerica Delaval, Inc. Diesel Generator Owners' Group Program Plan," August 13, 1984. [8408240115]
788. Memorandum for W. Minners from B. Sheron, "Additional Low-Temperature-Overpressure Protection Issues for Light-Water Reactors," August 1, 1984. [8408130012]
789. IE Information Notice No. 83-26, "Failure of Safety/Relief Valve Discharge Line Vacuum Breakers," U.S. Nuclear Regulatory Commission, May 3, 1983. [8303040028]
790. NUREG/CR-3384, "VISA - A Computer Code for Predicting the Probability of Reactor Pressure Vessel Failure," U.S. Nuclear Regulatory Commission, September 1983.
791. Memorandum for K. Seyfrit from C. Hsu, "EE No. AEOD/E322 Damage to Vacuum Breaker Valves as a Result of Relief Valve Lifting," September 21, 1983. [8310060353]
792. AEOD/C401, "Low Temperature Overpressure Events at Turkey Point Unit 4," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, March 1984. [8404050445]
793. Memorandum for B. Sheron from B. Liaw, "Additional Low-Temperature-Overpressure Protection (LTOP) Issues for Light-Water Reactors," August 30, 1984. [8409130397]
794. Memorandum for K. Seyfrit from E. Imbro, "Single Failure Vulnerability of Power Operated Relief Valve Actuation Circuitry for Low Temperature Overpressure Protection (LTOP)," October 24, 1984. [8411070245]
795. AEOD/C403, "Edwin I. Hatch Unit No. 2 Plant Systems Interaction Event on August 25, 1982," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, May 1984. [8405300746]
796. Memorandum for R. Mattson from T. Dunning, "RHR Interlocks for Westinghouse Plants," April 17, 1984. [8404300085]
797. Memorandum for F. Rowsome from W. Houston, "RCS/RHR Suction Line Valve Interlock on PWRs," August 27, 1984. [8409070331]

798. NSAC-52, "Residual Heat Removal Experience Review and Safety Analysis, Pressurized Water Reactor," Nuclear Safety Analysis Center, January 1983.
799. Memorandum for W. Dircks from H. Denton, "Resolution of Generic Issue III.D.2.3 -- Liquid Pathway Studies," August 28, 1985. [8509050212]
800. NUREG/CR-4258, "An Approach to Team Skills Training of Nuclear Power Plant Control Room Crews," U.S. Nuclear Regulatory Commission, July 1985.
801. Memorandum for W. Dircks from H. Denton, "Team Training for Nuclear Power Plant Control Room Crews," July 10, 1985. [8507220495]
802. NUREG/CR-3739, "The Operator Feedback Workshop: A Technique for Obtaining Feedback from Operations Personnel," U.S. Nuclear Regulatory Commission, September 1984.
803. NUREG/CR-4139, "The Mailed Survey: A Technique for Obtaining Feedback from Operations Personnel," U.S. Nuclear Regulatory Commission, May 1985.
804. Memorandum for W. Dircks from H. Denton, "TMI Action Plan Item I.A.2.6(4)," September 25, 1985. [8510030079]
805. Memorandum for T. Cornbs from H. Denton, "Revised SRP Section 13.5.2 and Appendix A to SRP Section 13.5.2 of NUREG-0800," July 17, 1985. [8508050283]
806. Memorandum for W. Dircks from H. Denton, "Closeout of TMI Action Plan, Task II.B.6, 'Risk Reduction for Operating Reactors at Sites With High Population Densities,'" September 25, 1985. [8510030342]
807. Memorandum for W. Dircks from R. Minogue, "Closeout of TMI Action Plan Task II.B.8 'Rulemaking Proceeding on Degraded Core Accidents - Hydrogen Control,'" July 19, 1985. [8508010066]
808. Memorandum for W. Dircks from H. Denton, "Close Out of TMI Action Plan, Task II.B.8," August 12, 1985. [8508210316]
809. NUREG-1070, "NRC Policy on Future Reactor Designs," U.S. Nuclear Regulatory Commission, July 1985.
810. NUREG/CR-3085, "Interim Reliability Evaluation Program: Analysis of the Millstone Point Unit 1 Nuclear Power Plant," U.S. Nuclear Regulatory Commission, (Vol. 1) April 1983, (Vol. 2) August 1983, (Vol. 3) July 1983, (Vol. 4) July 1983.
811. NUREG/CR-3511, "Interim Reliability Evaluation Program: Analysis of the Calvert Cliffs Unit 1 Nuclear Power Plant," U.S. Nuclear Regulatory Commission, (Vol. 1) May 1984, (Vol. 2) October 1984.
812. NUREG/CR-2728, "Interim Reliability Evaluation Program Procedures Guide," U.S. Nuclear Regulatory Commission, March 1983.

813. Memorandum for W. Dircks from R. Minogue, "Closeout of TMI Action Plan, Task II.C.1, 'Interim Reliability Evaluation Program,'" July 9, 1985. [8507180593]
814. SECY-84-133, "Integrated Safety Assessment Program (ISAP)," March 23, 1984. [8404100072]
815. SECY-85-160, "Integrated Safety Assessment Program - Implementation Plan," May 6, 1985. [8505230571]
816. Memorandum for W. Dircks from H. Denton, "Close-out of Generic Issues II.C.2, 'Continuation of IREP,' and IV.E.5, 'Assess Currently Operating Reactors,'" September 25, 1985. [9909290069]
817. Memorandum for W. Dircks from R. Minogue, "Closeout of TMI Action Plan Task II.E.2.2, 'Research on Small Break LOCA's and Anomalous Transients,'" July 25, 1985. [9909290072]
818. Memorandum for W. Dircks from J. Taylor, "TMI Action Plan - Completed Item," August 15, 1985. [8508200726]
819. EPRI EL-3209, "Workshop Proceedings: Retaining Rings for Electric Generators," Electric Power Research Institute, August 1983.
820. Memorandum for R. Fraley from R. Vollmer, "Proposed NRR Revisions to Review Procedures for Turbine Missile Issue," May 12, 1983. [8305250286]
821. Memorandum for W. Johnson from T. Novak, "Midland SSER #3 - Turbine Missile Review," November 1, 1983. [8311140470]
822. Memorandum for V. Stello from H. Denton, "NRR Plans for Approval of WCAP-10271," January 11, 1985. [8501220433, 8501220440]
823. Letter to J. Sheppard (Westinghouse Owners Group) from C. Thomas (NRC), "Acceptance for Referencing of Licensing Topical Report WCAP-10271, 'Evaluation of Surveillance Frequencies and Out of Service Times for the Reactor Protection Instrumentation Systems,'" February 21, 1985. [8503010427]
824. Memorandum for T. Speis from R. Mattson, "Request for Prioritization of Generic Safety Issue - Failure of HPCI Steam Line Without Isolation," October 18, 1983. [8311020209]
825. Memorandum for K. Seyfrit from P. Lam, "Failure of an Isolation Valve of the Reactor Core Isolation Cooling System to Open Against Operating Reactor Pressure," August 23, 1984. [8411010554, 8411010453]
826. Letter to A. Schwencer (NRC) from J. Kemper (Philadelphia Electric Company), "Limerick Generating Station, Units 1 & 2, Request for Additional Information from NRC Equipment Qualification Branch (EQB)," February 27, 1984. [9909290076]

827. NEDO-24708A, "Additional Information Required for NRC Staff Generic Report on Boiling Water Reactors," General Electric Company, August 1979 [7909130302, 7909130304], December 1980.
828. NUREG/CR-3933, "Risk Related Reliability Requirements for BWR Safety-Important Systems with Emphasis on the Residual Heat Removal System," U.S. Nuclear Regulatory Commission, August 1984.
829. "An Evaluation of Unisolated LOCA Outside the Drywell in the Shoreham Nuclear Power Station," Brookhaven National Laboratory, June 1985. [9909290080]
830. Memorandum for W. Minners from A. Thadani, "Comments on Generic Issue No. 87 - Failure of HPCI Steam Line Without Isolation," June 28, 1985. [8507170422]
831. NUREG/CR-1433, "Examination of the Use of Potassium Iodide (KI) as an Emergency Protective Measure for Nuclear Reactor Accidents," U.S. Nuclear Regulatory Commission, October 1980.
832. SECY-83-362, "Emergency Planning - Predistribution/Stockpiling of Potassium Iodide for the General Public," August 30, 1983. [8309080120]
833. SECY-85-167, "Federal Policy Statement on the Distribution and Use of Potassium Iodide," May 13, 1985. [8505310621]
834. Memorandum for H. Denton and R. Minogue from W. Dircks, "Review of NRC Requirements for Nuclear Power Plant Piping," August 1, 1983. [8308300212]
835. Memorandum for W. Dircks from R. Minogue, "Plan to Implement Piping Review Committee Recommendations," July 30, 1985. [9705050005]
836. Memorandum for T. Murley, et al., from J. Taylor, "Results of Regional Survey of Plant Specific Information Relating to the Potential for Uncontrolled Radiation Exposures in PWR Reactor Cavities," June 18, 1985. [8506250113]
837. Note to R. Vollmer from T. Speis, "Proposed Request to Perform Research on the Stress Corrosion Cracking of Pressure Boundary Ferritic Steels in Selected Environments," January 7, 1985. [9909290082]
838. NUREG-1165, "Environmental Standard Review Plan for ES Section 7.1.1," U.S. Nuclear Regulatory Commission, November 1985.
839. Letter to J. Bayne (PASNY) from S. Varga (NRC), "Steam Generator Tube and Girth Weld Repairs at the Indian Point Nuclear Generating Plant, Unit No. 3 (IP-3)," May 27, 1983. [8306150627]
840. "Value-Impact Analysis of Recommendations Concerning Steam Generator Tube Degradations and Rupture Events," Science Applications, Inc., February 2, 1983.

841. Regulatory Guide 1.99, "Effects of Residual Elements on Predicted Radiation Damage to Reactor Vessel Materials," U.S. Nuclear Regulatory Commission, July 1975, (Rev. 1) April 1977 [7907100362], (Rev. 2) May 1988 [8907270187].
842. IE Information Notice No. 82-37, "Cracking in the Upper Shell to Transition Cone Girth Weld of a Steam Generator at an Operating Pressurized Water Reactor," U.S. Nuclear Regulatory Commission, September 16, 1982. [8208190220]
843. Letter to D. Smith (NRC) from E. Rahe (Westinghouse), January 17, 1982. [9909290085]
844. NUREG/CR-3281, "Investigation of Shell Cracking on the Steam Generators at Indian Point Unit No. 3," U.S. Nuclear Regulatory Commission, June 1983.
845. NUREG/CR-3614, "Constant Extension Rate Testing of SA302 Grade B Material in Neutral and Chloride Solutions," U.S. Nuclear Regulatory Commission, February 1984.
846. Letter to W. Hazelton (NRC) from H. Watanabe (GE), "Laboratory Examination of Garigliano Secondary Steam Generator-B Core Samples," NEDE-25162, July 1979," December 13, 1979. [7912130566]
847. EPRI NP-1136, "Limiting Factor Analysis of High Availability Nuclear Plants (Boiling Water Reactors)," Electric Power Research Institute, (Vol. 1) August 1979.
848. Regulatory Guide 1.56, "Maintenance of Water Purity in Boiling Water Reactors," U.S. Nuclear Regulatory Commission, (Rev. 1) July 1978.
849. NUREG/CR-3842, "Steam Generator Group Project Task 8 - Selective Tube Unplugging," U.S. Nuclear Regulator Commission, July 1984.
850. NRC Letter to All PWR Licensees of Operating Reactors, Applicants for Operating Licenses, and Holders of Construction Permits, and Ft. St. Vrain, "Staff Recommended Actions Stemming from NRC Integrated Program for the Resolution of Unresolved Safety Issues Regarding Steam Generator Tube Integrity (Generic Letter 85-02)," April 17, 1985. [8504120031]
851. NUREG/CP-0058, "Twelfth Water Reactor Safety Research Information Meeting," U.S. Nuclear Regulatory Commission, (Vol. 4) January 1985.
852. NUREG/CP-0044, "Proceedings of the International Atomic Energy Agency Specialists' Meeting on Subcritical Crack Growth," U.S. Nuclear Regulatory Commission, (Vol. 1) May 1983, (Vol. 2) May 1983.
853. "Corrosion Fatigue Crack Growth in Reactor Pressure Vessel Steels - Structural Integrity of Light Water Reactor Components," Scott, P. et al., Elsevier Science Publishing Co., Inc., 1982.
854. NUREG/CR-4121, "The Effects of Sulfur Chemistry and Flow Rate on Fatigue Crack Growth Rates in LWR Environments," U.S. Nuclear Regulatory Commission, February 1985.

855. NUREG-0975, "Compilation of Contract Research for the Materials Engineering Branch, Division of Engineering Technology," U.S. Nuclear Regulatory Commission, (Vol. 2) March 1984.
856. PNO-II-85-41, "Small Steam Generator Surface Cracks," U.S. Nuclear Regulatory Commission, April 23, 1985. [8504290412]
857. Memorandum for W. Minners from B. Liaw, "Prioritization of Generic Issue No. (111) Stress Corrosion Cracking of RCPB Ferritic Steels and Steam Generator Vessels," June 7, 1985. [8506170320]
858. IE Information Notice No. 85-65, "Crack Growth in Steam Generator Girth Welds," U.S. Nuclear Regulatory Commission, July 31, 1985. [8507290456]
859. Memorandum for H. Thompson from J. Knight, "Steam Generator Shell Transition Joint Cracking," July 10, 1985. [8507190409]
860. NUREG-0937, "Evaluation of PWR Response to Main Steamline Break With Concurrent Steam Generator Tube Rupture and Small-Break LOCA," U.S. Nuclear Regulatory Commission, December 1982. [8412190335]
861. SECY-83-357B, "Status of Hydrogen Control Issue and Rulemaking Recommendations in SECY-83-357A," December 3, 1984. [8412190335]
862. IE Bulletin No. 79-13, "Cracking in Feedwater System Piping," June 25, 1979 [7906250348], (Rev. 1) August 29, 1979 [7908220101], (Rev. 2) October 17, 1979 [7908220135].
863. Memorandum for T. Speis from H. Denton, "Closeout of Generic Issues B-58 and C-11," July 9, 1985. [8507180530]
864. AEOD/C301, "Failures of Class 1E Safety-Related Switchgear Circuit Breakers to Close on Demand," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, April 1983. [8305230531]
865. Memorandum for T. Speis from H. Denton, "Resolution of Generic Issue 14, 'PWR Pipe Cracks,'" October 4, 1985. [9909290092]
866. Federal Register Notice 47 FR 7023, "Proposed Policy Statement on Safety Goals for Nuclear Power Plants," February 17, 1982.
867. Federal Register Notice 48 FR 10772, "Safety Goal Development Program," March 14, 1983.
868. Letter to J. Ahearn from M. Plesset, "Recommendations of President's Commission on ACRS Role," January 15, 1980. [8002150071]
869. Federal Register Notice 46 FR 22358, "10 CFR Part 2, ACRS Participation in NRC Rulemaking," April 17, 1981.

870. Memorandum for Commissioner Ahearn, et al., from L. Bickwit, et al., "TMI Action Plan, Chapter V, Formal Procedures for Ensuring Periodic Public Interaction," October 2, 1980.
871. Memorandum for W. Dircks, et al., from J. Hoyle, "Staff Requirements - Discussion of Action Plan, Chapter V (See SECY-80-230B), 2:00 p.m. Monday, July 7, 1980, Commissioners' Conference Room, D.C. Office (Open to Public Attendance)," July 9, 1980. [8012030928]
872. Federal Register Notice 45 FR 49535, "10 CFR Part 2, Procedural Assistance in Adjudicatory Licensing Proceedings," July 25, 1980.
873. Federal Register Notice 46 FR 13681, "10 CFR Part 2, Domestic Licensing Proceedings; Procedural Assistance Program," February 24, 1981.
874. Memorandum for L. Bickwit from S. Chilk, "SECY-81-391 - Provision of Free Transcripts to All Full Participants in Adjudicatory Proceedings: May 11, 1981 Comptroller General Decision," February 25, 1982.
875. Federal Register Notice 45 FR 34279, "10 CFR Parts 2, 50, Possible Amendments to 'Immediate Effectiveness Rule,'" May 22, 1980.
876. Federal Register Notice 47 FR 47260, "10 CFR Part 2, Commission Review Procedures for Power Reactor Construction Permits; Immediate Effectiveness Rule," October 25, 1982.
877. Federal Register Notice 51 FR 10393, "10 CFR Parts 0 and 2, Revision of Ex Parte and Separation of Functions Rules Applicable to Formal Adjudicatory Proceedings," March 26, 1986.
878. NUREG-0632, "NRC Views and Analysis of the Recommendations of the President's Commission on the Accident at Three Mile Island," U.S. Nuclear Regulatory Commission, November 1979.
879. Federal Register Notice 46 FR 28533, "Statement of Policy on Conduct of Licensing Proceedings," May 27, 1981.
880. Memorandum to All Employees from N. Palladino, "Regulatory Reform Task Force," November 17, 1981.
881. Letter to the Honorable Thomas P. O'Neill, Jr. from N. Palladino, February 21, 1983.
882. Federal Register Notice 48 FR 44173, "10 CFR Part 50, Revision of Backfitting Process for Power Reactors," September 28, 1983.
883. Federal Register Notice 48 FR 44217, "10 CFR Part 50, Revision of Backfitting Process for Power Reactors," September 28, 1983.
884. Federal Register Notice 50 FR 38097, "10 CFR Parts 2 and 50, Revision of Backfitting Process for Power Reactors," September 20, 1985.

885. Memorandum for H. Thompson from D. Crutchfield, "Potential Immediate Generic Actions as a Result of the Davis-Besse Event of June 9, 1985," August 5, 1985. [8508090679]
886. NUREG-1154, "Loss of Main and Auxiliary Feedwater Event at the Davis-Besse Plant on June 9, 1985," U.S. Nuclear Regulatory Commission, July 1985.
887. Memorandum for T. Speis from H. Thompson, "Short Term Generic Actions as a Result of the Davis-Besse Event of June 9, 1985," August 19, 1985. [8508270246]
888. Memorandum for H. Denton from T. Speis, "Adequacy of the Auxiliary Feedwater System at Davis-Besse," July 23, 1985. [8508010086]
889. NSAC-60, "A Probabilistic Risk Assessment of Oconee Unit 3," Electric Power Research Institute, June 1984.
890. NUREG-1032, "Evaluation of Station Blackout Accidents at Nuclear Power Plants," U.S. Nuclear Regulatory Commission, June 1988.
891. Letter to T. Novak (NRC) from R. Crouse (Toledo Edison Company), December 31, 1981. [8201060607]
892. NUREG/CR-2770, "Common Cause Fault Rates for Valves," U.S. Nuclear Regulatory Commission, February 1983.
893. NUREG/CR-2098, "Common Cause Fault Rates for Pumps," U.S. Nuclear Regulatory Commission, February 1983.
894. Memorandum for O. Parr from A. Thadani, "Auxiliary Feedwater System - CRGR Package," November 9, 1984. [8411280233]
895. Memorandum for H. Denton, et al., from W. Dircks, "Staff Actions Resulting from the Investigation of the June 9 Davis-Besse Event (NUREG-1154)," August 5, 1985. [8508090534]
896. SECY-86-56, "Status of Staff Study to Determine if PORVs Should be Safety Grade," February 18, 1986. [8611100428]
897. Memorandum for G. Lainas from F. Rowsome, "Safety Evaluation of the CE Licensees' Responses to TMI Action Item II.K.3.2," August 26, 1983. [8309060394]
898. Memorandum for G. Lainas from F. Rowsome, "Safety Evaluation of the B&W Licensees' Responses to TMI Action Item II.K.3.2," August 24, 1983. [8308310422]
899. Memorandum for G. Lainas from F. Rowsome, "Safety Evaluation of the Westinghouse Licensees' Responses to TMI Action Item II.K.3.2," July 22, 1983. [8308040054]
900. Memorandum for H. Thompson from W. Russell, "Comments on Draft List of Longer Term Generic Actions as a Result of the Davis-Besse Event of June 9, 1985," September 19, 1985. [8509240326]

901. Memorandum for T. Combs from H. Denton, "Revised SRP Section 9.2.1 and SRP Section 9.2.2 of NUREG-0800," June 24, 1986. [8607080481]
902. Memorandum for J. Sniezek and R. Fraley from H. Denton, "Resolution of Generic Issue No. 36, 'Loss of Service Water,'" May 13, 1986. [8605300159]
903. Memorandum for T. Speis from H. Denton, "Resolution of Generic Issue 3, 'Setpoint Drift in Instrumentation,'" May 19, 1986. [8606110638]
904. SECY-83-293, "Amendments to 10 CFR 50 Related to Anticipated Transients Without Scram (ATWS) Events," July 9, 1983. [8308080642]
905. Memorandum for T. Speis from R. Bernero, "Enhancement of the Reliability of Westinghouse Solid State Protection System (SSPS)," April 5, 1985. [8504160610]
906. NUREG/CR-3971, "A Handbook for Cost Estimating," U.S. Nuclear Regulatory Commission, October 1984.
907. Memorandum for W. Minners from B. Sheron, "Generic Issues C-4, C-5, C-6," May 29, 1985. [8506100882]
908. SECY-83-472, "Emergency Core Cooling System Analysis Methods," November 17, 1983. [8401060169]
909. AEOD/C503, "Decay Heat Removal Problems at U.S. Pressurized Water Reactors," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, December 1985. [8601060316]
910. Memorandum for H. Denton from C. Heltemes, "Case Study Report - Decay Heat Removal Problems at U.S. Pressurized Water Reactors," December 23, 1985. [8601060315]
911. Memorandum for C. Heltemes from H. Denton, "AEOD's Report on Decay Heat Removal Problems at U.S. PWRs," February 10, 1986. [8602200004]
912. Memorandum to T. Murley, et al., from H. Denton, "Evaluation of Industry Success in Achieving ALARA-Integrated Radiation Protection Plans - Data Trend Assessments," May 19, 1986.
913. Memorandum for V. Stello from H. Denton, "Resolution of Generic Issue III.D.3.1, 'Radiation Protection Plans,'" May 19, 1986.
914. Memorandum for H. Thompson and T. Speis from R. Bernero, "Request for Comments on Draft CRGR Package with Requirements for Upgrading Auxiliary Feedwater Systems in Certain Operating Plants," October 3, 1985. [8510090228]
915. Memorandum for W. Minners from A. Thadani, "Seismic Induced Relay Chatter Issue," March 22, 1985.

916. Regulatory Guide 1.29, "Seismic Design Classification," U.S. Nuclear Regulatory Commission, June 1972, (Rev. 1) August 1973 [8003280778], (Rev. 2) February 1976, (Rev. 3) September 1978 [7810030052].
917. Regulatory Guide 1.100, "Seismic Qualification of Electrical Equipment for Nuclear Power Plants," U.S. Nuclear Regulatory Commission, March 1976, (Rev. 1) August 1977.
918. NUREG/CP-0070, "Proceeding of the Workshop on Seismic and Dynamic Fragility of Nuclear Power Plant Components," U.S. Nuclear Regulatory Commission, August 1985.
919. NUREG-1030, "Seismic Qualification of Equipment in Operating Nuclear Power Plants," U.S. Nuclear Regulatory Commission, February 1987.
920. ANSI/ANS 5.1, "Decay Heat Power in Light Water Reactors," American National Standards Institute, 1979.
921. Letter to the Honorable Morris K. Udall from Joseph M. Hendrie, August 7, 1978. [7901030172, 8001230259]
922. Letter to Joseph Hendrie from Morris K. Udall, January 27, 1978. [8007210279, 8007180431]
923. Memorandum for J. Taylor from D. Morrison, "Resolution of Generic Safety Issue 15, 'Radiation Effects on Reactor Vessel Supports,'" May 29, 1996. [9606190081]
924. SECY-96-107, "Uniform Tracking of Agency Generic Technical Issues," May 14, 1996. [9605230140]
925. Memorandum for E. Beckjord from T. Murley, "Regulatory Guide 1.44," April 30, 1992. [9205110015]
926. Memorandum for Record from E. McGregor, "SECY-80-366 - NRC Legislative Program for 97th Congress," April 8, 1981.
927. Memorandum for Chairman Palladino, et al., from A. Kenneke, "TMI Action Plan, Chapter V," May 18, 1984.
928. Memorandum for A. Thadani from T. Speis, "Generic Safety Issue (GSI)-166, 'Adequacy of Fatigue Life of Metal Components,'" August 26, 1996. [9808210022]
929. Regulatory Guide 1.139, "Guidance for Residual Heat Removal," U.S. Nuclear Regulatory Commission, May 1978.
930. NUREG-0957, "The Price-Anderson Act - The Third Decade," U.S. Nuclear Regulatory Commission, December 1983.
931. NUREG-0689, "Potential Impact of Licensee Default on Cleanup of TMI-2," U.S. Nuclear Regulatory Commission, November 1980.

932. SECY-83-64A, "10 CFR 140: Proposed Rule to Revise the Criteria for Determination of an Extraordinary Nuclear Occurrence," August 9, 1983. [8308250291]
933. Memorandum for A. Kenneke from W. Olmstead, "Chapter 5 of TMI Action Plan," March 16, 1984. [8404040211]
934. Letter to the Honorable Alan Simpson from Joseph Hendrie, March 24, 1981. [8104030556]
935. NUREG/CR-1368, "Development of a Checklist for Evaluating Maintenance, Test and Calibration Procedures Used in Nuclear Power Plants," U.S. Nuclear Regulatory Commission, May 1980.
936. NUREG/CR-1369, "Procedures Evaluation Checklist for Maintenance, Test and Calibration Procedures," U.S. Nuclear Regulatory Commission, May 1980.
937. Memorandum for Chairman Ahearn from W. Dircks, "Manual Chapters Delegation of Authority to Staff Office Directors," December 23, 1980.
938. SECY-80-497, "Review of Delegations of Authority and Other Documentation," November 10, 1980. [8011190612]
939. Federal Register Notice 51 FR 28044, "Safety Goals for the Operations of Nuclear Power Plants," August 4, 1986.
940. Memorandum for T. Speis from H. Thompson, "Longer-Term Generic Actions as a Result of the Davis-Besse Event of June 9, 1985," November 6, 1985. [8511120162]
941. Memorandum for B. Morris from D. Basdekas, "Concerns Related to the Davis-Besse Incident on June 9, 1985," August 13, 1985. [8508230349]
942. Memorandum for F. Gillespie from D. Basdekas, "Concerns Related to the Davis-Besse Incident on June 9, 1985," September 27, 1985. [9909290115]
943. Memorandum for A. DeAgazio from D. Crutchfield, "Davis-Besse Restart Safety Evaluation (TAC No. 59702)," December 17, 1985. [8512230373]
944. Letter to G. Ogeka (BNL) from T. Speis (NRC), "BNL Technical Assistance to the Division of Safety Review and Oversight, Office of Nuclear Reactor Regulation, NRC - 'Reduction of Risk Uncertainty' (FIN A-3846)," April 28, 1986. [9909290117]
945. Memorandum for K. Kniel from R. Riggs, "OTSG Thermal Stress (GI-125.II.4)," June 17, 1986. [8608070348]
946. Memorandum for H. Thompson from R. Bernero, "Auxiliary Feedwater Systems," August 23, 1985. [8509030040]
947. Memorandum for B. Boger from A. Gody, "Implementation of the Resolution for Generic Issue 142, 'Leakage Through Electrical Isolators,'" May 28, 1993. [9803260145]

948. Memorandum for H. Thompson from G. Edison, "Recommendation for Longer Term Generic Action as a Result of Davis-Besse Event of June 9, 1985," September 11, 1985. [9909290121]
949. Memorandum for F. Miraglia from G. Edison, "Prioritization of Generic Issue 125.II.I.D.," April 25, 1986. [8605050358]
950. BAW-1919, "B&W Owners' Group Trip Reduction and Transient Response Improvement Program," May 31, 1986. [8606020079, 8605190153]
951. Memorandum for H. Thompson and W. Minners from F. Rowsome, "Another Generic Safety Issue Suggested by the Davis-Besse Incident of June 9, 1985," September 9, 1985. [8509110328]
952. Memorandum for W. Minners from K. Kniel, "Value/Impact Assessment for Draft CRGR Package Requiring Upgrading of Auxiliary Feedwater Systems in Certain Operating Plants," January 16, 1986. [8601240311]
953. Memorandum for G. Mazetis from A. Marchese, "Revised Outline of Regulatory Analysis for USI A-45," January 14, 1986. [9909290124]
954. Memorandum for V. Stello from E. Beckjord, "Closeout of TMI Action Plan Items," November 13, 1986.
955. Memorandum for W. Dircks from H. Denton, "Close Out of Completed TMI Action Plan Item I.C.9, 'Long-Term Program Plan for Upgrading of Procedures,'" June 7, 1985. [8506200155]
956. Memorandum for V. Stello from H. Denton, "Close-out of the Division of Human Factors Technology TMI Action Plan Items," January 6, 1987. [8701140115]
957. Federal Register Notice 49 FR 46428, "10 CFR Parts 50 and 55, Operator's Licenses and Conforming Amendment," November 26, 1984.
958. Memorandum for T. Speis from T. Novak, "Need for Oversight Guidance - Byron 2-Pump Service Water Issue and Related Generic Issues," May 6, 1986. [8605130362]
959. EGG-EA-5524, "Data Summaries of Licensee Event Reports of Pumps at U.S. Commercial Nuclear Power Plants from January 1, 1972, to September 30, 1980," Idaho National Engineering Laboratory, September 1981.
960. Letter to D. Ericson (Sandia National Laboratories) from J. Mulligan (United Engineers & Constructors), "Decay Heat Removal Systems Evaluations Feasibility and Cost Evaluations of Special Issues Related to Decay Heat Removal," January 20, 1986. [9910200312]
961. NUREG/CR-4627, "Generic Cost Estimates," U.S. Nuclear Regulatory Commission, June 1986, (Rev. 1) February 1989, (Rev. 2) February 1992.
962. NUREG-1021, "Operator Licensing Examiner Standards," U.S. Nuclear Regulatory Commission, October 1983.

963. SECY-85-21, "Policy Statement on Fitness for Duty of Nuclear Power Plant Personnel," January 17, 1985. [8502280427]
964. SECY-85-21A, "Withdrawal Notice: Fitness for Duty of Nuclear Power Plant Personnel," April 12, 1985. [8505030703]
965. SECY-85-21B, "Fitness for Duty of Nuclear Power Plant Personnel," August 26, 1985. [8510150472]
966. Federal Register Notice 50 FR 11147, "10 CFR Ch. 1, Commission Policy Statement on Training and Qualification of Nuclear Power Plant Personnel," March 20, 1985.
967. Federal Register Notice 51 FR 27921, "Commission Policy Statement on Fitness for Duty of Nuclear Power Plant Personnel," August 4, 1986.
968. Memorandum for J. Roe from R. Minogue, "Nuclear Plant Analyzer (NPA) Management Plan," December 12, 1985. [9909290129]
969. NUREG/CR-3403, "Criteria and Test Method for Certifying Air-Purifying Respirator Cartridges and Canisters Against Radioiodine," U.S. Nuclear Regulatory Commission, November 1983.
970. NUREG/CR-3568, "A Handbook for Value-Impact Assessment," U.S. Nuclear Regulatory Commission, December 1983.
971. NUREG/CR-4330, "Review of Light Water Reactor Regulatory Requirements," U.S. Nuclear Regulatory Commission, (Vol. 1) April 1986, (Vol. 2) June 1986.
972. SECY-80-230B, "Update of Chapter V of TMI Action Plan: NRC Policy, Organization, and Management," June 20, 1980. [8009160065]
973. Memorandum for T. Speis from W. Minners, "Schedule for Resolving Generic Issue No. 125.II.1.b, 'Review Existing AFW Systems for Single Failure,'" December 10, 1986. [8612180094]
974. NUREG-1122, "Knowledges and Abilities Catalog for Nuclear Power Plant Operators: Pressurized Water Reactors," U.S. Nuclear Regulatory Commission, July 1985.
975. IE Circular No. 80-02, "Nuclear Power Plant Staff Work Hours," U.S. Nuclear Regulatory Commission, February 1, 1980. [7912190657]
976. NRC Letter to All Licensees of Operating Plants and Applicants for Operating Licenses and Holders of Construction Permits, "Interim Criteria for Shift Staffing (Generic Letter 80-72)," July 31, 1980. [8009020297]
977. Federal Register Notice 47 FR 7352, "Nuclear Power Plant Staff Working Hours," February 18, 1982.
978. Federal Register Notice 47 FR 23836, "Nuclear Power Plant Staff Working Hours," June 1, 1982.

979. NRC Letter to All Licensees of Operating Plants, Applicants for an Operating License, and Holders of Construction Permits, "Nuclear Power Plant Staff Working Hours (Generic Letter No. 82-12)," June 15, 1982. [8206160341]
980. NRC Letter to All Pressurized Power Reactor Licensees, "NUREG-0737 Technical Specifications (Generic Letter No. 82-16)," September 20, 1982. [8209210027]
981. NRC Letter to All Boiling Water Reactor Licensees, "NUREG-0737 Technical Specifications (Generic Letter No. 83-02)," January 10, 1983. [8301110134]
982. NRC Letter to All Licensees of Operating Plants, Applicants for Operating Licenses, and Holders of Construction Permits, "Definition of 'Key Maintenance Personnel,' Clarification of Generic Letter 82-12 (Generic Letter 83-14)," March 7, 1983. [8303040005]
983. Memorandum for W. Dircks from J. Hoyle, "Updating NRC Policy Statements," September 30, 1985. [8611190084]
984. Memorandum for J. Tourtelotte, et al., from S. Chilk, "Addendum to SRM M841218 - Briefing and Discussion on the Hearing Process, 2:00 p.m., Tuesday, December 18, 1984, Commissioners' Conference Room, D.C. Office (Open to Public Attendance)," January 31, 1985. [8502060511]
985. Federal Register Notice 48 FR 50550, "10 CFR Part 2, Rules of Practice for Domestic Licensing Proceedings; Role of NRC Staff in Adjudicatory Licensing Hearings," November 2, 1983.
986. Federal Register Notice 51 FR 36811, "10 CFR Part 2, Rules of Practice for Domestic Licensing Proceedings; Role of NRC Staff in Adjudicatory Licensing Hearings," October 16, 1986.
987. Federal Register Notice 49 FR 14698, "10 CFR Parts 2 and 50, Request for Public Comment on Regulatory Reform Proposal Concerning the Rules of Practice, Rules for Licensing of Production and Utilization Facilities," April 12, 1984.
988. Federal Register Notice 51 FR 24365, "10 CFR Part 2, Rules of Practice for Domestic Licensing Proceedings - Procedural Changes in the Hearing Process," July 3, 1986.
989. Federal Register Notice 50 FR 13978, "10 CFR Part 140, Criteria for an Extraordinary Nuclear Occurrence," April 9, 1985.
990. Memorandum for J. Funches from F. Rowsome, "Handling of DHFT Issues in GIMCS," June 6, 1986. [8606120789]
991. Memorandum for T. Speis from R. Bernero, "Resolution of Comment No. 9 of CRGR/OIA Issues on Potential Generic Concerns Regarding BWR Drywell Coolers," July 31, 1986. [8608190656]
992. Federal Register Notice 50 FR 42145, "10 CFR Part 1, Statement of Organization and General Information," October 18, 1985.

993. NUREG-1220, "Training Review Criteria and Procedures," U.S. Nuclear Regulatory Commission, July 1986.
994. Federal Register Notice 48 FR 31611, "10 CFR Part 50, Licensed Operator Staffing at Nuclear Power Plants," July 11, 1983.
995. Regulatory Guide 1.114, "Guidance on Being Operator at the Controls of a Nuclear Power Plant," U.S. Nuclear Regulatory Commission, February 1976 [8012110846], (Rev. 1) November 1976 [8307070393], (Rev. 2) May 1989 [8906200342].
996. Federal Register Notice 50 FR 43621, "Commission Policy Statement on Engineering Expertise on Shift," October 28, 1985.
997. Memorandum for W. Dircks from H. Denton, "Human Factors Program Plan (HFPP)," December 6, 1984. [8501080482]
998. Memorandum for T. Speis from H. Denton, "Resolution of Generic Safety Issue 61, 'SRV Line Break Inside the Wetwell Airspace of Mark I and II Containments,'" August 8, 1986. [8608180209]
999. NUREG/CR-4594, "Estimated Safety Significance of Generic Safety Issue 61," U.S. Nuclear Regulatory Commission, June 1986.
1000. Memorandum for T. Speis, et al., from R. Mattson, "Generic Issue 23, 'Reactor Coolant Pump Seal Failures' - Task Action Plan," October 26, 1983. [8311080469]
1001. Memorandum for H. Denton from T. Speis, "Integration of Electrical Power Issues into Proposed Generic Issue 128, 'Electrical Power Reliability,'" November 28, 1986. [8612080528]
1002. Memorandum for H. Clayton from B. Sheron, "Criteria for Initiating Feed and Bleed," September 13, 1985. [8509180314]
1003. Memorandum for W. Russell from K. Perkins, "Generic Issue 125.I.8, 'Procedures and Staffing for Reporting to NRC Operations Center,'" November 25, 1986. [8612050442]
1004. Memorandum for G. Lainas and D. Crutchfield from F. Rowsome, "Davis-Besse Restart Considerations," August 13, 1985. [8508210208]
1005. Memorandum for V. Stello from D. Ward, "ACRS Comments on Proposed Resolution of Generic Issue 124, 'Auxiliary Feedwater System Reliability,'" September 17, 1986. [8609230137]
1006. NUREG-1195, "Loss of Integrated Control System Power and Overcooling Transient at Rancho Seco on December 26, 1985," U.S. Nuclear Regulatory Commission, February 1986.
1007. Memorandum for T. Speis from F. Miraglia, "Generic Action as a Result of the Rancho Seco Event of December 26, 1985," May 14, 1986. [8605200493]

1008. Memorandum for E. Jordan from G. Holahan, "Proposed IE Information Notice," June 6, 1986. [8606110821]
1009. NUREG/CR-4568, "A Handbook for Quick Cost Estimates," U.S. Nuclear Regulatory Commission, April 1986.
1010. IE Information Notice No. 86-61, "Failure of Auxiliary Feedwater Manual Isolation Valve," U.S. Nuclear Regulatory Commission, July 28, 1986. [8607240026]
1011. NUREG-1177, "Safety Evaluation Report Related to the Restart of Davis-Besse Nuclear Power Station, Unit 1, Following the Event of June 9, 1985," U.S. Nuclear Regulatory Commission, June 1986.
1012. Federal Register Notice 50 FR 29937, "10 CFR Part 50, Analysis of Potential Pressurized Thermal Shock Events," July 23, 1985.
1013. NUREG-1212, "Status of Maintenance in the U.S. Nuclear Power Industry 1985," U.S. Nuclear Regulatory Commission, (Volumes 1 and 2), June 1986.
1014. Memorandum for F. Schroeder from D. Crutchfield, "Dynamic Qualification Testing of Large Bore Hydraulic Snubbers," March 6, 1985. [8503180471]
1015. Memorandum for R. DeYoung, et al., from C. Heltemes, "Failure of Large Hydraulic Snubbers to Lock-up," September 21, 1984. [8410290312, 8410290114]
1016. NUREG/CR-4334, "An Approach to the Quantification of Seismic Margins in Nuclear Power Plants," U.S. Nuclear Regulatory Commission, August 1985.
1017. NUREG/CR-4279, "Aging and Service Wear of Hydraulic and Mechanical Snubbers Used on Safety-Related Piping and Components of Nuclear Power Plants," U.S. Nuclear Regulatory Commission, February 1986.
1018. NUREG/CR-4263, "Reliability Analysis of Stiff Versus Flexible Piping Final Project Report," U.S. Nuclear Regulatory Commission, May 1985.
1019. NUREG/CR-3756, "Seismic Hazard Characterization of the Eastern United States," U.S. Nuclear Regulatory Commission, April 1984.
1020. NRC Letter to All Power Reactor Licensees (Except SEP Licensees) and All Applicants for Licenses to Operate Power Reactors, "Technical Specification for Snubbers (Generic Letter 84-13)," May 3, 1984. [8405040043]
1021. EPRI NP-2297, "Snubber Reliability Improvement Study," Electric Power Research Institute, March 1982.
1022. NUREG-1144, "Nuclear Plant Aging Research (NPAR) Program Plan," U.S. Nuclear Regulatory Commission, July 1985.
1023. SECY-86-231, "Survey on Engineering Expertise on Shift," August 6, 1986. [8608200375]

1024. Memorandum for K. Kniel from C. Ferrell, "Modification of Generic Issue No. 106, 'Highly Combustible Gases in Vital Areas,'" February 20, 1986. [8602280811]
1025. IE Information Notice No. 83-41, "Actuation of Fire Suppression System Causing Inoperability of Safety-Related Equipment," U.S. Nuclear Regulatory Commission, June 22, 1983. [8305110477]
1026. Letter to D. Farrar (Commonwealth Edison Co.) from J. Zwolinski (NRC), "Technical Specifications Relating to the Use of a Mobile Volume Reduction System (MVRS) at Dresden Station (TAC 56373, 56374)," August 13, 1986. [8608210177]
1027. Memorandum for D. Eisenhut from G. Lainas, "Summary of the Operating Reactor Events Meeting," January 28, 1982. [8310260053]
1028. Memorandum for R. Vollmer and E. Jordan from C. Michelson, "Effects of Fire Protection System Actuation on Safety Related Equipment," January 28, 1982. [8202220663]
1029. "Guidelines for Permanent BWR Hydrogen Water Chemistry Installations," BWR Owners Group for IGSCC Research, Hydrogen Installation Subcommittee, Electric Power Research Institute, 1987.
1030. NASA TMX-71565, "Review of Hydrogen Accidents and Incidents in NASA Operation," National Aeronautics and Space Administration, August 1974.
1031. Memorandum for T. Murley from E. Beckjord, "A New Generic Issue: Multiple Steam Generator Tube Leakage," June 16, 1992. [9212040356]
1032. Memorandum for H. Denton from T. Speis, "Earthquakes and Emergency Planning," January 18, 1984. [8402020014]
1033. Letter to W. Dircks (NRC) from S. Sholly (Union of Concerned Scientists), December 22, 1983. [8502270371]
1034. Letter to J. Asselstine (NRC) from S. Sholly (Union of Concerned Scientists), December 22, 1983. [8502090516]
1035. SECY-85-283, "Final Amendments to 10 CFR Part 50, Appendix E; Consideration of Earthquakes in Emergency Planning," August 21, 1985. [8508300319]
1036. IE Bulletin No. 85-03, "Motor-Operated Valve Common Mode Failures During Plant Transients Due to Improper Switch Settings," U.S. Nuclear Regulatory Commission, November 15, 1985 [8511130441], (Supplement 1) April 27, 1988 [8804210018].
1037. SECY-83-484, "Requirements for Emergency Response Capability," November 29, 1983. [8312130459]
1038. IE Information Notice No. 86-10, "Safety Parameter Display System Malfunctions," U.S. Nuclear Regulatory Commission, February 13, 1986. [8602100408]

1039. Memorandum for H. Denton from T. Speis, "Prioritization of Selected MPAs (Operating Plan, Item VI.B.6.b)," October 19, 1984. [8411010640]
1040. NUREG/CR-3246, "The Effect of Some Operations and Control Room Improvements on the Safety of the Arkansas Nuclear One, Unit One, Nuclear Power Plant," U.S. Nuclear Regulatory Commission, June 1983.
1041. Memorandum for K. Kniel from R. Bosnak, "Request for Subsumption of Generic Issue B-6 (GI B-6) Into Generic Issue 119.1 (GI 119.1)," January 8, 1987. [8701200186]
1042. SECY-87-101, "Issues and Proposed Options Concerning Degree Requirement for Senior Operators," April 16, 1987. [8706030157]
1043. SECY-86-348, "Final Rulemaking for Revisions to Operator Licensing - 10 CFR 55 and Conforming Amendments," November 21, 1986. [8701020003]
1044. Federal Register Notice 52 FR 16007, "Regulatory Guides; Issuance and Availability," May 1, 1987.
1045. Memorandum for V. Stello from E. Beckjord, "Resolution of TMI Action Plan Items and Human Factors Issues," May 18, 1987. [8710280270]
1046. Memorandum for V. Stello from E. Beckjord, "Closeout of TMI Action Plan Item," February 27, 1987. [9704150146]
1047. Memorandum for K. Kniel from B. Sheron, "Request for the Prioritization of a Generic Issue on the Reliability of PWR Main Steam Safety Valves," May 27, 1986. [8604030313]
1048. IE Information Notice No. 86-05, "Main Steam Safety Valve Test Failures and Ring Setting Adjustments," U.S. Nuclear Regulatory Commission, January 31, 1986 [8601290054], (Supplement 1) October 16, 1986 [8610100107].
1049. Memorandum for F. Cherny from R. Baer, "50.55(e) Report on Crosby Main Steam Valve Ring Settings," February 5, 1985. [8502140267, 9704090262]
1050. Memorandum for R. Bosnak from F. Cherny, "Trip Report - Meeting of ASME Section III Subgroup on Pressure Relief, February 11, 1987," March 13, 1987. [8703190114]
1051. INPO 82-025, "Review of NRC Report: Precursors to Potential Severe Core Damage Accidents: 1969-1979 A Status Report, NUREG/CR-2497," Institute for Nuclear Power Operations, September 1982.
1052. NUREG/CR-2228, "Containment Response During Degraded Core Accidents Initiated by Transients and Small Break LOCA in the Zion/Indian Point Reactor Plants," U.S. Nuclear Regulatory Commission, July 1981.
1053. NUREG/CR-4752, "Coincident Steam Generator Tube Rupture and Stuck-Open Safety Relief Valve Carryover Test," U.S. Nuclear Regulatory Commission, March 1987.

1054. Memorandum for W. Russell, et al., from R. Starostecki, "Request for Regional Inspection to Verify Adequate Flow Capacity of Main Steam Code Safety Valves and Proper Ring Adjustments," November 8, 1987. [8711120155]
1055. AEOD/C204, "San Onofre Unit 1 Loss of Salt Water Cooling Event on March 10, 1980," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, July 1982. [8208260403]
1056. NUREG-0869, "USIA-43 Regulatory Analysis," U.S. Nuclear Regulatory Commission, (Rev. 1) October 1985.
1057. NUREG-0897, "Containment Emergency Sump Performance," U.S. Nuclear Regulatory Commission, (Rev. 1) October 1985.
1058. Regulatory Guide 1.82, "Water Sources for Long-Term Recirculation Cooling Following a Loss-of-Coolant Accident," U.S. Nuclear Regulatory Commission, June 30, 1974 [7902090041], (Rev. 1) November 30, 1985 [8512100138], (Rev. 2) May 31, 1996 [9605210504].
1059. NRC Letter to All Licensees of Operating Reactors, Applicants for Operating Licenses, and Holders of Construction Permits, "Potential for Loss of Post-LOCA Recirculation Capability Due to Insulation Debris Blockage (Generic Letter 85-22)," December 3, 1985. [8511270253]
1060. SECY-85-349, "Resolution of Unresolved Safety Issue A-43, 'Containment Emergency Sump Performance,'" October 31, 1985. [8511070302]
1061. NUREG-0649, "Task Action Plans for Unresolved Safety Issues Related to Nuclear Power Plants," U.S. Nuclear Regulatory Commission, February 1980, (Rev. 1) September 1984.
1062. Federal Register Notice 51 FR 39390, "10 CFR Part 50, Emergency Planning and Preparedness; Withdrawal," October 28, 1986.
1063. NUREG/CR-3017, "Correlation of Seismic Experience Data in Non-Nuclear Facilities with Seismic Equipment Qualification in Nuclear Plants (A-46)," U.S. Nuclear Regulatory Commission, August 1983.
1064. NUREG/CR-3875, "The Use of In-Situ Procedures for Seismic Qualification of Equipment in Currently Operating Plants," U.S. Nuclear Regulatory Commission, June 1984.
1065. NUREG/CR-3357, "Identification of Seismically Risk Sensitive Systems and Components in Nuclear Power Plants," U.S. Nuclear Regulatory Commission, June 1983.
1066. NUREG/CR-3266, "Seismic and Dynamic Qualification of Safety-Related Electrical and Mechanical Equipment in Operating Nuclear Power Plants," U.S. Nuclear Regulatory Commission, September 1983.
1067. NUREG-1211, "Regulatory Analysis for Resolution of Unresolved Safety Issue A-46, 'Seismic Qualification of Equipment in Operating Plants,'" U.S. Nuclear Regulatory Commission, February 1987.

1068. Regulatory Guide 1.154, "Format and Content of Plant-Specific Pressurized Thermal Shock Safety Analysis Reports for Pressurized Water Reactors," U.S. Nuclear Regulatory Commission, January 1987.
1069. NRC Letter to All Holders of Operating Licenses Not Reviewed to Current Licensing Criteria on Seismic Qualification of Equipment, "Verification of Seismic Adequacy of Mechanical and Electrical Equipment in Operating Reactors, Unresolved Safety Issue (USI) A-46 (Generic Letter 87-02)," February 19, 1987. [8702200135]
1070. NUREG-1216, "Safety Evaluation Report Related to the Operability and Reliability of Emergency Diesel Generators Manufactured by Transamerica Delaval, Inc.," U.S. Nuclear Regulatory Commission, August 1986.
1071. Memorandum for T. Speis, et al., from C. Berlinger, "Closeout of Generic Issue 91 - TDI Emergency Diesel Generator Reliability," September 3, 1987. [8709080427]
1072. Memorandum for W. Russell from T. Speis, "Generic Issue 125.II.13 - Operator Job Aids," June 12, 1986. [8606250128]
1073. SECY-83-288, "Pressurized Thermal Shock (PTS) Rule," July 15, 1983. [8307270206]
1074. Memorandum for W. Dircks from S. Chilk, "SECY-83-288, 'Proposed Pressurized Thermal Shock (PTS) Rule,'" January 13, 1984. [8402100267]
1075. Memorandum for K. Kniel from R. Bosnak, "Integration of NUREG-0933 Issues," May 27, 1986. [8606090491]
1076. NUREG-1230, "Compendium of ECCS Research for Realistic LOCA Analysis," U.S. Nuclear Regulatory Commission, December 1988. [8903030340]
1077. Federal Register Notice 52 FR 9453, "10 CFR Parts 50 and 55, Operators' Licenses and Conforming Amendments," March 25, 1987.
1078. AEOD/C701, "Air Systems Problems at U.S. Light Water Reactors," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, March 1987. [8707240066]
1079. NUREG-1275, "Operating Experience Feedback Report," U.S. Nuclear Regulatory Commission, (Vol. 1) July 1987, (Vol. 2) December 1987, (Vol. 3) November 1988, (Vol. 4) March 1989, (Vol. 5) March 1989, (Vol. 5, Addendum) August 1989, (Vol. 6) February 1991, (Vol. 7) September 1992, (Vol. 8) December 1992, (Vol. 9) March 1993.
1080. NUREG/CR-4374, "A Review of the Oconee-3 Probabilistic Risk Assessment," U.S. Nuclear Regulatory Commission, (Vol. 1) March 1986, (Vol. 2) March 1986, (Vol. 3) June 1986.
1081. NUREG-1150, "Severe Accident Risks: An Assessment for Five U.S. Nuclear Power Plants," U.S. Nuclear Regulatory Commission, (Vol. 1) December 1990, (Vol. 2) December 1990, (Vol. 3) January 1991.

1082. NUREG/CR-3673, "Economic Risks of Nuclear Power Reactor Accidents," U.S. Nuclear Regulatory Commission, May 1984.
1083. Memorandum for T. Speis from F. Gillespie, "Review of RES Proposed Prioritization of Generic Issue (GI) 125.II.11, 'Recovery of Main Feedwater as an Alternative to Auxiliary Feedwater,'" April 27, 1988. [8805120322]
1084. NUREG-1258, "Evaluation Procedure for Simulation Facilities Certified Under 10 CFR 55," U.S. Nuclear Regulatory Commission, December 1987.
1085. NRC Letter to All Operating Reactor Licensees, Applicants for an Operating License and Holders of Construction Permits for Babcock & Wilcox Pressurized Water Reactors, "Safety Evaluation of 'Abnormal Transient Operating Guidelines,' (Generic Letter 83-31)," September 19, 1983. [8309190017]
1086. Memorandum for B. Morris from B. Sheron, "LOCA Concern of SCE Employee," April 28, 1987. [9704150141]
1087. Federal Register Notice 50 FR 27006, "10 CFR Part 50, Modification of General Design Criterion 4 Requirements for Protection Against Dynamic Effects of Postulated Pipe Ruptures," July 1, 1985.
1088. UCID-20397, "Assessment of Value-Impact Associated with the Elimination of Postulated Pipe Ruptures from the Design Basis for Nuclear Power Plants," Lawrence Livermore National Laboratory, March 29, 1985.
1089. Letter to the Honorable Edward J. Markey (Committee on Energy and Commerce, U.S. House of Representatives) from L. Zech (NRC), March 20, 1987. [8703270224]
1090. GAO/RCED-88-73, "Nuclear Regulation - Action Needed to Ensure that Utilities Monitor and Repair Pipe Damage," U.S. General Accounting Office, March 1988.
1091. Memorandum for D. Morrison from H. Thompson, "Generic Issue Management Control System," January 17, 1997. [9803260111]
1092. EPRI NP-5410, "Nondestructive Evaluation of Ferritic Piping for Erosion-Corrosion," Electric Power Research Institute, September 1987.
1093. NRC Bulletin No. 87-01, "Thinning of Pipe Walls in Nuclear Power Plants," U.S. Nuclear Regulatory Commission, July 9, 1987. [8707020018]
1094. SECY-88-50, "Status Report on Pipe Wall Thinning (Responses to NRC Bulletin 87-01 on Pipe Wall Thinning in Nuclear Power Plants)," February 22, 1988. [8809090066]
1095. NRC Information Notice No. 88-17, "Summary of Responses to NRC Bulletin 87-01, Thinning of Pipe Walls in Nuclear Power Plants," U.S. Nuclear Regulatory Commission, April 22, 1988. [8804180039]
1096. SECY-88-50A, "Report on the Meeting with NUMARC, EPRI, and INPO on Status of Industry's Erosion/Corrosion Program," May 10, 1988. [8805230074]

1097. Memorandum to J. Taylor and W. Parler from S. Chilk, "COMSECY-93-029 - Draft Rulemaking Package on License Renewal; SECY-93-049 - Implementation of 10 CFR Part 54, 'Requirements for Renewal of Operating Licenses for Nuclear Power Plants'; SECY-93-113 - Additional Implementation Information for 10 CFR Part 54, 'Requirements for Renewal of Operating Licenses for Nuclear Power Plants,'" June 28, 1993. [9409010107]
1098. Memorandum for V. Stello from T. Murley, "Resolution of Generic Issue I.A.4.2(4) 'Review Simulators for Conformance to Criteria,'" May 28, 1988. [8806020275]
1099. Memorandum for B. Morris from B. Sheron, "Updated GIMCS for GI I.D.5(5)," February 2, 1988. [9704150145]
1100. Memorandum for V. Stello from E. Beckjord, "Redesignation of Generic Issue I.D.5(5), 'Disturbance Analysis Systems,'" February 22, 1988. [8809190312]
1101. Memorandum for V. Stello from E. Beckjord, "Closure of Generic Issue I.D.4 'Control Room Design Standard,'" March 28, 1988. [9704160014]
1102. Memorandum for T. Speis from R. Houston, "Integration of Generic Issue Resolution," November 4, 1987. [9704150161]
1103. Memorandum for V. Stello from E. Beckjord, "Resolution of Generic Safety Issue II.E.4.3, 'Containment Integrity Check,'" March 22, 1988. [8809150125]
1104. NUREG-1273, "Technical Findings and Regulatory Analysis for Generic Safety Issue II.E.4.3, 'Containment Integrity Check,'" U.S. Nuclear Regulatory Commission, April 1988.
1105. Memorandum for T. Speis from G. Arlotto, "Generic Issues Program," January 14, 1988. [9704160053]
1106. Memorandum for R. Baer from G. Bagchi, "Proposed Resolution of Generic Issue B-5, 'Buckling of Steel Containment,'" March 1, 1988. [8804270290]
1107. Memorandum for E. Beckjord from G. Arlotto, "Closeout of Generic Issue B-5, Buckling Behavior of Steel Containments," April 28, 1988. [8805050117]
1108. NUREG-1109, "Regulatory/Backfit Analysis for the Resolution of Unresolved Safety Issue A-44, Station Blackout," U.S. Nuclear Regulatory Commission, June 1988.
1109. Federal Register Notice 53 FR 23203, "10 CFR 50, Station Blackout," June 21, 1988.
1110. Regulatory Guide 1.155, "Station Blackout," U.S. Nuclear Regulatory Commission, June 1988. [8907270193]
1111. NRC Letter to All Licensees of Operating Boiling Water Reactors (BWRs), and Holders of Construction Permits, "NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping (Generic Letter 88-01)," January 25, 1988. [8801260537]
1112. IE Bulletin No. 85-01, "Steam Binding of Auxiliary Feedwater Pumps," U.S. Nuclear Regulatory Commission, October 29, 1985. [8510250539]

1113. NRC Letter to All Licensees, Applicants for Operating Licenses, and Holders of Construction Permits for Pressurized Water Reactors, "Resolution of Generic Safety Issue 93, 'Steam Binding of Auxiliary Feedwater Pumps' (Generic Letter 88-03)," February 17, 1988. [8802180267]
1114. Memorandum for E. Beckjord from T. Murley, "Resolution of Generic Safety Issue 93, 'Steam Binding of Auxiliary Feedwater Pumps,'" August 14, 1987. [8708210408]
1115. Memorandum for E. Beckjord from F. Gillespie, "Review of RES-Proposed Prioritization of Generic Issue No. 136, 'Storage and Use of Large Quantities of Cryogenic Combustibles on Site,'" March 25, 1988. [8804050182]
1116. Federal Register Notice 53 FR 9430, "Final Commission Policy Statement on Maintenance of Nuclear Power Plants," March 23, 1988.
1117. Memorandum for V. Stello from T. Murley, "Closeout of Generic Issue HF-08, 'Maintenance and Surveillance Program,'" May 4, 1988. [8805160004]
1118. SECY-88-248, "Implementation of the Severe Accident Policy for Future Light Water Reactors," September 6, 1988. [8809160019]
1119. NUREG/CR-4780, "Procedures for Treating Common Cause Failures in Safety and Reliability Studies," U.S. Nuclear Regulatory Commission, (Vol. 1) January 1988, (Vol. 2) January 1989.
1120. NUREG-1192, "An Investigation of the Contributors to Wrong Unit or Wrong Train Events," U.S. Nuclear Regulatory Commission, April 1986.
1121. Information Notice No. 87-25, "Potentially Significant Problems Resulting from Human Error Involving Wrong Unit, Wrong Train, or Wrong Components," U.S. Nuclear Regulatory Commission, June 11, 1987. [8706050211]
1122. Memorandum for V. Stello from T. Murley, "Final Resolution of Generic Issue (GI) 102: Human Error in Events Involving Wrong Unit or Wrong Train," September 12, 1988. [8810070118]
1123. Dam Failure Model, Pacific Northwest Laboratories, October 1983.
1124. "Analysis of Gradual Earth-Dam Failure," Journal of Hydraulic Engineering, Volume 114, No. 1, American Society of Civil Engineers, January 1988.
1125. "Use of A Dam Break Model to Assess Flooding at Haddam Neck Nuclear Power Plant," Water Resources Bulletin, Vol. 20, No. 6, American Water Resources Association, December 1984.
1126. Technical Evaluation Report, "Quabbin Dam Failure Flooding Consequences at Haddam Neck Plant," Franklin Research Center, August 25, 1983.
1127. "Dam Breach Parameters, Outflow Peaks, and Flood Stages," International Symposium on Hydrometeorology, American Water Resources Association, June 1982.

1128. PB82-224577, "Application of and Guidelines for Using Available DAM Break Models," Tennessee Water Resources Research Center, May 1981.
1129. IE Bulletin No. 82-02, "Degradation of Threaded Fasteners in the Reactor Coolant Pressure Boundary of PWR Plants," U.S. Nuclear Regulatory Commission, June 2, 1982. [8204210380]
1130. RIL 158, "Operational Safety Reliability Program," U.S. Nuclear Regulatory Commission, October 31, 1988. [8811070111]
1131. Memorandum for V. Stello from E. Beckjord, "Closure of Generic Issue II.C.4, 'Reliability Engineering,'" October 31, 1988. [8811150124]
1132. Memorandum for E. Beckjord from F. Gillespie, "Generic Issue 139, 'Thinning of Carbon Steel Piping in LWRs,'" December 27, 1988. [8901130015]
1133. NUREG-1332, "Regulatory Analysis for the Resolution of Generic Issue 125.II.7, 'Reevaluate Provision to Automatically Isolate Feedwater from Steam Generator During a Line Break,'" U.S. Nuclear Regulatory Commission, September 1988.
1134. Memorandum for V. Stello from E. Beckjord, "Resolution of Generic Issue 125.II.7, 'Reevaluate Provision to Automatically Isolate Feedwater from Steam Generator During a Line Break,'" September 9, 1988. [8811290524]
1135. NUREG-0848, "Final Environmental Statement Related to the Operation of Byron Station Units 1 and 2," U.S. Nuclear Regulatory Commission, April 1982.
1136. Memorandum to C. Miller from R. Borchardt, "Review of Temporary Instruction 2515/131, 'Licensee Offsite Communication Capabilities,' for Deletion from the NRC Inspection Manual," December 3, 1996. [9612060074]
1137. SECY-86-97, "Steam Generator USI Program - Utility Responses to Staff Recommendations in Generic Letter 85-02," March 24, 1986. [8609160048]
1138. NRC Bulletin No. 88-02, "Rapidly Propagating Fatigue Cracks in Steam Generator Tubes," February 5, 1988. [8802020035]
1139. SECY-88-272, "Technical Resolution of Unresolved Safety Issues A-3, A-4, and A-5 Regarding Steam Generator Tube Integrity," September 27, 1988. [8811040042]
1140. NRC Information Notice No. 87-28, "Air Systems Problems at U.S. Light Water Reactors," June 22, 1987 [8706170115], (Supplement 1) December 28, 1987 [8712230003].
1141. NRC Letter to All Holders of Operating Licenses or Construction Permits for Nuclear Power Plants, "Instrument Air Supply System Problems Affecting Safety-Related Equipment (Generic Letter 88-14)," August 8, 1988. [8808120294]
1142. Memorandum for V. Stello from E. Beckjord, "Resolution of Generic Issue 43, Air Systems Reliability," September 30, 1988. [9704160039]

1143. SECY-88-260, "Shutdown Decay Heat Removal Requirements (USI A-45)," September 13, 1988. [8811040098]
1144. NUREG/CR-5015, "Improved Reliability of Residual Heat Removal Capability in PWRs as Related to Resolution of Generic Issue 99," U.S. Nuclear Regulatory Commission, May 1988.
1145. NRC Letter to All Holders of Operating Licenses or Construction Permits for Pressurized Water Reactors (PWRs), "Loss of Decay Heat Removal (Generic Letter No. 88-17) 10 CFR 50.54f," October 17, 1988. [8810180350]
1146. Memorandum for V. Stello from E. Beckjord, "Resolution of Generic Issue 99, 'Loss of RHR Capability in PWRs,'" November 2, 1988. [8811290361]
1147. Memorandum for V. Stello from T. Murley, "Final Resolution of Generic Issue (GI) 66, 'Steam Generator Requirements,'" November 28, 1988. [8812010081]
1148. Memorandum for W. Minners from F. Rowsome, "A Candidate Generic Issue," December 11, 1984. [8501080138]
1149. Regulatory Guide 1.65, "Materials and Inspections for Reactor Vessel Closure Studs," U.S. Atomic Energy Commission, October 1973. [7907100246]
1150. Memorandum for W. Minners from A. Thadani, "Prioritization of RHR Suction Valve Testing," May 7, 1984. [8405180403]
1151. NUREG/CR-2934, "Review and Evaluation of the Indian Point Probabilistic Safety Study," U.S. Nuclear Regulatory Commission, December 1982.
1152. NUREG/CR-3300, "Review and Evaluation of the Zion Probabilistic Safety Study," U.S. Nuclear Regulatory Commission, (Vol. 1) May 1984.
1153. Memorandum for F. Cherny from W. Minners, "Reactor Coolant System Pressure Isolation Valve (PIV) Leak Test Requirements," July 2, 1985. [8507120595]
1154. Regulatory Guide 1.25, "Assumptions Used for Evaluating the Potential Radiological Consequences of a Fuel Handling Accident in the Fuel Handling and Storage Facility for Boiling and Pressurized Water Reactors," U.S. Atomic Energy Commission, March 1972. [7907100118]
1155. Memorandum for R. Bernero from T. Speis, "Relationship of TIA 84-72 (Haddam Neck Refueling Cavity Seal Failure) to Generic Issue No. 82 (Beyond Design Basis Accidents in Spent Fuel Pools)," April 11, 1985. [8504240705]
1156. Memorandum for K. Kniel from W. Minners, "Refueling Cavity Seal Failure," April 1, 1986. [8604080427]
1157. NUREG/CR-4982, "Severe Accidents in Spent Fuel Pools in Support of Generic Safety Issue 82," U.S. Nuclear Regulatory Commission, July 1987.

1158. IE Bulletin No. 84-03, "Refueling Cavity Seal Failure," U.S. Nuclear Regulatory Commission, August 24, 1984. [8408240358]
1159. Letter to D. Crutchfield (NRC) from W. Council (Connecticut Yankee Atomic Power Company), "Haddam Neck Plant Reactor Cavity Seal Ring Failure," September 12, 1984. [8409250335]
1160. Memorandum for K. Kniel from W. Minners, "Refueling Cavity Seal Failure," May 8, 1986. [8605210217]
1161. Memorandum for K. Kniel from W. Minners, "Proposed Generic Issue - Fission Product Removal by Containment Sprays or Pools," March 10, 1987. [8703170451]
1162. Federal Register Notice 54 FR 3701, "[NUREG-0800] Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants; Issuance and Availability," January 25, 1989.
1163. Memorandum for T. Speis from K. Kniel, "Treatment of Lessons-Learned from Surry Event as Related to Generic Issues," March 31, 1987. [8704030542]
1164. Memorandum for T. Speis from R. Bernero, "Prioritization of Generic Issue - Valve Interlocks to Prevent Vessel Draining During Shutdown Cooling," May 21, 1986. [8606120635]
1165. AEOD/E609, "Inadvertent Draining of Reactor Vessel During Shutdown Cooling Operation," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, August 1986. [8608290176, 8608290040]
1166. Memorandum for T. King from K. Kniel, "Additional Comments Regarding Prioritization of Generic Issue-129, 'Residual Heat Removal System Valve Mis-alignment during Shutdown Cooling Operations,'" December 7, 1988. [8812210402]
1167. Memorandum for W. Minners from B. Sheron, "Proposed Generic Issue, 'Potential Seismic Interaction Involving the Movable In-Core Flux Mapping System Used in Westinghouse Designed Plants,'" August 27, 1985. [8509050358]
1168. Letter to F. Miraglia (NRC) from G. Goering (Westinghouse Owners Group), "Potential Seismic Interaction Associated with the Flux Mapping System in Westinghouse Plants," June 10, 1985. [8509050363]
1169. NUREG/CR-2000, "Licensee Event Report (LER) Compilation," U.S. Nuclear Regulatory Commission, (Vol. 3, No. 7) August 1984.
1170. Memorandum for T. King from R. Riggs, "Computer Program 'SEALCOM' Used in Generic Issue 131," May 1, 1989. [9704160010]
1171. IE Information Notice No. 85-45, "Potential Seismic Interaction Involving the Movable In-Core Flux Mapping System Used in Westinghouse Designed Plants," U.S. Nuclear Regulatory Commission, June 6, 1985. [8506060677]

1172. Letter to R. Engelken (NRC) from H. Ray (Southern California Edison Company), "Docket No. 50-361, Licensee Event Report, Numbers 82-002 and 82-003, San Onofre Nuclear Generating Station, Unit 2," March 30, 1982. [8204140262]
1173. Letter to R. Haynes (NRC) from C. Mathis (Boston Edison Company), "Docket No. 50-293, License DPR-35," September 15, 1982. [8209280087]
1174. NUREG-1251, "Implications of the Accident at Chernobyl for Safety Regulation of Commercial Nuclear Power Plants in the United States," U.S. Nuclear Regulatory Commission, (Vols. I and II) April 1989.
1175. SECY-89-081, "Final Report on Chernobyl Implications," March 7, 1989. [8903200205]
1176. AEOD/S801, "Significant Events that Involved Procedures," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, March 1988. [8907310351, 8906090032]
1177. NRC Letter to All Holders of Operating Licenses, Applicants for Operating Licenses and Holders of Construction Permits for Power Reactors, "NRC Use of the Terms, 'Important to Safety' and 'Safety Related'" (Generic Letter 84-01)," January 5, 1984. [8401050382]
1178. NRC Memorandum and Order CLI-84-9, June 6, 1984. [8406070146]
1179. SECY-85-119, "Issuance of Proposed Rule on the Important-to-Safety Issue," April 5, 1985. [8505030656]
1180. Memorandum for W. Dircks from S. Chilk, "Staff Requirements -- SECY-85-119 - 'Issuance of Proposed Rule on the Important-to-Safety Issue,'" December 31, 1985. [8601160559]
1181. SECY-86-164, "Proposed Rule on the Important-to-Safety Issue," May 29, 1986. [8607010004]
1182. Memorandum for V. Stello from E. Beckjord, "Resolution of Generic Issue I.F.1, "Expand QA List,"" January 12, 1989. [9704150147]
1183. Memorandum for W. Minners from L. Engle, "Generic Implications/LLNL Technical Evaluation Report on Seven Main Transformer Failures at the North Anna Power Station, Units 1 and 2," November 16, 1984. [8411270057]
1184. UCID-20053, "Technical Evaluation Report on the Seven Main Transformer Failures at the North Anna Power Station, Units 1 and 2," Lawrence Livermore National Laboratory, March 29, 1984. [8412120181, 8412070065]
1185. Regulatory Guide 1.120, "Fire Protection Guidelines for Nuclear Power Plants," U.S. Nuclear Regulatory Commission, (Draft) June 1976, (Draft) November 1977.
1186. NUREG/CR-3862, "Development of Transient Initiating Event Frequencies for Use in Probabilistic Risk Assessments," U.S. Nuclear Regulatory Commission, May 1985.

1187. Memorandum for V. Stello from E. Beckjord, "Closeout of Generic Issue II.F.5, 'Classification of Instrumentation, Control and Electrical Equipment,'" May 5, 1989. [8906270390]
1188. SECY-83-221, "Prioritization of Generic Safety Issues," June 7, 1983. [8306150099]
1189. Memorandum for W. Dircks from S. Chilk, "SECY-83-221 - Prioritization of Generic Safety Issues," December 9, 1983. [9704150148]
1190. Federal Register Notice 43 FR 1565, "Program for Resolution of Generic Issues Related to Nuclear Power Plants," January 10, 1978.
1191. Federal Register Notice 54 FR 24432, "Program for Resolution of Generic Issues Related to Nuclear Power Plants; Policy Statement," June 7, 1989.
1192. RES Office Letter No. 1, "Procedure for Identification, Prioritization, Resolution, and Tracking of Generic Issues," Office of Nuclear Regulatory Research, December 3, 1987 [9704150149], (Rev. 1) March 22, 1989 [9609200344], (Rev. 2) July 12, 1991 [9107250098], (Rev. 3) November 26, 1991 [9704150154], (Rev. 4) June 2, 1994 [9704150218].
1193. RES Office Letter No. 2, "Procedures for Obtaining Regulatory Impact Analysis Review and Support," Office of Nuclear Regulatory Research, November 18, 1988. [8901180069]
1194. RES Office Letter No. 3, "Procedure and Guidance for the Resolution of Generic Issues," May 10, 1988 [8809220069], (Rev. 1) December 21, 1988 [9704100054], (Rev. 2) March 27, 1989 [9609200351].
1195. NUREG/CR-1251, "Implications of the Accident at Chernobyl for Safety Regulation of Commercial Nuclear Power Plants in the United States," U.S. Nuclear Regulatory Commission, (Vol. 1) April 1989, (Vol. 2) April 1989.
1196. NUREG/CR-5176, "Seismic Failure and Cask Drop Analyses of the Spent Fuel Pools at Two Representative Nuclear Power Plants," U.S. Nuclear Regulatory Commission, January 1989.
1197. NUREG/CR-5281, "Value/Impact Analyses of Accident Preventive and Mitigative Options for Spent Fuel Pools," U.S. Nuclear Regulatory Commission, March 1989.
1198. NUREG/CR-1353, "Regulatory Analysis for the Resolution of Generic Issue 82 'Beyond Design Basis Accidents in Spent Fuel Pools,'" U.S. Nuclear Regulatory Commission, April 1989.
1199. Memorandum for V. Stello from E. Beckjord, "Resolution of Generic Issue, 'Beyond Design Basis Accidents in Spent Fuel Pools,'" April 24, 1989. [9704100053]
1200. NUREG/CR-5197, "Evaluation of Generic Issue 115, 'Enhancement of the Reliability of Westinghouse Solid State Protection System,'" U.S. Nuclear Regulatory Commission, January 1989.

1201. NUREG-1341, "Regulatory Analysis for the Resolution of Generic Issue 115, 'Enhancement of the Reliability of the Westinghouse Solid State Protection System,'" U.S. Nuclear Regulatory Commission, May 1989.
1202. Memorandum for V. Stello from E. Beckjord, "Resolution of Generic Issue 115, 'Enhancement of the Reliability of Westinghouse Solid State Protection Systems,' NUREG-1341," April 17, 1989. [9608210072]
1203. Memorandum for V. Stello from T. Murley, "Plant-Specific Backfit for Improved Auxiliary Feedwater System Reliability at Arkansas Nuclear One, Unit 2 and Rancho Seco," January 31, 1989. [8902030163]
1204. Memorandum for V. Stello from T. Murley, "Final Resolution of Generic Issue (GI) 122.2, 'Initiating Feed and Bleed,'" April 26, 1989. [8905090075]
1205. NRC Letter to All Licensees of Operating Plants, Applicants for Operating Licenses, and Holders of Construction Permits, "Task Action Plan I.D.2 - Safety Parameter Display System - 10 CFR §50.54(f) - (Generic Letter No. 89-06)," April 12, 1989. [8904120042]
1206. NUREG-1342, "A Status Report Regarding Industry Implementation of Safety Parameter Display Systems," U.S. Nuclear Regulatory Commission, April 1989.
1207. Memorandum for V. Stello from T. Murley, "Final Resolution of Generic Issue 125.I.3, 'SPDS Availability,'" April 26, 1989. [8905050362]
1208. Memorandum for V. Stello from T. Murley, "Final Resolution of Generic Issue (GI) HF4.1, Inspection Procedure for Upgraded Emergency Operating Procedures," October 17, 1988. [8811070169]
1209. NUREG-1358, "Lessons Learned from the Special Inspection Program for Emergency Operating Procedures," U.S. Nuclear Regulatory Commission, April 1989.
1210. NRC Information Notice No. 86-64, "Deficiencies in Upgrade Programs for Plant Emergency Operating Procedures," August 14, 1986 [8608120028], (Supplement 1) April 20, 1987 [8704160062].
1211. NUREG/CR-5088, "Fire Risk Scoping Study: Investigation of Nuclear Power Plant Fire Risk, Including Previously Unaddressed Issues," U.S. Nuclear Regulatory Commission, January 1989.
1212. NUREG/CR-5112, "Evaluation of Boiling Water Reactor Water-Level Sensing Line Break and Single Failure," U.S. Nuclear Regulatory Commission, March 1989.
1213. NRC Letter to All Holders of Operating Licenses or Construction Permits for Boiling Water Reactors, "Resolution of Generic Issue 101, 'Boiling Water Reactor Water Level Redundancy' (Generic Letter 89-11)," June 30, 1989. [8906300178]
1214. Memorandum for V. Stello from E. Beckjord, "Closeout of GI 101, 'Boiling Water Reactor Water Level Redundancy,'" April 24, 1989. [9704100038]

1215. Regulatory Guide 1.106, "Thermal Overload Protection for Electric Motors on Motor-Operated Valves," U.S. Nuclear Regulatory Commission, November 1975, (Rev. 1) March 1977. [7907100392]
1216. NUREG-1296, "Thermal Overload Protection for Electric Motors on Safety-Related Motor-Operated Valves - Generic Issue II.E.6.1," U.S. Nuclear Regulatory Commission, June 1988.
1217. NRC Letter to All Licensees of Operating Power Plants and Holders of Construction Permits for Nuclear Power Plants, "Safety-Related Motor-Operated Valve Testing and Surveillance (Generic Letter No. 89-10) - 10 CFR 50.54(f)," June 28, 1989 [8906290082], (Supplement 1) June 13, 1990 [9201300217], (Supplement 2) August 3, 1990 [9007310052], (Supplement 3) October 25, 1990 [9010220146], (Supplement 4) February 12, 1992 [9202250311], (Supplement 5) June 28, 1993 [9306230099], (Supplement 6) March 8, 1994 [9402280155].
1218. Memorandum for V. Stello from E. Beckjord, "Close-out of Generic Issue II.E.6.1, 'In Situ Testing of Valves,'" June 30, 1989. [8907100275]
1219. Memorandum for F. Rowsome from D. Crutchfield, "Potential Generic Issue: Loss of Effective Volume for Containment Recirculation Spray," July 13, 1984. [8407240406]
1220. Memorandum for G. Lainas from R. Houston, "Task Interface Agreement (TIA) #83-144: Loss of Effective Volume for Containment Recirculation Spray for H.B. Robinson, Unit 2 (TAC #53223)," August 6, 1984. [8408130232]
1221. Memorandum for W. Minners from F. Rowsome, "Candidate Generic Safety Issue: Allowable Outage Times for Diverse, Simultaneous Equipment Outages," May 9, 1985. [8506030097]
1222. NRC Letter to All Licensees Holding Operating Licenses and Construction Permits for Nuclear Power Reactor Facilities, "Individual Plant Examination for Severe Accident Vulnerabilities - 10 CFR § 50.54(f), (Generic Letter No. 88-20)," November 23, 1988 [8811280048], (Supplement 1) August 29, 1989 [8908300001], (Supplement 2) April 4, 1990 [9003300127], (Supplement 3) July 6, 1990 [9007020114], (Supplement 4) June 28, 1991 [9106270324], (Supplement 5) September 8, 1995.
1223. Proceedings of the International Topical Meeting on Probability, Reliability, and Safety Assessment, PSA '89, p.48, "Potential Underestimation of Test and Maintenance Unavailabilities in Probabilistic Risk Assessments," American Nuclear Society, April 2-7, 1989.
1224. Memorandum for B. Morris from F. Gillespie, "Prioritization of GI-117, 'Allowable Outage Times for Diverse Simultaneous Equipment Outages,'" August 4, 1989. [9704100058]
1225. Federal Register Notice 46 FR 58484, "10 CFR Part 50, Interim Requirements Related to Hydrogen Control," December 2, 1981.
1226. Federal Register Notice 50 FR 3498, "10 CFR Part 50, Hydrogen Control Requirements," January 25, 1985.

1227. SECY-89-122, "Resolution of Unresolved Safety Issue (USI) A-48, 'Hydrogen Control Measures and Effects of Hydrogen Burns on Safety Equipment,'" April 19, 1989. [8905010149]
1228. NUREG-0943, "Threaded-Fastener Experience in Nuclear Power Plants," U.S. Nuclear Regulatory Commission, January 1983.
1229. EPRI NP-3784, "A Survey of the Literature on Low-Alloy Steel Fastener Corrosion in PWR Power Plants," Electric Power Research Institute, December 1984.
1230. EPRI RP 2520-7, "Degradation and Failure of Bolting in Nuclear Power Plants," Electric Power Research Institute, June 1987.
1231. EPRI NP-2174, "A Study of Bolting Problems, Tools, and Practices in the Nuclear Industry," Electric Power Research Institute, December 1981.
1232. NUREG-1174, "Evaluation of Systems Interactions in Nuclear Power Plants," U.S. Nuclear Regulatory Commission, May 1989.
1233. NUREG-1229, "Regulatory Analysis for Resolution for USI A-17," U.S. Nuclear Regulatory Commission, August 1989.
1234. SECY-89-230, "Unresolved Safety Issue A-17, 'Systems Interactions in Nuclear Power Plants,'" August 1, 1989. [8908140127]
1235. NRC Letter to All Holders of Operating Licenses or Construction Permits for Nuclear Power Plants, "Resolution of Unresolved Safety Issue A-17, 'Systems Interactions in Nuclear Power Plants' (Generic Letter 89-18)," September 6, 1989. [8909070029]
1236. Federal Register Notice 54 FR 34836, "Issuance and Availability of NUREG-1174, 'Evaluation of Systems Interactions in Nuclear Power Plants: Technical Findings Related to Unresolved Safety Issue A17,' and NUREG-1229, 'Regulatory Analysis for Resolution of USI A-17, - Systems Interactions in Nuclear Power Plants,'" August 22, 1989.
1237. NUREG/CR-5420, "Multiple System Responses Program - Identification of Concerns Related to A Number of Specific Regulatory Issues," U.S. Nuclear Regulatory Commission, October 1989.
1238. NUREG/CR-5437, "Recommendations for Resolution of Public Comments on USI A-40, 'Seismic Design Criteria,'" U.S. Nuclear Regulatory Commission, June 1989.
1239. IE Bulletin No. 79-02, "Pipe Support Base Plate Designs Using Concrete Expansion Anchor Bolts," U.S. Nuclear Regulatory Commission, March 8, 1979 [7903140038], (Rev. 1) June 20, 1979 [7906200183], (Rev. 2) November 8, 1979 [7908220136].
1240. IE Bulletin No. 79-14, "Seismic Analysis for As-Built Safety-Related Piping Systems," U.S. Nuclear Regulatory Commission, July 2, 1979 [7907060295], (Rev. 1) July 18, 1979 [7907250430].

1241. IE Bulletin No. 80-11, "Masonry Wall Design," U.S. Nuclear Regulatory Commission, May 8, 1980. [7912190695]
1242. NUREG/CR-1161, "Recommended Revisions to Nuclear Regulatory Commission Seismic Design Criteria," U.S. Nuclear Regulatory Commission, May 1980.
1243. NUREG/CR-3480, "Value/Impact Assessment for Seismic Design Criteria USI A-40," U.S. Nuclear Regulatory Commission, August 1984.
1244. NUREG-1233, "Regulatory Analysis for USI A-40, 'Seismic Design Criteria,'" U.S. Nuclear Regulatory Commission, September 1989.
1245. SECY-89-296, "Unresolved Safety Issue A-40, 'Seismic Design Criteria,'" September 22, 1989. [8910060116]
1246. Federal Register Notice 54 FR 40220, "Issuance and Availability Final Resolution of Unresolved Safety Issue (USI) A-40; Seismic Design Criteria," September 29, 1989.
1247. NUREG-1217, "Evaluation of Safety Implications of Control Systems in LWR Nuclear Power Plants," U.S. Nuclear Regulatory Commission, June 1989.
1248. NUREG-1218, "Regulatory Analysis for Resolution of USI A-47," U.S. Nuclear Regulatory Commission, July 1989.
1249. SECY-89-255, "Unresolved Safety Issue A-47, 'Safety Implications of Control Systems,'" August 23, 1989. [8908250318]
1250. NRC Letter to All Licensees of Operating Reactors, Applicants for Operating Licenses and Holders of Construction Permits for Light Water Reactor Nuclear Power Plants, "Request for Action Related to Resolution of Unresolved Safety Issue A-47, 'Safety Implication of Control Systems in LWR Nuclear Power Plants' Pursuant to 10 CFR 50.54(f) - Generic Letter 89-19," September 20, 1989. [8909200223]
1251. Federal Register Notice 54 FR 36922, "Issuance and Availability of NUREG-1217, 'Evaluation of Safety Implications of Control Systems in LWR Nuclear Power Plants - Technical Findings Related to USI A-47,' and NUREG-1218, 'Regulatory Analysis for Resolution of USI A-47,'" September 5, 1989.
1252. Memorandum for T. King from C. Serpan, "Reevaluation of Issue 15, 'Radiation Effects on Reactor Vessel Supports,'" September 30, 1988. [9704100071]
1253. ORNL/TM-10444, "Evaluation of HFIR Pressure-Vessel Integrity Considering Radiation Embrittlement," Oak Ridge National Laboratory, April 1988.
1254. NUREG/CR-5320, "Impact of Radiation Embrittlement on Integrity of Pressure Vessel Supports for Two PWR Plants," U.S. Nuclear Regulatory Commission, January 1989.
1255. UCLA-ENG-76113, "Some Probabilistic Aspects of the Seismic Risk of Nuclear Reactors," University of California, Los Angeles, December 1976.

1256. SECY-89-180, "Generic Safety Issue 15, 'Radiation Effects on Reactor Vessel Supports,'" June 13, 1989. [8906190110]
1257. NUREG/CR-5210, "Technical Findings Document for Generic Issue 51: Improving the Reliability of Open-Cycle Service-Water Systems," U.S. Nuclear Regulatory Commission, August 1988.
1258. NUREG/CR-5234, "Value/Impact Analysis for Generic Issue 51: Improving the Reliability of Open-Cycle Service-Water Systems," U.S. Nuclear Regulatory Commission, February 1989.
1259. NRC Letter to All Holders of Operating Licenses or Construction Permits for Nuclear Power Plants, "Service Water System Problems Affecting Safety-Related Equipment (Generic Letter 89-13)," July 18, 1989. [8907180211]
1260. Memorandum for J. Taylor from E. Beckjord, "Closeout of GI-51, 'Improving the Reliability of Open-Cycle Service Water Systems,'" August 10, 1989. [9704100044]
1261. Federal Register Notice 54 FR 31268, "'Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants'; Issuance and Availability Revised SRP Sections 2.4.2 and 2.4.3," July 27, 1989.
1262. NRC Letter to All Licensees of Operating Reactors and Holders of Construction Permits, "Potential for Increased Roof Loads and Plant Area Flood Runoff Depth at Licensed Nuclear Power Plants Due to Recent Change in Probable Maximum Precipitation Criteria Developed by the National Weather Service (Generic Letter 89-22)," October 19, 1989. [8910180273]
1263. Memorandum for J. Taylor from E. Beckjord, "Close-out of Generic Safety Issue No. 103, 'Design for Probable Maximum Precipitation,'" November 28, 1989. [8912180025]
1264. Memorandum for V. Stello from S. Chilk, "Degree Operators: Advance Notice of Rulemaking," January 23, 1986. [8601280245]
1265. Federal Register Notice 54 FR 33639, "Education for Senior Reactor Operators and Shift Supervisors at Nuclear Power Plants; Policy Statement," August 15, 1989.
1266. Federal Register Notice 54 FR 33568, "Education and Experience Requirements for Senior Reactor Operators and Supervisors at Nuclear Power Plants; Withdrawal of Proposed Rulemaking," August 15, 1989.
1267. NUREG-1267, "Technical Resolution of Generic Safety Issue A-29," U.S. Nuclear Regulatory Commission, September 1989.
1268. Memorandum for J. Taylor from E. Beckjord, "Resolution of Generic Safety Issue A-29, 'Nuclear Power Plant Design for Reduction of Vulnerability to Industrial Sabotage,'" October 6, 1989. [8910190129]
1269. NUREG/CR-3453, "Electronic Isolators Used in Safety Systems of U.S. Nuclear Power Plants," U.S. Nuclear Regulatory Commission, March 1986.

1270. Memorandum for B. Morris from B. Sheron, "Proposed Generic Issue on Leakage Through Electrical Isolators," June 23, 1987. [9704100047]
1271. Memorandum for T. Speis from R. Bernero, "Request for Prioritization of Potential Generic Issue Per Office Letter No. 40," August 4, 1985. [8508120299]
1272. Memorandum for R. Mattson from F. Rosa, "Combustion Engineering Standard Technical Specifications (NUREG-0212) - Proposed Revision 3 - Relay Testing," October 8, 1982. [8211030387]
1273. NUREG-0693, "Analysis of Ultimate Heat Sink Cooling Ponds," U.S. Nuclear Regulatory Commission, November 1980.
1274. NUREG-0733, "Analysis of Ultimate Heat-Sink Spray Ponds," U.S. Nuclear Regulatory Commission, August 1981.
1275. NUREG-0858, "Comparison Between Field Data and Ultimate Heat Sink Cooling Pond and Spray Pond Models," U.S. Nuclear Regulatory Commission, September 1982.
1276. Regulatory Guide 1.27, "Ultimate Heat Sink for Nuclear Power Plants," U.S. Nuclear Regulatory Commission, (Rev. 2) January 1976.
1277. NUREG/CR-4120, "Mathematical Modeling of Ultimate Heat Sink Cooling Ponds," U.S. Nuclear Regulatory Commission, March 1985.
1278. "Performance Model for Ultimate Heat Spray Ponds," Journal of Energy Engineering, Vol. 112, No. 2, August 1986.
1279. "Method for Analysis of Ultimate Heat Sink Cooling Tower Performance," University of Illinois at Urbana-Champaign, April 1986. [9704090166]
1280. Memorandum for C. Ader from K. Kniel, "Request for Prioritization of New Generic Safety Issue 'Loss of Essential Service Water in LWRs,'" May 2, 1990. [9704090120]
1281. Memorandum for W. Minners from F. Rowsome, "A New Generic Safety Issue: Accident Management," April 16, 1985. [8505080417]
1282. SECY-88-147, "Integration Plan for Closure of Severe Accident Issues," May 25, 1988. [8806030338]
1283. Memorandum for C. Ader from W. Minners, "GI 116, Accident Management," May 9, 1990. [9704090138]
1284. Memorandum for J. Olshinski from D. Eisenhut, "Control Rod Guide Tube Pin Failures and Peening Damage on Integrity of Steam Generator Tube to Tubesheet Welds and Tube Ends -- North Anna Power Station, Unit No. 1 (NA-1)," December 13, 1982. [8212270164]
1285. EPRI NP-5544, "Nuclear Unit Operating Experience: 1985-1986 Update," Electric Power Research Institute, December 1987.

1286. Memorandum for M. Virgilio from S. Newberry, "Proposed Research Programs to Support SICB Regulation Needs," April 26, 1990. [9005090104]
1287. Memorandum for F. Gillespie, et. al., from T. Speis, "CRGR Combined Packages for the Proposed Resolution of Generic Issue 70, 'Power Operated Relief Valve and Block Valve Reliability,' and Generic Issue 94, 'Additional Low-Temperature Overpressure Protection for Light-Water Reactors,'" December 7, 1988. [9507280258]
1288. SECY-90-232, "Evaluation of the Need for Primary System High Capacity Manual Venting Capability on Combustion Engineering (CE) Plants Without PORVs (GI-84)," June 28, 1990. [9007020274]
1289. Letter to K. Carr from C. Michelson, "Generic Issue - 84, Combustion Engineering Plants Without Power Operated Relief Valves," June 12, 1990. [9006220172]
1290. NRC Letter to All Pressurized Water Reactor Licensees and Construction Permit Holders, "Resolution of Generic Issue 70, 'Power-Operated Relief Valve and Block Valve Reliability,' and Generic Issue 94, 'Additional Low-Temperature Overpressure Protection for Light-Water Reactors,' Pursuant to 10 CFR 50.54(f) (Generic Letter 90-06)," June 25, 1990. [9006200120]
1291. NUREG-1326, "Regulatory Analysis for the Resolution of Generic Issue 94, 'Additional Low-Temperature Overpressure Protection for Light-Water Reactors,'" U.S. Nuclear Regulatory Commission, December 1989.
1292. Memorandum for J. Taylor from E. Beckjord, "Close-out of Generic Issue 70, 'Power-Operated Relief Valve and Block Valve Reliability,' and Generic Issue 94, 'Additional Low-Temperature Overpressure Protection for Light-Water Reactors,'" July 26, 1990. [9507280267]
1293. NUREG-1316, "Technical Findings and Regulatory Analysis Related to Generic Issue 70," U.S. Nuclear Regulatory Commission, December 1989.
1294. Memorandum for F. Gillespie from E. Beckjord, "Resolutions of Generic Issue 70, 'Power Operated Relief Valve and Block Valve Reliability,' and Generic Issue 94, 'Additional Low-Temperature Overpressure Protection for Light-Water Reactors,'" November 16, 1989. [8911290064]
1295. SECY-90-153, "Staff Conclusions Relative to the Classification of PORVs as Safety Grade," April 27, 1990. [9005030123]
1296. Information Notice No. 90-19, "Potential Loss of Effective Volume for Containment Recirculation Spray at PWR Facilities," U.S. Nuclear Regulatory Commission, March 14, 1990. [9003080213]
1297. IE Bulletin No. 83-01, "Failure of Reactor Trip Breakers (Westinghouse DB-50) to Open on Automatic Trip Signal," U.S. Nuclear Regulatory Commission, February 25, 1983. [8212060367]

1298. IE Bulletin No. 83-04, "Failure of the Undervoltage Trip Function of Reactor Trip Breakers," U.S. Nuclear Regulatory Commission, March 11, 1983. [8212060380]
1299. IE Bulletin No. 83-08, "Electrical Circuit Breakers With an Undervoltage Trip Feature in Use in Safety-Related Applications Other than the Reactor Trip System," U.S. Nuclear Regulatory Commission, December 28, 1983. [8312120090]
1300. Memorandum for J. Taylor from E. Beckjord, "Resolution of Generic Safety Issue 75, 'Generic Implications of Salem ATWS - QA,'" May 18, 1990. [9507280290]
1301. Memorandum for W. Dircks from R. Minogue, "Preliminary Survey of Requirements and Guidance by Functional Areas for Operating Nuclear Power Plants, Dated February 1984," August 21, 1984. [8409110305]
1302. Memorandum for E. Beckjord from W. Minners, "Generic Issue 131, 'Potential Seismic Interaction Involving the Movable In-Core Flux Mapping System Used in Westinghouse Plants,'" July 18, 1990. [9007240192]
1303. NRC Letter to All Power Reactor Licensees and Applicants, "Relaxation of Staff Position in Generic Letter 83-28, Item 2.2 Part 2, 'Vendor Interface for Safety-Related Components' (Generic Letter No. 90-03)," March 20, 1990. [9003140089]
1304. IE Bulletin No. 77-02, "Potential Failure Mechanism in Certain Westinghouse (W) AR Relays With Latch Attachments," U.S. Nuclear Regulatory Commission, September 12, 1977. [7909050215]
1305. IE Bulletin No. 79-09, "Failures of GE Type AK-2 Circuit Breaker in Safety Related Systems," U.S. Nuclear Regulatory Commission, April 17, 1979. [7905010083]
1306. IE Circular No. 81-12, "Inadequate Periodic Test Procedure of PWR Protection System," U.S. Nuclear Regulatory Commission, July 22, 1981. [8103300406]
1307. NUREG-1372, "Regulatory Analysis for the Resolution of Generic Issue C-8: 'Main Steam Isolation Valve Leakage and LCS Failure,'" U.S. Nuclear Regulatory Commission, June 1990.
1308. NUREG-1169, "Resolution of Generic Issue C-8," U.S. Nuclear Regulatory Commission, August 1986.
1309. NUREG/CR-5397, "Value-Impact Analysis of Regulatory Options for Resolution of Generic Issue C-8: MSIV Leakage and LCS Failure," U.S. Nuclear Regulatory Commission, May 1990.
1310. Memorandum for J. Taylor from E. Beckjord, "Resolution of Generic Safety Issue C-8," March 15, 1990. [9507280304]
1311. IE Information Notice No. 82-09, "Cracking in Piping of Makeup Coolant Lines at B&W Plants," U.S. Nuclear Regulatory Commission, March 31, 1982. [8202040131]

1312. IE Information No. 82-30, "Loss of Thermal Sleeves in Reactor Coolant System Piping at Certain Westinghouse PWR Power Plants," U.S. Nuclear Regulatory Commission, July 26, 1982. [8204210403]
1313. SECY-82-186, "Status of Make-up Nozzle Cracking in Babcock & Wilcox (B&W) Plants," May 7, 1982. [8205280495]
1314. Letter to R. Gridley (General Electric Company) from D. Eisenhut, "Safety Evaluation for the General Electric Topical Report NEDE-218121-02, 'BWR Feedwater Nozzle/Sparger Final Report, Supplement 2,'" January 14, 1980. [8002070141]
1315. Memorandum for D. Sternberg from R. Clark, "Degradation of Thermal Sleeves - Trojan Nuclear Plant," August 11, 1982. [8208250478]
1316. Memorandum for T. Novak, et al., from J. Knight, "Evaluation of Thermal Sleeve Problems in Westinghouse Plants," October 28, 1983. [8311140192]
1317. Letter to V. Stello from W. Kerr, "ACRS Comments on Nuclear Power Plant Air Cooling Systems," October 15, 1987. [8710210001]
1318. NUREG/CR-4550, "Analysis of Core Damage Frequency from Internal Events," U.S. Nuclear Regulatory Commission, (Vol. 1, Rev. 1) January 1990, (Vol. 2) April 1989, (Vol. 3, Rev. 1) April 1990, (Vol. 4, Rev. 1) August 1989, (Vol. 5, Rev. 1) April 1990, (Vol. 6) April 1987, (Vol. 7, Rev. 1) May 1990.
1319. NRC Information Notice 92-18, "Potential for Loss of Remote Shutdown Capability During a Control Room Fire," U.S. Nuclear Regulatory Commission, February 28, 1992. [9202240025]
1320. SECY-89-170, "Fire Risk Scoping Study: Summary of Results and Proposed Staff Actions," June 7, 1989. [8906260024]
1321. NRC Information Notice 91-53, "Failure of Remote Shutdown System Instrumentation Because of Incorrectly Installed Components," U.S. Nuclear Regulatory Commission, September 4, 1991. [9108280089]
1322. IE Information Notice 87-12, "Potential Problems with Metal Clad Circuit Breakers, General Electric Type AKF-2-25," U.S. Nuclear Regulatory Commission, February 13, 1987. [8702110132]
1323. Memorandum for W. Minners, et al., from F. Rowsome, "Generic Issue 123, 'Deficiencies in the Regulations Suggested by the Davis-Besse Incident,'" November 21, 1985. [8512100189]
1324. ANSI/IEEE-ANS-7-4.3.2-1982, "Application Criteria for Programmable Digital Computer Systems in Safety Systems of Nuclear Power Generating Stations," American Nuclear Society, July 6, 1982.

1325. Regulatory Guide 1.152, "Criteria for Programmable Digital Computer System Software in Safety-Related Systems of Nuclear Power Plants," U.S. Nuclear Regulatory Commission, November 1985. [8511220286]
1326. NUREG-1289, "Regulatory and Backfit Analysis: Unresolved Safety Issue A-45, Shutdown Decay Heat Removal Requirements," U.S. Nuclear Regulatory Commission, November 1988.
1327. NUREG/CR-4639, "Nuclear Computerized Library for Assessing Reactor Reliability (NUCLARR)," U.S. Nuclear Regulatory Commission, (Vol. 1) February 1988, (Vol. 2) September 1988, (Vol. 3) November 1988, (Vol. 4) June 1988, (Vol. 5) June 1988.
1328. AEOD/E804, "Reliability of Non-Safety Related Field Breakers During ATWS Events," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, July 26, 1988. [8905020208]
1329. Memorandum for T. King from K. Kniel, "Request for Prioritization of New Generic Safety Issue 'Reliability of Recirculation Pump Trip (RPT) During an ATWS,'" March 17, 1989. [9507280112]
1330. Memorandum for T. King from W. Minners, "Overpressurization of Containment Penetrations," March 16, 1989. [9507280122]
1331. NUREG/CR-4220, "Reliability Analysis of Containment Isolation Systems," U.S. Nuclear Regulatory Commission, June 1985.
1332. NUREG-0797, "Safety Evaluation Report Related to the Operation of Comanche Peak Steam Electric Station, Units 1 and 2," U.S. Nuclear Regulatory Commission, (Supplement 9) March 1985.
1333. NSAC-148, "Service Water Systems and Nuclear Plant Safety," Electric Power Research Institute, May 1990.
1334. NUREG/CR-2797, "Evaluation of Events Involving Service Water Systems in Nuclear Power Plants," U.S. Nuclear Regulatory Commission, November 1982.
1335. Memorandum for W. Minners from F. Rowsome, "A Candidate Generic Safety Issue," December 11, 1984. [8501090105]
1336. Memorandum for B. Morris from L. Shao, "Resolution of Generic Issue 119.2," July 16, 1990. [9507280130]
1337. Memorandum for J. Taylor from E. Beckjord, "Proposed Resolution and Closeout of Generic Issue 135, 'Steam Generator and Steam Line Overfill Issues,'" March 29, 1991. [9507280149]
1338. RES Office Letter No. 7, "Procedures for Identification, Prioritization, Resolution, and Tracking of Generic Issues," February 16, 1996. [9608070117]

1339. Memorandum for All RES Employees from E. Beckjord, "Withdrawal of RES Office Letter No. 3, 'Procedure and Guidance for the Resolution of Generic Issues,'" June 2, 1994. [9704100042]
1340. Federal Register Notice 51 FR 12502, "10 CFR Part 50, Modification of General Design Criterion 4 Requirements for Protection Against Dynamic Effects of Postulated Pipe Ruptures," April 11, 1986.
1341. Federal Register Notice 51 FR 26393, "10 CFR Part 50, Modification of General Design Criterion 4 Requirements for Protection Against Dynamic Effects of Postulated Pipe Ruptures," July 23, 1986.
1342. Federal Register Notice 52 FR 41288, "10 CFR Part 50, Modification of General Design Criterion 4 Requirements for Protection Against Dynamic Effects of Postulated Pipe Ruptures," October 27, 1987.
1343. Federal Register Notice 53 FR 1968, "Standard Review Plan Revision," January 25, 1988.
1344. Federal Register Notice 52 FR 23376, "Standard Review Plan Issuance," June 19, 1987.
1345. NRC Letter to All Operating Licensees, Construction Permit Holders and Applicants for Construction Permits, "Relaxation in Arbitrary Intermediate Pipe Rupture Requirements (Generic Letter 87-11)," June 19, 1987. [8706230486]
1346. Memorandum for Distribution from G. Arlotto, "Termination of Proposed Revision to SRP 3.9.3," October 2, 1986. [8811180136]
1347. Regulatory Guide 1.84, "Design and Fabrication Code Case Acceptability -- ASME III, Division 1," U.S. Nuclear Regulatory Commission, (Rev. 30) October 31, 1994. [9411040236]
1348. Regulatory Guide 1.61, "Damping Values for Seismic Design of Nuclear Power Plants," U.S. Nuclear Regulatory Commission, October 1973. [7907100231]
1349. Memorandum for J. Roe from J. Wermiel, "Closure of Generic Issue No. 133, 'Update Policy on Nuclear Plant Staff Working Hours,'" July 10, 1991. [9107230263]
1350. NRC Information Notice No. 91-36, "Nuclear Plant Staff Working Hours," June 10, 1991. [9106040339]
1351. SECY-90-343, "Status of the Staff Program to Determine How the Lessons Learned from the Systematic Evaluation Program Have Been Factored into the Licensing Bases of Operating Plants," October 4, 1990. [9010150030]
1352. Memorandum for J. Knight, et al., from H. Thompson, "Action Plan for Resolving Failure of Tendon Anchorage at Farley 2 and for Determining Need for Immediate Licensing Action on Other Facilities," June 25, 1985. [8507030479]

1353. NUREG/CR-4712, "Regulatory Analysis of Regulatory Guide 1.35 (Revision 3, Draft 2) - In-Service Inspection of UngROUTed Tendons in Prestressed Concrete Containments," U.S. Nuclear Regulatory Commission, February 1987.
1354. NUREG-1407, "Procedural and Submittal Guidance for the Individual Plant Examination of External Events (IPEEE) for Severe Accident Vulnerabilities," U.S. Nuclear Regulatory Commission, June 1991.
1355. NRC Letter to All Licensees of Operating Pressurized Water Nuclear Power Reactors and Applicants for Operating Licenses (Except for St. Lucie, Unit No. 1), "Natural Circulation Cooldown (Generic Letter No. 81-21)," May 5, 1981. [8105140267]
1356. NRC Letter to All Operating Pressurized Water Reactors (PWR's) (Generic Letter 80-53), June 11, 1980. [8007230099]
1357. ANSI/ANS 2.3, "Estimating Tornado and Extreme Wind Characteristics at Nuclear Power Sites," American National Standards Institute, Inc., October 17, 1983.
1358. Memorandum for H. Thompson from G. Arlotto, "RES Input - Action Plan for Resolving Failure of Tendon Anchorage at Farley-2 and for Determining Need for Immediate Licensing Action on Other Facilities," July 31, 1985. [9312220342]
1359. Memorandum for T. Speis and E. Jordan from J. Knight, "Tendon Anchor Head Failure - Needed Licensing Action at Other Facilities," December 6, 1985. [8512240278]
1360. Regulatory Guide 1.35.1, "Determining Prestressing Forces for Inspection of Prestressed Concrete Containments," U.S. Nuclear Regulatory Commission, July 1990. [9503290310]
1361. Memorandum for E. Jordan from E. Beckjord, "CRGR Review of: 1. Regulatory Guide 1.35, Rev. 3, 'Inservice Inspection of UngROUTed Tendons in Prestressed Concrete Containments,' 2. Regulatory Guide 1.35.1, 'Determining Prestressing Forces for Inspection of Prestressed Concrete Containments,'" July 28, 1989. [8908100273]
1362. Memorandum for E. Beckjord from F. Gillespie, "Generic Concerns Arising from TMI-2 Cleanup," February 21, 1991. [9103010101]
1363. Memorandum for E. Beckjord from F. Gillespie, "Request for Generic Rulemaking Concerning Decommissioning Issues," January 7, 1992. [9201150209]
1364. Federal Register Notice 53 FR 24018, "10 CFR Parts 30, 40, 50, 51, 70, and 72, General Requirements for Decommissioning Nuclear Facilities," June 27, 1988.
1365. Memorandum for Z. Rosztoczy from S. Bajwa, "Generic Issue 148: Smoke Control and Manual Fire Fighting Effectiveness; Generic Issue 149: Adequacy of Fire Barriers," April 3, 1991. [9104080111]
1366. NUREG-1286, "Safety Evaluation Report Related to the Restart of Rancho Seco Nuclear Generating Station, Unit 1 Following the Event of December 26, 1985," U.S. Nuclear Regulatory Commission, October 1987.

1367. Memorandum for W. Russell from A. Thadani, "Task Action Plan for Resolution of Service Water System Problems," June 27, 1991. [9107120290]
1368. NRC Letter to Licensees and Applicants of the Following Pressurized-Water Reactor Nuclear Power Plants: 1. Braidwood Units 1 and 2; 2. Byron Units 1 and 2; 3. Catawba Units 1 and 2; 4. Comanche Peak Units 1 and 2; 5. Cook Units 1 and 2; 6. Diablo Canyon Units 1 and 2; 7. McGuire Units 1 and 2, "Request for Information Related to the Resolution of Generic Issue 130, 'Essential Service Water System Failures at Multi-Unit Sites,' Pursuant to 10 CFR 50.54(f) - Generic Letter 91-13," September 19, 1991. [9109160253]
1369. NUREG-1269, "Loss of Residual Heat Removal System, Diablo Canyon Unit 2, April 10, 1987," U.S. Nuclear Regulatory Commission, June 1987.
1370. SECY-91-283, "Evaluation of Shutdown and Low Power Risk Issues," September 9, 1991. [9109120134]
1371. NUREG/CR-4960, "Control Room Habitability Survey of Licensed Commercial Nuclear Power Generating Stations," U.S. Nuclear Regulatory Commission, October 1988.
1372. Regulatory Guide 4.7, "General Site Suitability Criteria for Nuclear Power Stations," U.S. Nuclear Regulatory Commission, September 1974, (Rev. 1) November 1975. [7907200072]
1373. Regulatory Guide 1.78, "Assumptions for Evaluating the Habitability of a Nuclear Power Plant Control Room During a Postulated Hazardous Chemical Release," U.S. Nuclear Regulatory Commission, June 1974. [8001240567]
1374. Regulatory Guide 1.91, "Evaluations of Explosions Postulated to Occur on Transportation Routes Near Nuclear Power Plants," U. S. Nuclear Regulatory Commission, January 1975, (Rev. 1) February 1978 [8808230010].
1375. Regulatory Guide 1.95, "Protection of Nuclear Power Plant Control Room Operators Against an Accidental Chlorine Release," U.S. Nuclear Regulatory Commission, February 1975, (Rev. 1) January 1977 [8001240569].
1376. Letter to D. Solberg (NRC) from T. Charlton (INEL), "Transmittal of Letter Report on Turbine Trip Failure Events - TRC-28-88," April 27, 1988. [9502070128]
1377. Memorandum for R. Baer from C. Hrabal, "Prioritization of GI-144, 'SCRAM Without a Turbine/Generator Trip,'" September 24, 1991. [9312220339]
1378. Memorandum for T. King from K. Kniel, "Request for Prioritization of New Generic Safety Issue 'SCRAM Without a Turbine/Generator Trip,'" March 22, 1988. [9312220315]
1379. NUREG/CR-5653, "Recriticality in a BWR Following a Core Damage Event," U.S. Nuclear Regulatory Commission, November 1990.
1380. Memorandum for W. Minners from B. Sheron, "Request for Prioritization of Potential Generic Issues," September 4, 1984. [8409170085]

1381. Memorandum for W. Minners from B. Sheron, "Update of Generic Issue Management Control System (GIMCS)," July 5, 1991. [9312220300]
1382. NUREG-1365, "Revised Severe Accident Research Program Plan," U.S. Nuclear Regulatory Commission, August 1989, (Rev. 1) December 1992.
1383. Memorandum for R. Baer from S. Diab, "Supporting Analyses for Prioritization of Issue 110, 'Equipment Protective Devices on Engineered Safety Features,'" April 16, 1992. [9312220226]
1384. Memorandum for W. Dircks for R. Fraley, "Bolt Failures in Nuclear Power Plants," October 20, 1981. [8201200698]
1385. NRC Letter to All Holders of Operating Licenses or Construction Permits for Nuclear Power Plants, "Generic Safety Issue 29, 'Bolting Degradation or Failure in Nuclear Power Plants,' (Generic Letter 91-17)," October 17, 1991. [9110150302]
1386. NRC Letter to All Licensees of Operating PWRs and Holders of Construction Permits for PWRs, "Boric Acid Corrosion of Carbon Steel Reactor Pressure Boundary Components in PWR Plants (Generic Letter 88-05)," March 17, 1988. [8803220364]
1387. NRC Letter to All Licensees, Applicants and Holders of Operating Licenses Not Required to be Reviewed for Seismic Adequacy of Equipment Under the Provisions of USI A-46, 'Seismic Qualification of Equipment in Operating Plants,' "Verification of Seismic Adequacy of Mechanical and Electrical Equipment in Operating Reactors, Unresolved Safety Issue (USI) A-46 (Generic Letter 87-03)," February 27, 1987. [8703060307]
1388. NRC Bulletin No. 89-02, "Stress Corrosion Cracking of High-Hardness Type 410 Stainless Steel Internal Preloaded Bolting in Anchor Darling Model S350W Swing Check Valves or Valves of Similar Design," U.S. Nuclear Regulatory Commission, July 19, 1989. [8907110441]
1389. NRC Compliance Bulletin No. 87-02, "Fastener Testing to Determine Conformance with Applicable Material Specifications," U.S. Nuclear Regulatory Commission, November 6, 1987 [8711050040], (Supplement 1) April 22, 1988 [8804180142], (Supplement 2) June 10, 1988 [8806090301].
1390. NRC Information Notice No. 89-22, "Questionable Certification of Fasteners," U.S. Nuclear Regulatory Commission, March 3, 1989. [8902270158]
1391. NRC Information Notice No. 89-56, "Questionable Certification of Material Supplied to the Defense Department by Nuclear Suppliers," U.S. Nuclear Regulatory Commission, July 20, 1989 [8907140274], (Supplement 1) November 22, 1989 [8911160058], (Supplement 2) July 19, 1991 [9107120259].
1392. NRC Information Notice No. 89-70, "Possible Indications of Misrepresented Vendor Products," U.S. Nuclear Regulatory Commission, October 11, 1989 [8910040381], (Supplement 1) April 26, 1990 [9004200525].

1393. IE Information Notice No. 86-25, "Traceability and Material Control of Material and Equipment, Particularly Fasteners," U.S. Nuclear Regulatory Commission, April 11, 1986. [8604090451]
1394. Memorandum for J. Taylor from E. Beckjord, "Resolution of Generic Safety Issue 29, 'Bolting Degradation or Failure in Nuclear Power Plants,'" October 25, 1991. [9312220296]
1395. NUREG-1339, "Resolution of Generic Safety Issue 29: Bolting Degradation or Failure in Nuclear Power Plants," U.S. Nuclear Regulatory Commission, June 1990.
1396. Federal Register Notice 56 FR 36081, "10 CFR Parts 21 and 50, Criteria and Procedures for the Reporting of Defects and Conditions of Construction Permits," July 31, 1991.
1397. SECY-91-150, "Proposed Amendments to 10 CFR Part 21, 'Reporting of Defects and Noncompliance' and 10 CFR 50.55(e), 'Conditions of Construction Permits,'" May 22, 1991. [9106040262]
1398. NUREG-1445, "Regulatory Analysis for the Resolution of Generic Safety Issue-29: Bolting Degradation or Failure in Nuclear Power Plants," U.S. Nuclear Regulatory Commission, September 1991.
1399. NRC Letter to All Holders of Operating Licenses, "Resolution of Generic Issue A-30, 'Adequacy of Safety-Related DC Power Supplies,' Pursuant to 10 CFR 50.54(f) (Generic Letter 91-06)," April 29, 1991. [9104170256]
1400. NRC Letter to All Holders of Operating Licenses, "Resolution of Generic Issues 48, 'LCOs for Class 1E Vital Instrument Buses,' and 49, 'Interlocks and LCOs for Class 1E Tie Breakers' Pursuant to 10 CFR 50.54(f) (Generic Letter 91-11)," July 18, 1991. [9107160296]
1401. NUREG/CR-5414, "Technical Findings for Proposed Integrated Resolution of Generic Issue 128, Electric Power Reliability," U.S. Nuclear Regulatory Commission, November 1989.
1402. Memorandum for J. Taylor from E. Beckjord, "Resolution of GI-128, Electrical Power Reliability," September 12, 1991. [9312220229]
1403. NUREG/CR-5406, "BWR Reactor Water Cleanup System Flexible Wedge Gate Isolation Valve Qualification and High Energy Flow Interruption Test," U.S. Nuclear Regulatory Commission, (Vol. 1) October 1989, (Vol. 2) October 1989, (Vol. 3) October 1989.
1404. NUREG/CR-5558, "Generic Issue 87: Flexible Wedge Gate Valve Test Program," U.S. Nuclear Regulatory Commission, January 1991.
1405. SECY-82-1B, "Proposed Commission Policy Statement on Severe Accidents and Related Views on Nuclear Reactor Regulation," November 24, 1982. [8301120513]
1406. Memorandum for J. Taylor from E. Beckjord, "Technical Resolution of Generic Issue 87, 'Failure of HPCI Steam Line Without Isolation,'" December 9, 1991. [9312220344]

1407. NUREG/CR-4681, "Enclosure Environment Characterization Testing for the Base Line Validation of Computer Fire Simulation Codes," U.S. Nuclear Regulatory Commission, March 1987.
1408. NUREG/CR-5526, "Analysis of Risk Reduction Measures Applied to Shared Essential Service Water Systems at Multi-Unit Sites," U.S. Nuclear Regulatory Commission, June 1991.
1409. NUREG-1421, "Regulatory Analysis for the Resolution of Generic Issue 130: Essential Service Water System Failures at Multi-Unit Sites," U.S. Nuclear Regulatory Commission, June 1991.
1410. Memorandum for J. Taylor from E. Beckjord, "Resolution of Generic Issue 130, 'Essential Service Water System Failures at Multi-Unit Sites,'" September 23, 1991. [9312220347]
1411. NUREG/CR-4893, "Technical Findings Report for Generic Issue 135, Steam Generator and Steam Line Overfill Issues," U.S. Nuclear Regulatory Commission, May 1991.
1412. Memorandum for J. Taylor, et al., from S. Chilk "SECY-91-132 - Evaluation of the Feasibility of Initiating a Consensus Process to Address Issues Related to the Below Regulatory Concern Policy," June 28, 1991. [9109060094]
1413. SECY-92-045, "Enhanced Participatory Rulemaking Process," February 7, 1992. [9202130092]
1414. Memorandum for K. Kniel from G. Lainas, "Proposed Generic Issue Deinitiating Upon Discovery of Reactor Coolant System Leakage," August 1, 1986. [8608110015]
1415. Letter to D. Basdekas (NRC) from J. Lambricht (SNL), "Generic Issue 148, 'Smoke Control and Manual Fire Fighting Effectiveness,'" March 4, 1992. [9502070165]
1416. Memorandum for B. Morris from W. Minners, "Prioritization of Proposed New Generic Issue," December 4, 1989. [9312220350]
1417. NUREG/CR-5856 "Identification and Evaluation of PWR In-Vessel Severe Accident Management Strategies," U.S. Nuclear Regulatory Commission, March 1992.
1418. Memorandum for T. Murley from E. Beckjord, "A New Generic Issue: Multiple Steam Generator Tube Leakage," June 16, 1992. [9212040356]
1419. Memorandum for C. Serpan from J. Muscara, "Steam Generator Tube Inspection, Integrity and Plugging Issues," March 16, 1992. [9212040327]
1420. Letter to J. Cross (Portland General Electric Company) from L. Kokajko (NRC), "Issuance of Amendment for Trojan Nuclear Plant (TAC No. M82287)," February 5, 1992. [9202130137]
1421. Letter to the NRC from J. Cross (Portland General Electric Company), "Request for Additional Information Regarding Trojan Steam Generator Tube Structural Integrity Report

and License Change Application (LCA) 219 Dated January 3, 1992 (TAC No. M82287),  
January 16, 1992. [9201220023]

1422. NUREG/CR-0718, "Steam Generator Tube Integrity Program Phase I Report," U.S. Nuclear Regulatory Commission, September 1979.
1423. NUREG-1350, "Nuclear Regulatory Commission Information Digest," U.S. Nuclear Regulatory Commission, (Vol. 4) March 1992, (Vol. 7) March 1995.
1424. EGG-PE-6670, "Generic Cost Analysis for Steam Generator Repairs and Replacement," Idaho National Engineering Laboratory, August 1984.
1425. SECY-91-270, "Interim Guidance on Staff Implementation of the Commission's Safety Goal Policy," August 27, 1991. [9109030213]
1426. Memorandum for R. Emrit from G. Burdick, "Multiple Steam Generator Tube Leakage," October 30, 1992. [9502070227]
1427. SECY-92-292, "Advance Notice of Proposed Rulemaking on Severe Accident Plant Performance Criteria for Future LWRs," August 21, 1992. [9208250010]
1428. NUREG/CR-4470, "Survey and Evaluation of Vital Instrumentation and Control Power Supply Events," U.S. Nuclear Regulatory Commission, August 1986.
1429. NUREG-1455, "Transformer Failure and Common-Mode Loss of Instrument Power at Nine Mile Point Unit 2 on August 13, 1991," U.S. Nuclear Regulatory Commission, October 1991.
1430. Memorandum for T. Martin et al., from T. Murley, "Preliminary Results from Individual Plant Examinations (IPE)," April 22, 1991. [9105020194]
1431. NRC Letter to All Holders of Operating Licenses or Construction Permits for Pressurized Water Reactors (PWRs), "Resolution of Generic Issue 79, 'Unanalyzed Reactor Vessel (PWR) Thermal Stress During Natural Convection Cooldown' (Generic Letter 92-02)," March 6, 1992. [9203030209]
1432. Memorandum for J. Taylor from E. Beckjord, "Resolution of Generic Issue 79, 'Unanalyzed Reactor Vessel (PWR) Thermal Stress During Natural Convection Cooldown,'" May 4, 1992. [9312220157]
1433. Memorandum for C. Heltemes from F. Gillespie, "Generic Issue 163, 'Multiple Steam Generator Tube Leakage,'" November 24, 1992. [9212040320]
1434. Memorandum for E. Beckjord from L. Shao, "Interim Plugging Criteria for Trojan Nuclear Plant," December 9, 1992. [9212140066]
1435. Memorandum for F. Gillespie from C. Heltemes, "GI-163, 'Multiple Steam Generator Tube Leakage,'" September 28, 1992. [9212040379]
1436. Federal Register Notice 51 FR 27817, "10 CFR Parts 50 and 73, Miscellaneous Amendments Concerning Physical Protection of Nuclear Power Plants," August 4, 1986.

1437. NRC Letter to All Power Reactor Licensees, "Implementation of 10 CFR 73.55 Miscellaneous Amendments and Search Requirements (Generic Letter 87-08)," May 11, 1987. [8705110372]
1438. Regulatory Guide 5.65, "Vital Area Access Controls, Protection of Physical Security Equipment, and Key and Lock Controls," U.S. Nuclear Regulatory Commission, September 1986. [8610030129]
1439. Memorandum for J. Taylor from E. Beckjord, "Resolution of Generic Issue 151 'Reliability of ATWS Recirculation Pump Trip in BWRs,'" September 29, 1992. [9312220159]
1440. NUREG/CR-2336, "Steam Generator Tube Integrity Program," U.S. Nuclear Regulatory Commission, August 1988.
1441. Memorandum for T. Murley from E. Beckjord, "Interim Plugging Criteria for Trojan Nuclear Plant," January 5, 1993. [9301110331]
1442. Memorandum for T. Murley from E. Beckjord, "Interim Plugging Criteria for Trojan Nuclear Plant," January 15, 1993. [9301250251]
1443. SECY-90-160, "Proposed Rule on Nuclear Power Plant License Renewal," May 3, 1990. [9005080305]
1444. NUREG-1412, "Foundation for the Adequacy of the Licensing Bases," U.S. Nuclear Regulatory Commission, December 1991.
1445. NUREG/CR-6010, "History and Current Status of Generation 3 Thermal Sleeves in Westinghouse Nuclear Power Plants," U.S. Nuclear Regulatory Commission, July 1992.
1446. Memorandum for J. Taylor from E. Beckjord, "Technical Resolution of Generic Issue 73, 'Detached Thermal Sleeves,'" September 2, 1992. [9210070203]
1447. Memorandum for D. Eisenhut, et al., from R. Vollmer, "Evaluation of Allegations Regarding Class 1 Piping Design Deficiencies (TAC #49242)," September 1, 1983. [8309210477]
1448. IE Information Notice No. 83-80, "Use of Specialized 'Stiff' Pipe Clamps," November 23, 1983. [8311010020]
1449. NUREG/CR-2405, "Subsystem Fragility - Seismic Safety Margins Research Program (Phase 1)," U.S. Nuclear Regulatory Commission, February 1982.
1450. Memorandum for J. Taylor from E. Beckjord, "Resolution of Generic Issue 113, 'Dynamic Qualification and Testing of Large Bore Hydraulic Snubbers,'" August 27, 1992. [9312220197]
1451. Memorandum for J. Taylor from E. Beckjord, "Resolution of Generic Issue (GI) 121, 'Hydrogen Control for PWR Dry Containments,'" March 24, 1992. [9312220194]

1452. Memorandum for W. Minners from F. Gillespie, "Prioritization of Generic Issue 78, 'Monitoring of Design Basis Transient Fatigue Limits for Reactor Coolant System,'" June 10, 1992. [9312220188]
1453. NRC Information Notice 92-06, "Reliability of ATWS Mitigation System and Other NRC Required Equipment Not Controlled by Plant Technical Specifications," January 15, 1992 [9201080305], (Supplement 1) July 1, 1993 [9306250303].
1454. Memorandum for W. Minners from B. Sheron, "Proposed Generic Issue 'RHR Pumps Inside Containment,'" August 23, 1985. [8508290373]
1455. NUREG/CR-5300, "Integrated Reliability and Risk Analysis System (IRRAS) Version 2.5," U.S. Nuclear Regulatory Commission, (Vol. 1) March 1991.
1456. NUREG/CR-5303, "System Analysis and Risk Assessment System (SARA) Version 4.0," U.S. Nuclear Regulatory Commission, (Vol. 1) February 1992, (Vol. 2) January 1992.
1457. Letter to C. Rourke (NRC) from N. Anderson (INEL), "Transmittal of Final Report, 'Analysis of Plant Specific Responses for the Resolution of Generic Issue A-30, Adequacy of Safety-Related DC Power Supplies,' (FIN D6025) NRA-20-92," July 9, 1992. [9502070242]
1458. SECY-87-297, "MARK I Containment Performance Program Plan," December 8, 1987. [8803080354]
1459. SECY-89-017, "MARK I Containment Performance Improvement Program," January 23, 1989. [8903090205]
1460. Memorandum for V. Stello from S. Chilk, "SECY-89-017 - MARK I Containment Performance Improvement Program," July 11, 1989. [8907270013]
1461. SECY-91-316, "Status of Severe Accident Research," October 7, 1991. [9110160271]
1462. Letter to D. Grace (BWR Owners Group) from A. Thadani (NRC), "Safety Evaluation of 'BWR Owners' Group - Emergency Procedure Guidelines, Revision 4,' NEDO-31331, March 1987," September 12, 1988. [8809190198]
1463. NRC Letter to All Holders of Operating Licenses for Nuclear Power Reactors With Mark I Containments, "Installation of a Hardened Wetwell Vent (Generic Letter No. 89-16)," September 1, 1989. [8909010375]
1464. NUREG/CR-5662, "Hydrogen Combustion, Control, and Value-Impact Analysis for PWR Dry Containments," U.S. Nuclear Regulatory Commission, June 1991.
1465. NUREG-1465, "Accident Source Terms for Light-Water Nuclear Power Plants," U.S. Nuclear Regulatory Commission, February 1995.
1466. NUREG/CR-5460, "A Cause-Defense Approach to the Understanding and Analysis of Common Cause Failures," U.S. Nuclear Regulatory Commission, March 1990.

1467. Federal Register Notice 56 FR 31306, "10 CFR 50, RIN 3150-AD00, Monitoring the Effectiveness of Maintenance at Nuclear Power Plants," July 10, 1991.
1468. Memorandum for E. Beckjord from F. Gillespie, "Potential Generic Issue - Adequacy of Emergency and Essential Lighting - (RES Office Letter No. 1, Rev. 1)," September 14, 1990. [9009210192]
1469. NUREG/CR-4834, "Recovery Actions in PRA for the Risk Methods Integration and Evaluation Program (RMIEP)," U.S. Nuclear Regulatory Commission, (Vol. 1) June 1987.
1470. NUREG/CR-4674, "Precursors to Potential Severe Core Damage Accidents," U.S. Nuclear Regulatory Commission, (Vols. 15 and 16) September 1992.
1471. NRC Information Notice No. 90-69, "Adequacy of Emergency and Essential Lighting," U.S. Nuclear Regulatory Commission, October 31, 1990. [9010250054]
1472. NUREG-1272, "Office for Analysis and Evaluation of Operational Data 1991 Annual Report," U.S. Nuclear Regulatory Commission, (Vol. 6, No. 1) August 1992.
1473. Memorandum for J. Taylor from S. Chilk, "SECY-89-102 - Implementation of the Safety Goals," June 15, 1990. [9007090094]
1474. Letter to W. Conway (Arizona Public Service Company) from C. Trammell (NRC), "Review of Eddy-Current Inspections of Steam Generator Tubes - Palo Verde Nuclear Generating Station, Unit No. 2 (TAC No. M86178)," June 8, 1993. [9306100267]
1475. NUREG-1477, "Voltage-Based Interim Plugging Criteria for Steam Generator Tubes," U.S. Nuclear Regulatory Commission, (Draft) June 1993.
1476. Memorandum for T. Murley from E. Beckjord, "Recommendations Regarding Revision of Standard Review Plan Sections Related to 'Stiff Pipe Clamps,'" August 12, 1992. [9312220199]
1477. Memorandum for T. Speis from F. Gillespie, "Consideration of New Generic Issue on 'Support Flexibility of Equipment and Components,'" January 30, 1989. [8903010215]
1478. NUREG/CR-2999, "Final Report USNRC Anchor Bolt Study: Data Survey and Dynamic Testing," U.S. Nuclear Regulatory Commission, December 1982.
1479. SECY-93-108, "Revised Guidelines for Prioritization of Generic Safety Issues," April 28, 1993. [9308230261]
1480. EPRI NP-6154, "Proceedings: EPRI/NRC/TPC Workshop on Seismic Soil-Structure Interaction Analysis Techniques Using Data From Lotung, Taiwan," Electric Power Research Institute, (Vol. 1) March 1989, (Vol. 2) March 1989.
1481. Memorandum for E. Beckjord from T. Murley, "Potential New Generic Issues," September 25, 1991. [9110250132]

1482. Memorandum for T. Murley from E. Beckjord, "Prioritization of Generic Issue 161, 'Associated Circuits,'" March 12, 1993. [9312220201]
1483. Regulatory Guide 1.9, "Selection, Design, and Qualification of Diesel-Generator Units Used as Standby (Onsite) Electric Power Systems at Nuclear Power Plants," March 1971, (Rev. 1) November 1978, (Rev. 2) December 1979 [8001220580], (Rev. 3) July 1993 [9308180045].
1484. Regulatory Guide 1.160, "Monitoring the Effectiveness of Maintenance at Nuclear Power Plants," U.S. Nuclear Regulatory Commission, June 1993 [9306250035], (Rev. 1) January 1995 [9501300137].
1485. Federal Register Notice 58 FR 41813, "Regulatory Guide; Withdrawal," August 5, 1993.
1486. Memorandum for J. Taylor from E. Beckjord, "Resolution of Generic Safety Issue B-56, 'Diesel Generator Reliability,'" June 29, 1993. [9312220205]
1487. NRC Letter to All Licensees of Operating Reactors, Applicants for An Operating License, and Holders of Construction Permits, "Proposed Staff Actions to Improve and Maintain Diesel Generator Reliability (Generic Letter 84-15)," July 2, 1984. [8407020206]
1488. Memorandum for E. Jordan from T. Novak, "Engineering Evaluation Report - Pump Damage Due to Low Flow Cavitation (AEOD/E807)," October 18, 1988. [9312220206, 8811170140, 8810250191]
1489. NRC Bulletin No. 88-04, "Potential Safety-Related Pump Loss," U.S. Nuclear Regulatory Commission, May 5, 1988. [8804290177]
1490. NUREG/CR-5706, "Potential Safety-Related Pump Loss: An Assessment of Industry Data," U.S. Nuclear Regulatory Commission, June 1991.
1491. Memorandum for J. Norberg from R. Jones, "Review of Responses to Bulletin 88-04," July 22, 1991. [9108010062]
1492. NUREG/CR-5404, "Auxiliary Feedwater System Aging Study," U.S. Nuclear Regulatory Commission, (Vol. 1) March 1990, (Vol. 2) July 1993.
1493. Memorandum for V. Stello from S. Chilk, "Staff Requirements - Briefing on Status of Unresolved Safety/Generic Issues, 10:00 a.m., Wednesday, October 21, 1987, Commissioners' Conference Room, D. C. Office (Open to Public Attendance)," November 6, 1987. [8711100418]
1494. NUREG/CR-5604, "Assessment of ISLOCA Risk - Methodology and Application to a Babcock and Wilcox Nuclear Power Plant," U.S. Nuclear Regulatory Commission, (Vol. 1) April 1992, (Vol. 2) April 1992, (Vol. 3) April 1992.
1495. NUREG/CR-5744, "Assessment of ISLOCA Risk - Methodology and Application to a Westinghouse Four-Loop Ice Condenser Plant," U.S. Nuclear Regulatory Commission, April 1992.

1496. NUREG/CR-5745, "Assessment of ISLOCA Risk - Methodology and Application to a Combustion Engineering Plant," U.S. Nuclear Regulatory Commission, April 1992.
1497. NUREG/CR-5603, "Pressure-Dependent Fragilities for Piping Components," U.S. Nuclear Regulatory Commission, October 1990.
1498. NUREG/CR-5862, "Screening Methods for Developing Internal Pressure Capacities for Components in Systems Interfacing With Nuclear Power Plant Reactor Coolant Systems," U.S. Nuclear Regulatory Commission, May 1992.
1499. NUREG/CR-5928, "ISLOCA Research Program Final Report," U.S. Nuclear Regulatory Commission, July 1993.
1500. NUREG/CR-5102, "Interfacing System LOCA: Pressurized Water Reactors," U.S. Nuclear Regulatory Commission, February 1989.
1501. NUREG-1463, "Regulatory Analysis for the Resolution of Generic Safety Issue 105: Interfacing System Loss-of-Coolant Accident in Light-Water Reactors," U.S. Nuclear Regulatory Commission, July 1993.
1502. NRC Information Notice 92-36, "Intersystem LOCA Outside Containment," May 7, 1992 [9205010045], (Supplement 1) February 22, 1994 [9402150320].
1503. Memorandum for F. Gillespie from W. Minners, "Proposed Resolution of Generic Issue 105, 'Interfacing Systems LOCA in LWRs,'" April 2, 1993. [9312220208]
1504. Memorandum for J. Taylor from E. Beckjord, "Technical Resolution of Generic Issue 105 (GI-105) 'Interfacing Systems Loss of Coolant Accident (ISLOCA) in LWRs,'" June 3, 1993. [9312220210]
1505. Memorandum for J. Taylor from S. Chilk, "SECY-93-108 - Revised Guidelines for Prioritization of Generic Safety Issues," July 23, 1993. [9308270094]
1506. Memorandum for W. Minners from L. Shao, "Closeout of GSI 119.4," July 17, 1992. [9312220212]
1507. Regulatory Guide 1.44, "Control of the Use of Sensitized Stainless Steel," U.S. Atomic Energy Commission, May 1973. [7907100182]
1508. Memorandum for J. Taylor from E. Beckjord, "Final Technical Resolution of Generic Safety Issue 120, 'On-Line Testability of Protection System,'" March 4, 1993. [9502070269]
1509. Letter to J. Taylor from P. Shewmon, "Prioritization of Generic Issue 152, 'Design Basis for Valves that Might Be Subjected to Significant Blowdown Loads,'" April 23, 1993. [9305060143]
1510. Letter to J. Wilkins (ACRS) from J. Taylor (EDO) June 8, 1993. [9306210081, 9305200137]
1511. Memorandum for J. Taylor from E. Beckjord, "Resolution of GI-142, 'Leakage Through Electrical Isolators,'" March 9, 1993. [9312220214]

1512. NUREG-1461, "Regulatory Analysis for the Resolution of Generic Issue 153: Loss of Essential Service Water in LWRs," U.S. Nuclear Regulatory Commission, August 1993.
1513. Memorandum for J. Taylor from E. Beckjord, "Resolution of Generic Issue 153 (GI-153), 'Loss of Essential Service Water in LWRs,'" June 14, 1993. [9312220216]
1514. NUREG/CR-5910, "Loss of Essential Service Water in LWRs (GI-153)," U.S. Nuclear Regulatory Commission, August 1992.
1515. Memorandum for E. Beckjord from T. Murley, "Proposed New Generic Issue: 'Determination of Neutron Fluence to PWR Pressure Vessels,'" October 8, 1992. [9210190215]
1516. Memorandum for T. Murley from E. Beckjord, "Proposed New Generic Issue: Determination of Neutron Fluence to PWR Pressure Vessels," November 30, 1992. [9312220218]
1517. Memorandum for J. Sniezek from T. Murley and E. Beckjord, "Resolution of Fatigue and Environmental Qualification Issues Related to License Renewal," April 1, 1993. [9304270324]
1518. Memorandum for The Chairman, et al., from J. Taylor, "Environmental Qualification of Electric Equipment," May 27, 1993. [9308180153]
1519. SEASF-LR-92-022, "Supplemental Study of Generic Issue No. 153, 'Loss of Essential Service Water in LWRs,'" Science and Engineering Associates, Inc., (Rev. 1) January 1993. [9502070279]
1520. Memorandum for E. Beckjord from T. Murley, "Request to Prioritize a New Generic Issue for Spring-Actuated Safety and Relief Valve Reliability," October 8, 1992. [9312280153]
1521. NUREG/CR-3696, "Potential Human Factors Deficiencies in the Design of Local Control Stations and Operator Interfaces in Nuclear Power Plants," U.S. Nuclear Regulatory Commission, April 1984.
1522. NUREG/CR-3217, "Near-Term Improvements for Nuclear Power Plant Control Room Annunciator Systems," U.S. Nuclear Regulatory Commission, April 1983.
1523. NUREG/CR-3987, "Computerized Annunciator Systems," U.S. Nuclear Regulatory Commission, June 1985.
1524. NUREG/CR-5572, "An Evaluation of the Effects of Local Control Station Design Configurations on Human Performance and Nuclear Power Plant Risk," U.S. Nuclear Regulatory Commission, September 1990.
1525. Memorandum for J. Taylor from E. Beckjord, "Termination of Work on Generic Safety Issue HF5.1 'Local Control Stations,'" June 29, 1993. [9312220224]
1526. Memorandum for J. Taylor from E. Beckjord, "Resolution of Human Factors Generic Issue 5.2, 'Review Criteria for Human Factors Aspects of Advanced Controls and Instrumentation,'" June 29, 1993. [9312220225]

1527. NUREG/CR-5186, "Value/Impact Analysis of Generic Issue 94, 'Additional Low Temperature Overpressure Protection for Light Water Reactors,'" U.S. Nuclear Regulatory Commission, November 1988.
1528. NRC Information Notice No. 90-22, "Unanticipated Equipment Actuations Following Restoration of Power to Rosemount Transmitter Trip Units," U.S. Nuclear Regulatory Commission, March 23, 1990. [9003190349]
1529. NUREG-1422, "Summary of Chernobyl Followup Research Activities," U.S. Nuclear Regulatory Commission, June 1992.
1530. Memorandum for J. Taylor from E. Beckjord, "Resolution of Generic Issue 155.1, 'More Realistic Source Term Assumptions,'" March 13, 1995. [9511090074]
1531. NUREG-0940, "Enforcement Actions: Significant Actions Resolved," U.S. Nuclear Regulatory Commission, (Vol. 14, No. 2, Parts 1, 2, and 3) August 1995.
1532. Memorandum for C. Serpan from W. Minners, "Identification of New Generic Issue: Hydrogen Storage Facility Separation," December 16, 1993. [9312290134]
1533. Letter to the NRC from M. Tuckman (Duke Power Company), "Oconee Nuclear Station, Docket Nos. 50-269, 50-270, and 50-287, Generic Letter 88-20," November 30, 1990. [9012060005]
1534. EGG-SSRE-9747, "Improved Estimates of Separation Distances to Prevent Unacceptable Damage to Nuclear Power Plant Structures from Hydrogen Detonation for Gaseous Hydrogen Storage," Idaho National Engineering Laboratory, (Draft) November 1993. [9502070287]
1535. SCIE-EGG-103-89, "Draft Technical Evaluation Report on U.S. Commercial Power Reactor Hydrogen Tank Farms and Their Compliance With Separation Distance Safety Criteria," Scientech, Inc., March 1990. [9502070289]
1536. Memorandum for J. Taylor from E. Beckjord, "Resolution of GI-I.D.3, 'Safety System Status Monitoring,'" August 20, 1993. [9502070295]
1537. Memorandum for T. Murley from E. Beckjord, "Research Information Letter Number 171, 'Continuous On-Line Reactor Surveillance System,'" May 4, 1993. [9305100271]
1538. Memorandum for J. Taylor from E. Beckjord, "Closure of Generic Issue I.D.5(3), 'On-Line Automated Continuous Reactor Surveillance Systems,'" November 12, 1993. [9502070301]
1539. SECY-93-119, "TMI-2 Vessel Investigation Project," May 5, 1993. [9305100253]
1540. Memorandum for J. Taylor from E. Beckjord, "Closure of Generic Issue II.H.2, 'Obtain Data on Conditions Inside TMI-2 Containment,'" February 9, 1994. [9502070304]
1541. NUREG-1472, "Regulatory Analysis for the Resolution of Generic Issue 57: Effects of Fire Protection System Actuation on Safety-Related Equipment," U.S. Nuclear Regulatory Commission, October 1993.

1542. Memorandum for J. Taylor from E. Beckjord, "Resolution of Generic Safety Issue (GSI)-57, 'Effects of Fire Protection System Actuation on Safety-Related Equipment,'" September 30, 1993. [9502070315]
1543. NRC Information Notice 94-12, "Insights Gained from Resolving Generic Issue 57: Effects of Fire Protection System Actuation on Safety-Related Equipment," U.S. Nuclear Regulatory Commission, February 9, 1994. [9402030011]
1544. NUREG/CR-5759, "Risk Analysis of Highly Combustible Gas Storage, Supply, and Distribution Systems in Pressurized Water Reactor Plants," U.S. Nuclear Regulatory Commission, June 1993.
1545. NUREG-1364, "Regulatory Analysis for the Resolution of Generic Safety Issue 106: Piping and the Use of Highly Combustible Gases in Vital Areas," U.S. Nuclear Regulatory Commission, June 1993.
1546. Memorandum for J. Taylor from E. Beckjord, "Proposed Resolution of GSI-106, 'Piping and the Use of Highly Combustible Gases in Vital Areas,'" November 3, 1993. [9502070320]
1547. Letter to All Holders of Operating Licenses or Construction Permits for Nuclear Power Reactors, "Research Results on Generic Safety Issue 106, 'Piping and the Use of Highly Combustible Gases in Vital Areas,' (Generic Letter 93-06)," U.S. Nuclear Regulatory Commission, October 25, 1993. [9310200286]
1548. Memorandum for F. Gillespie from E. Beckjord, "Generic Letter for Implementation of Resolution of Generic Safety Issue 106, 'Piping and the Use of Highly Combustible Gases in Vital Areas,'" December 14, 1992. [9502070322]
1549. NUREG-1427, "Regulatory Analysis for the Resolution of Generic Issue 143: Availability of Chilled Water System and Room Cooling," U.S. Nuclear Regulatory Commission, December 1993.
1550. NUREG/CR-6084, "Value Impact Analysis of Generic Issue 143, 'Availability of Heating, Ventilation, Air Conditioning (HVAC) and Chilled Water Systems,'" U.S. Nuclear Regulatory Commission, November 1993.
1551. Memorandum for J. Taylor from E. Beckjord, "Resolution of Generic Issue 143 (GI-143), 'Availability of Chilled Water System and Room Cooling,'" September 30, 1993. [9502070325]
1552. NRC Information Notice No. 89-44, "Hydrogen Storage on the Roof of the Control Room," U.S. Nuclear Regulatory Commission, April 27, 1989. [8904260247]
1553. Memorandum for A. Gody from G. Holahan, "Prioritization of Proposed Generic Issue 162, 'Inadequate Technical Specifications for Shared Systems at Multi-plant Sites When One Unit Is Shut Down,'" March 20, 1993. [9304070074]
1554. Memorandum for J. Taylor from E. Beckjord, "Resolution of GI 67.5.1, 'Reassessment of SGTR Radiological Consequences,'" June 30, 1994. [9407130262]

1555. Memorandum for E. Beckjord from J. Murphy, "Staff Review Guidance for Generic Safety Issue (GSI) 147, 'Fire-Induced Alternate Shutdown/Control Room Panel Interactions,'" March 9, 1994. [9502070329]
1556. Memorandum for C. Rossi, et al., from T. Novak, "Safety and Safety/Relief Valve Reliability," April 24, 1992. [9205060277]
1557. NRC Information Notice No. 90-05, "Inter-System Discharge of Reactor Coolant," U.S. Nuclear Regulatory Commission, January 29, 1990. [9001230126]
1558. NRC Information Notice 92-64, "Nozzle Ring Settings on Low Pressure Water-Relief Valves," U.S. Nuclear Regulatory Commission, August 28, 1992. [9208240139]
1559. NRC Information Notice 92-61, "Loss of High Head Safety Injection," U.S. Nuclear Regulatory Commission, August 20, 1992 [9208180039], (Supplement 1) November 6, 1992 [9211020211].
1560. NUREG/CR-6001, "Aging Assessment of BWR Standby Liquid Control Systems," U.S. Nuclear Regulatory Commission, August 1992.
1561. NRC Information Notice No. 90-18, "Potential Problems With Crosby Safety Relief Valves Used on Diesel Generator Air Start Receiver Tanks," U.S. Nuclear Regulatory Commission, March 9, 1990. [9003050043]
1562. Memorandum for J. Taylor from E. Beckjord, "Resolution of Human Factors Generic Issue 4.4, 'Guidelines for Upgrading Other Procedures,'" July 29, 1993. [9502070331]
1563. NUREG/CR-5382, "Screening of Generic Safety Issues for License Renewal Considerations," U.S. Nuclear Regulatory Commission, December 1991.
1564. Memorandum for W. Russell from E. Beckjord, "License Renewal Implications of Generic Safety Issues (GSIs) Prioritized and/or Resolved Between October 1990 and March 1994," May 5, 1994. [9406170365]
1565. Memorandum for T. Murley from W. Russell and J. Partlow, "Closeout of TMI Action Plan Items III.A.1.2 and III.A.2.2 (Multi-Plant Actions F-63, F-64, F-65, and F-68)," October 2, 1990. [9010160111]
1566. SECY-80-275, "Final Rulemaking on Emergency Preparedness," June 3, 1980. [8007090015]
1567. NUREG/CP-0011, "Proceedings to Workshops Held on Proposed Rulemaking on Emergency Planning for Nuclear Power Plants," U.S. Nuclear Regulatory Commission, April 1980.
1568. Federal Register Notice 46 FR 11666, "10 CFR Parts 30, 40, 50, 70 and 72 Decommissioning Criteria for Nuclear Facilities; Notice of Availability of Draft Generic Environment Impact Statement," February 10, 1981.

1569. SECY-87-309, "Final Rule Amendments to 10 CFR Parts 30, 40, 50, 51, 70, and 72: General Requirements for Decommissioning Nuclear Facilities," December 17, 1987. [8801130361]
1570. SECY-88-94, "Final Rule Amendments to 10 CFR Parts 30, 40, 50, 51, 70, and 72: General Requirements for Decommissioning Nuclear Facilities (SECY-87-309)," April 5, 1988. [8804120065]
1571. SECY-94-179, "Notice of Proposed Rulemaking on Decommissioning of Nuclear Power Reactors," July 7, 1994. [9407180100]
1572. Memorandum for J. Taylor and K. Cyr from J. Hoyle, "SECY-94-179 - Notice of Proposed Rulemaking on Decommissioning of Nuclear Power Reactors and COMKR-94-002 - Decommissioning of Nuclear Power Reactors and Comments on SECY-94-179," October 5, 1994. [9410270085]
1573. NRC Letter to All Pressurized Water Reactor Licensees, "Inadvertent Boron Dilution Events (Generic Letter 85-05)," January 31, 1985. [8502010366]
1574. Memorandum for J. Taylor from R. Bernero, "Resolution of Issue Number B-64, 'Decommissioning of Reactors,' of the Generic Issue Management Control System," September 26, 1994. [9410110028]
1575. Memorandum for C. Serpan and C. Ader from J. Greeves, "Reference to the U.S. Nuclear Regulatory Commission Dam Safety Program in NUREG-0933," August 12, 1994. [9409060217]
1576. "Approaches to Upgrading Procedures in Nuclear Power Plants," Pacific Northwest Laboratory, August 1994. [9507280167]
1577. NUREG/CR-6146, "Local Control Stations: Human Engineering Issues and Insights," U.S. Nuclear Regulatory Commission, September 1994.
1578. NUREG/CR-6105, "Human Factors Engineering Guidance for the Review of Advanced Alarm Systems," U.S. Nuclear Regulatory Commission, September 1994.
1579. Letter to L. Zech from F. Remick, "Resolution of Generic Issue 43, 'Air Systems Reliability,'" January 19, 1989. [8901260092]
1580. Memorandum for J. Larkins from E. Beckjord, "Evaluation of Potential Safety Issues from the Multiple System Responses Program," June 3, 1994. [9406230143]
1581. Memorandum for T. Speis from A. Thadani, "Review of NUREG/CR-5420," April 30, 1995. [9505230058]
1582. NUREG/CR-5455, "Development of NRC's Human Performance Investigation Process (HPIP)," U.S. Nuclear Regulatory Commission, (Vol. 1) October 1993, (Vol. 2) October 1993, (Vol. 3) October 1993.

1583. NUREG-0711, "Human Factors Engineering Program Review Model," U.S. Nuclear Regulatory Commission, July 1994.
1584. NUREG/CR-5908, "Advanced Human-System Interface Design Review Guideline," U.S. Nuclear Regulatory Commission, (Vol. 1) July 1994, (Vol. 2) July 1994.
1585. Memorandum for L. Shao from B. Sheron, "Proposed Generic Issue on Safety Systems' Response to the Sequential Occurrence of LOCA and Loss of Offsite Power Events," February 17, 1995. [9502270179]
1586. NRC Information Notice 93-17, "Safety Systems Response to Loss of Coolant and Loss of Offsite Power," March 8, 1993 [9303020536], (Rev. 1) March 25, 1994 [9403220236].
1587. NUREG-1335, "Individual Plant Examination: Submittal Guidance," U.S. Nuclear Regulatory Commission, August 1989.
1588. NUREG/CR-5580, "Evaluation of Generic Issue 57: Effects of Fire Protection System Actuation on Safety-Related Equipment," (Vol. 1) December 1992, (Vol. 2) December 1992, (Vol. 3) December 1992, (Vol. 4) December 1992, (Vol. 5) December 1992.
1589. NUREG/CR-5720, "Motor-Operated Valve Research Update," U.S. Nuclear Regulatory Commission, June 1992.
1590. EGG-REQ-7297, "Summary of Valve Assemblies in High Energy BWR Systems Outside of Containment -- Interim Report," EG&G Idaho, Inc., June 1986. [9511090106]
1591. Regulatory Guide 1.22, "Periodic Testing of Protection System Actuation Functions," U.S. Nuclear Regulatory Commission, February 1972. [7907100108]
1592. Regulatory Guide 1.118, "Periodic Testing of Electric Power and Protection Systems," U.S. Nuclear Regulatory Commission, June 1976, (Rev. 1) November 1977, (Rev. 2) June 1978 [7907110110], (Rev. 3) April 1995 [9505030214].
1593. NUREG-1453, "Regulatory Analysis for the Resolution of Generic Issue 142: Leakage Through Electrical Isolators in Instrumentation Circuits," U.S. Nuclear Regulatory Commission, September 1993.
1594. Regulatory Guide 1.77, "Assumptions Used for Evaluating a Control Rod Ejection Accident for Pressurized Water Reactors," U.S. Nuclear Regulatory Commission, May 1974. [7907100299]
1595. Memorandum for E. Beckjord from T. Murley, "User Need for Assistance on High Burnup Fuels," October 4, 1993. [9310270186]
1596. Memorandum for W. Russell from E. Beckjord, "Fuel Damage Criteria for Reactivity Transients," April 29, 1994. [9511090065]
1597. NRC Information Notice 94-64, "Reactivity Insertion Transient and Accident Limits for High Burnup Fuel," August 31, 1994 [9408250234], (Supplement 1) April 6, 1995 [9503310049].

1598. Memorandum to the Chairman, et al., from J. Taylor, "Reactivity Transients and High-Burnup Fuel," September 13, 1994. [9409300142]
1599. Memorandum to the Chairman, et al., from J. Taylor, "Reactivity Transients and Fuel Damage Criteria for High Burnup Fuel," November 9, 1994. [9511090217]
1600. NUREG/CP-0139, "Transactions of the Twenty-Second Water Reactor Safety Information Meeting," U.S. Nuclear Regulatory Commission, October 1994.
1601. Memorandum to C. Serpan from A. Chaffee, "Nuclear Reactor Regulation (NRR) Input Into Research NUREG-0933 (WITS Item 9400213)," February 13, 1996. [9602260124]
1602. NUREG-1352, "Action Plans for Motor-Operated Valves and Check Valves," U.S. Nuclear Regulatory Commission, June 1990.
1603. Memorandum to A. Thadani from J. Strosnider, "Plan for Addressing Generic Reactor Pressure Vessel Issues," August 9, 1995. [9508150078]
1604. NUREG-1511, "Reactor Pressure Vessel Status Report," U.S. Nuclear Regulatory Commission, December 1994.
1605. Memorandum to A. Thadani from J. Strosnider, "Assessment of Impact of Increased Variability in Chemistry on the RT<sub>PTS</sub> Value of PWR Reactor Vessels," May 5, 1995 [9505100187]
1606. SECY-91-161, "Schedules for the Advanced Reactor Reviews and Regulatory Guidance Revisions," May 31, 1991. [9106050174]
1607. Memorandum for W. Russell from F. Gillespie, "Action Plan for the Development of Draft SRP Revisions in the SRP-UDP," May 17, 1994. [9406280148, 9405270273]
1608. Memorandum to J. Taylor from C. Paperiello and W. Russell, "Dry Cask Storage Action Plan," July 28, 1995. [9508250186]
1609. Memorandum to J. Taylor from W. Russell and R. Bernero, "Realignment of Reactor Decommissioning Program," March 15, 1995. [9508250180]
1610. NUREG-1474, "Effect of Hurricane Andrew on the Turkey Point Nuclear Generating Station from August 20 - 30, 1992," U.S. Nuclear Regulatory Commission and the Institute of Nuclear Power Operations, March 1993. [9307060041]
1611. Memorandum for J. Taylor from T. Murley, "Office of Nuclear Reactor Regulation (NRR) Plan for Generic Follow-on Actions - 'Report on the Effect of Hurricane Andrew on the Turkey Point Nuclear Generating Station from August 20 - 30, 1992,'" July 22, 1993. [9308160297]
1612. NRC Information Notice 93-53, "Effect of Hurricane Andrew on Turkey Point Nuclear Generating Station and Lessons Learned," July 20, 1993 [9307140056], (Supplement 1) April 29, 1994 [9404280023].

1613. NUREG/CR-6224, "Parametric Study of the Potential for BWR ECCS Strainer Blockage Due to LOCA Generated Debris," U.S. Nuclear Regulatory Commission, October 1995.
1614. NRC Information Notice 92-71, "Partial Plugging of Suppression Pool Strainers at a Foreign BWR," September 30, 1992. [9209290014]
1615. NRC Bulletin No. 93-02, "Debris Plugging of Emergency Core Cooling Suction Strainers," May 11, 1993. [9305110015]
1616. NRC Information Notice 93-34, "Potential for Loss of Emergency Cooling Function Due to a Combination of Operational and Post-LOCA Debris in Containment," April 26, 1993 [9304260085], (Supplement 1) May 6, 1993 [9305050002].
1617. NRC Bulletin 95-02, "Unexpected Clogging of a Residual Heat Removal (RHR) Pump Strainer While Operating in Suppression Pool Cooling Mode," October 17, 1995. [9510040059]
1618. NRC Information Notice No. 88-28, "Potential for Loss of Post-LOCA Recirculation Capability Due to Insulation Debris Blockage," May 19, 1988. [8805130108]
1619. NRC Information Notice 92-85, "Potential Failures of Emergency Core Cooling Systems Caused by Foreign Material Blockage," December 23, 1992. [9212170209]
1620. NRC Information Notice 94-57, "Debris in Containment and the Residual Heat Removal System," August 12, 1994. [9408080111]
1621. NRC Information Notice 95-06, "Potential Blockage of Safety-Related Strainers by Material Brought Inside Containment," January 25, 1995. [9501190091]
1622. NRC Information Notice 95-47, "Unexpected Opening of a Safety/Relief Valve and Complications Involving Suppression Pool Cooling Strainer Blockage," October 4, 1995 [9510030107], (Rev. 1) November 30, 1995 [9511270084].
1623. Memorandum to A. Thadani from G. Holahan, "Task Action Plan for Spent Fuel Storage Pool Safety," October 13, 1994. [9410190155]
1624. NRC Information Notice 94-38, "Results of a Special NRC Inspection at Dresden Nuclear Power Station Unit 1 Following a Rupture of Service Water Inside Containment," May 27, 1994. [9405240025]
1625. NRC Bulletin 94-01, "Potential Fuel Pool Draindown Caused by Inadequate Maintenance Practices at Dresden Unit 1," April 14, 1994. [9404120041]
1626. Memorandum for A. Thadani from G. Holahan, "Revision to Report on the Re-Assessment of the NRC Fire Protection Program," February 27, 1993. [9504190319]
1627. SECY-93-143, "NRC Staff Actions to Address the Recommendations in the Report on the Reassessment of the NRC Fire Protection Program," May 21, 1993. [9306030231]

1628. SECY-95-034, "Status of Recommendations Resulting from the Reassessment of the NRC Fire Protection Program," February 13, 1995. [9503060019]
1629. Memorandum for Chairman Jackson, et al., from J. Taylor, "Semiannual Report on the Status of the Thermo-Lag Action Plan and Fire Protection Task Action Plan," September 20, 1995. [9509250375]
1630. SECY-94-219, "Proposed Agency-Wide Implementation Plan for Probabilistic Risk Assessment (PRA)," August 19, 1994. [9409090234]
1631. SECY-95-079, "Status Update of the Agency-Wide Implementation Plan for Probabilistic Risk Assessment," March 30, 1995. [9504100180]
1632. SECY-95-126, "Final Policy Statement on the Use of Probabilistic Risk Assessment Methods in Nuclear Regulatory Activities," May 18, 1995. [9506020152]
1633. SECY-95-280, "Framework for Applying Probabilistic Risk Analysis in Reactor Regulation," November 27, 1995. [9512180168, 9512040133]
1634. Memorandum to Chairman Jackson from J. Taylor, "Improvements Associated With Managing the Utilization of Probabilistic Risk Assessment (PRA) and Digital Instrumentation and Control Technology," January 3, 1996. [9601180203]
1635. Memorandum for J. Taylor from T. Murley, et al., "Agency Directions for Current and Future Uses of Probabilistic Risk Assessment (PRA)," November 2, 1993. [9311100145]
1636. Memorandum to A. Thadani from G. Holahan and R. Spessard, "Action Plan to Monitor, Review, and Improve Fuel and Core Components Operating Performance," October 7, 1994. [9411040040]
1637. NRC Letter to All Operating Reactor Licensees, "Licensee Qualification for Performing Safety Analyses in Support of Licensing Actions (Generic Letter No. 83-11)," February 4, 1983. [8302080304]
1638. NRC Letter to All Power Reactor Licensees and Applicants, "Removal of Cycle-Specific Parameter Limits from Technical Specifications (Generic Letter 88-16)," October 4, 1988. [8810050058, 8810140007]
1639. NRC Information Notice No. 91-47, "Failure of Thermo-Lag Fire Barrier Material to Pass Fire Endurance Test," August 6, 1991. [9108020180]
1640. NRC Information Notice 91-79, "Deficiencies in the Procedures for Installing Thermo-Lag Fire Barrier Materials," December 6, 1991 [9112020091], (Supplement 1) August 4, 1994 [9408030006].
1641. NRC Information Notice 92-55, "Current Fire Endurance Test Results for Thermo-Lag Fire Barrier Material," July 27, 1992. [9207270345]
1642. NRC Information Notice 92-82, "Results of Thermo-Lag 330-1 Combustibility Testing," December 15, 1992. [9212090211]

1643. NRC Information Notice 94-22, "Fire Endurance and Ampacity Derating Test Results for 3-Hour Fire-Rated Thermo-Lag 330-1 Fire Barriers," March 16, 1994. [9403150511]
1644. NRC Information Notice 94-34, "Thermo-Lag 330-660 Flexi-Blanket Ampacity Derating Concerns," May 13, 1994. [9405090108]
1645. NRC Information Notice 94-86, "Legal Actions Against Thermal Science, Inc., Manufacturer of Thermo-Lag," December 22, 1994. [9412160132]
1646. NRC Information Notice 95-27, "NRC Review of Nuclear Energy Institute, 'Thermo-Lag 330-1 Combustibility Evaluation Methodology Plant Screening Guide,'" May 31, 1995. [9505240424]
1647. NRC Information Notice 95-32, "Thermo-Lag 330-1 Flame Spread Test Results," August 10, 1995. [9508040074]
1648. NRC Information Notice 95-49, "Seismic Adequacy of Thermo-Lag Panels," October 27, 1995. [9510240388]
1649. NRC Information Notice 95-03, "Loss of Reactor Coolant Inventory and Potential Loss of Emergency Mitigation Functions While in a Shutdown Condition," January 18, 1995 [9501110412], (Supplement 1) March 25, 1996 [9602050208].
1650. NRC Letter to All Power Reactor Licensees and Applicants for Power Reactor Licenses, "Policy Statement on Engineering Expertise on Shift (Generic Letter 86-04)," February 13, 1986. [8602240459]
1651. NRC Information Notice 91-77, "Shift Staffing at Nuclear Power Plants," November 26, 1991. [9111200123]
1652. NRC Information Notice 93-44, "Operational Challenges During a Dual-Unit Transient," June 15, 1993. [9306080170]
1653. NRC Information Notice 93-81, "Implementation of Engineering Expertise on Shift," October 12, 1993. [9310060239]
1654. NRC Information Notice 95-48, "Results of Shift Staffing Study," October 10, 1995. [9510040181]
1655. Letter to B. Boger (NRC) from R. Whitesel (NUMARC), December 29, 1992. [9301080124]
1656. SECY-93-184, "Shift Staffing at Nuclear Power Plants," June 29, 1993. [9307080273]
1657. SECY-93-193, "Policy on Shift Technical Advisor Position at Nuclear Power Plants," July 13, 1993. [9307200065]
1658. NRC Bulletin No. 90-01, "Loss of Fill-Oil in Transmitters Manufactured by Rosemount," March 9, 1990 [9003050148], (Supplement 1) December 22, 1992 [9212170002].

1659. Memorandum for R. Zimmerman, et al., from J. Sniezek, "Review of Rosemount Transmitter Issues," May 21, 1993. [9308090287, 9310010241]
1660. SECY-95-078, "Staff Actions to Address Recommendations Resulting from Recent Evaluations of the Notice of Enforcement Discretion (NOED) Policy and Process," March 29, 1995. [9504100173]
1661. NUREG-1600, "General Statement of Policy and Procedures for NRC Enforcement Actions," U.S. Nuclear Regulatory Commission, July 1995.
1662. Memorandum for E. Jordan, et al., from J. Taylor, "Unauthorized Forced Entry Into the Protected Area at Three Mile Island Unit 1 on February 7, 1993 (NUREG-1485)," June 18, 1993. [9308110317]
1663. NRC Letter to All Holders of Operating Licenses or Construction or Construction Permits for Nuclear Power Reactors, Except for Big Rock Point and Facilities Permanently or Indefinitely Shut Down, "Emergency Response Data System Test Program (Generic Letter 93-01)," March 3, 1993. [9302240242]
1664. NRC Information Notice 93-94, "Unauthorized Forced Entry Into the Protected Area at Three Mile Island Unit 1 on February 7, 1993," December 9, 1993. [9312030104]
1665. NUREG-1485, "Unauthorized Forced Entry Into the Protected Area at Three Mile Island Unit 1 on February 7, 1993," U.S. Nuclear Regulatory Commission, April 1993.
1666. NUREG/CR-6432, "Estimated Net Value and Uncertainty for Automating ECCS Switchover at PWRs," U.S. Nuclear Regulatory Commission, February 1996.
1667. Memorandum for J. Taylor from D. Morrison, "Technical Resolution of Generic Issue 24 (GI-24), 'Automatic ECCS Switch to Recirculation,'" October 31, 1995. [9511140037]
1668. NUREG/CR-5904, "Functional Issues and Environmental Qualification of Digital Protection Systems of Advanced Light-Water Nuclear Reactors," U.S. Nuclear Regulatory Commission, April 1994.
1669. NUREG/CR-5941, "Technical Basis for Evaluating Electromagnetic and Radio-Frequency Interference in Safety-Related I&C Systems," U.S. Nuclear Regulatory Commission, April 1994.
1670. NRC Generic Letter 94-03, "Intergranular Stress Corrosion Cracking of Core Shrouds in Boiling Water Reactors," July 25, 1994. [9407210200]
1671. Memorandum for A. Thadani from B. Sheron, "Staff Action Plan for the Resolution of Issues Associated With Boiling Water Reactor Internals Cracking," April 26, 1995. [9505220070]
1672. NRC Letter to All Holders of Operating Licenses or Construction Permits for Nuclear Power Plants (Except Yankee Atomic Electric Company, Licensee for the Yankee Nuclear Power Station), "Reactor Vessel Structural Integrity, 10 CFR 50.54(f) (Generic Letter 92-01)," February 28, 1992, (Rev. 1) March 6, 1992 [9203060147], (Rev. 1, Supplement 1) May 19, 1995 [9505090312].

1673. Memorandum for E. Beckjord from W. Russell, "NRR User Need Request for Support of Resolving Problem of Stress Corrosion Cracking of Reactor Vessel Internal Components," December 2, 1994. [9505090299]
1674. Memorandum for D. Morrison from W. Russell, "Request for Research on Reactor Pressure Vessel Integrity," August 11, 1995. [9508220323]
1675. Memorandum to L. Shao from M. Mayfield, "Summary, NRC/NEI Workshop on Nuclear RPV Integrity," September 6, 1995. [9509200141]
1676. NRC Administrative Letter 95-03, "Availability of Reactor Vessel Integrity Database," August 4, 1995. [9508010148]
1677. AEOD/S95-01, "Reactor Coolant System Blowdown at Wolf Creek on September 17, 1994," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, March 1995. [9503310036]
1678. Memorandum to A. Thadani from R. Jones, "Proposed Action Plan for the 'Wolf Creek Draindown Event,'" September 8, 1995. [9509140225]
1679. Memorandum for the Chairman from J. Taylor, "Commercial Contract for Technical Assistance to Support the Standard Review Plan Update and Development Program," November 18, 1991.
1680. Memorandum for J. Taylor from J. Selin, "Commercial Contract for Technical Assistance to Support the Standard Review Plan Update and Development Program," December 13, 1991.
1681. Memorandum for J. Taylor from T. Murley, "Planned Actions to Address the Issues from the Office of Inspector General's Report on the NRC Staff's Review and Acceptance of Thermo-Lag 330-1 Fire Barrier Material," August 21, 1992. [9209250288]
1682. NRC Bulletin No. 92-01, "Failure of Thermo-Lag 330 Fire Barrier System to Maintain Cabling in Wide Cable Trays and Small Conduits Free from Fire Damage," June 24, 1992. [9206240122]
1683. Memorandum for W. Russell from T. Murley, "Final Report - Special Review Team for the Review of Thermo-Lag Fire Barrier Performance," April 21, 1992. [9205120277]
1684. Memorandum for E. Beckjord from T. Murley, "Request for Prioritization of Potential Generic Safety Issue - BWR MSIV Common Mode Failure Due to Loss of Accumulator Pressure," May 25, 1993. [9308160285]
1685. Memorandum for T. Murley from E. Beckjord, "Request for Prioritization of Potential Generic Safety Issue - BWR MSIV Common Mode Failure Due to Loss of Accumulator Pressure," June 29, 1993. [9509050193]
1686. SEA No. 95-3101-01-A:1, "Technical Information for Prioritization of Generic Safety Issues," Science and Engineering Associates, Inc., June 1996. [9704090123]

1687. SEA No. 97-3701-010-A:1, "Issue 107, Main Transformer Failures," Science and Engineering Associates, Inc., March 28, 1997. [9704090149]
1688. NUREG/CR-5595, "FORECAST: Regulatory Effects Cost Analysis Software Manual," (Rev. 1) July 1996.
1689. Memorandum to J. Taylor from J. Hoyle, "COMSECY-95-033 - Proposed Dollar per Person-Rem Conversion Factor; Response to SRM Concerning Issuance of Regulatory Analysis Guidelines of the U.S. Nuclear Regulatory Commission and SRM Concerning the Need for a Backfit Rule for Materials Licensees (RES-950225) (WITS-9100294)," September 18, 1995. [9803260148]
1690. Memorandum for All RES Employees from D. Morrison, "RES Office Letter No. 3C -- Procedures for Obtaining Regulatory Impact Analysis Review and Support," February 23, 1996. [9803260238]
1691. Memorandum to D. Morrison from W. Russell, "Third Supplemental User Need Request Regarding Potential for Loss of Emergency Core Cooling in a Boiling Water Reactor due to Clogging of the Suction Strainers by Loss-of-Coolant Accident Generated Debris," December 7, 1995. [9512140237]
1692. Memorandum to L. Shao from M. Marshall, "Expansion of Work Being Performed Under GSI-191, 'Assessment of Debris Accumulation on Pressurized Water Reactors Sump Performance,'" May 14, 1997. [9706060061]
1693. Memorandum to Chairman Jackson, et al., from J. Taylor, "Report on Survey of Refueling Practices," May 21, 1996. [9606030213]
1694. Memorandum to Chairman Jackson, et al., from J. Taylor, "Resolution of Spent Fuel Storage Pool Action Plan Issues," July 26, 1996. [9611180017]
1695. SECY-97-168, "Issuance for Public Comment of Proposed Rulemaking Package for Shutdown and Fuel Storage Pool Operation," July 30, 1997. [9708150168]
1696. Memorandum to L. Callan from J. Hoyle, "Staff Requirements - SECY-97-168 - Issuance for Public Comment of Proposed Rulemaking Package for Shutdown and Fuel Storage Pool Operation," December 11, 1997. [9712180222]
1697. Regulatory Guide 1.13, "Spent Fuel Storage Facility Design Basis," U.S. Nuclear Regulatory Commission, (Rev. 1) December 1975 [7907100079], (Draft Rev. 2) December 1981 [9803260142].
1698. Memorandum to E. Beckjord from E. Jordan, "Periodic Review of Low Priority Generic Safety Issues," April 7, 1995. [9701230176]
1699. Memorandum to D. Morrison from W. Russell, "Periodic Review of Low-Priority Generic Issues," April 11, 1996. [9604240169]
1700. Memorandum to L. Shao from D. Cool, "Submittal of Generic Safety Issues," April 12, 1996. [9605170029]

1701. Memorandum to F. Coffman from J. Piccone, "Status of NMSS Generic Safety Issues," December 15, 1997. [9712180068]
1702. NRC Information Notice 96-21, "Safety Concerns Related to the Design of the Door Interlock Circuit on Nucletron High-Dose Rate and Pulsed Dose Rate Remote Afterloading Brachytherapy Devices," April 10, 1996. [9604040106]
1703. NRC Information Notice 96-51, "Residual Contamination Remaining in Krypton-85 Handling System After Venting," September 11, 1996. [9609050281]
1704. NRC Information Notice 96-54, "Vulnerability of Stainless Steel to Corrosion When Sensitized," October 17, 1996. [9610100212]
1705. Memorandum to L. Shao from D. Cool, "Submittal of Generic Safety Issue," June 18, 1997. [9706240185]
1706. NRC Bulletin 97-01, "Potential for Erroneous Calibration, Dose Rate, or Radiation Exposure Measurements With Certain Victoreen Model 530 and 530Si Electrometer/ Dosemeters," April 30, 1997. [9704300128]
1707. Memorandum to R. Bangart, et al., from D. Cool, "Closeout Report for Bulletin 97-01, Potential for Erroneous Measurements With Certain Victoreen Electrometers," September 8, 1997. [9709170137]
1708. Memorandum to L. Shao from D. Cool, "Submittal of Generic Safety Issue," August 5, 1997. [9708130432]
1709. Memorandum to J. Craig from F. Combs, "Submittal of New Generic Issues for Tracking in the Generic Issues Management and Control System (GIMCS)," June 4, 1998. [9806090180]
1710. NRC Information Notice 96-70, "Year 2000 Effect on Computer System Software," December 24, 1996. [9612200319]
1711. NRC Information Notice 97-61, "U.S. Department of Health and Human Services Letter to Medical Device Manufacturers on the Year 2000 Problem," August 6, 1997. [9707310130]
1712. NRC Information Notice 97-91, "Recent Failures of Control Cables Used on Amersham Model 660 Posilock Radiography Systems," December 31, 1997 [9712310254], (Supplement 1) August 10, 1998 [9808050063].
1713. NRC Information Notice 96-52, "Cracked Insertion Rods on Troxler Model 3400 Series Portable Moisture Density Gauges," September 26, 1996. [9609200181]
1714. NRC Bulletin 97-02, "Puncture Testing of Shipping Packages Under 10 CFR Part 71," September 23, 1997. [9709180179]
1715. Memorandum to D. Morrison from T. Gwynn, "Periodic Review of Low-Priority Generic Safety Issues," April 16, 1997. [9909290132]

1716. Memorandum to T. Gwynn from T. Martin, "Periodic Review of Low-Priority Generic Safety Issues," July 13, 1998. [9909290134]
1717. UCRL-52156, "Advisability of Seismic Scram," Lawrence Livermore Laboratory, June 30, 1976. [8103270386]
1718. SECY-98-166, "Summary of Activities Related to Generic Safety Issues," July 6, 1998. [9807220129, 9807170226]
1719. NUREG-1631, "Source Disconnects Resulting from Radiography Drive Cable Failures," U.S. Nuclear Regulatory Commission, June 1998.
1720. Memorandum to J. Craig from F. Combs, "Closure of NMSS Generic Issues," October 13, 1998. [9810160185]
1721. NUREG-1536, "Standard Review Plan for Dry Cask Storage Systems," U.S. Nuclear Regulatory Commission, January 1997.
1722. Letter to E. Fuller (Sierra Nuclear Corporation) from M. Knapp (NRC), "Closure of Confirmatory Action Letter 97-7-001," July 22, 1998. [9807290363]
1723. Memorandum to J. Craig from F. Combs, "Submittal of New Generic Issues for Tracking in the Generic Issues Management and Control System (GIMCS)," July 23, 1998. [9807280039]
1724. SECY-98-001, "Staff Requirements Memorandum 951219A - Briefing on Mechanisms for Addressing Generic Safety Issues," January 2, 1998. [9801230188, 9801140109]
1725. Memorandum to E. Ten Eyck, et al., from C. Paperiello, "NMSS Policy and Procedures Letter 1-57, Rev. 1, 'NMSS Generic Issues Program,'" October 30, 1997. [9711050048]
1726. NUREG/CR-6538, "Evaluation of LOCA With Delayed Loop and Loop With Delayed LOCA Accident Scenarios," U.S. Nuclear Regulatory Commission, July 1997.
1727. Memorandum to W. Travers from A. Thadani, "Resolution of Generic Safety Issue (GSI) - 171, 'ESF Failure from LOOP Subsequent to LOCA,'" December 9, 1998. [9909290137]
1728. Letter to J. Birmingham, et al., (NRC) from W. Foster (The B&W Owners' Group), "Submittal of B&WOG Report 'Evaluation of Potential Boron Dilution following Small Break Loss of Coolant Accident,' 77-5002260-00, September 1998," September 11, 1998. [9809150094]
1729. Letter to W. Lyon (NRC) from J. Link (The B&W Owners' Group), "Transmittal of Report 'Status Report on Return to Criticality Following Small Break Loss of Coolant Accident,' June 1998, Document No. 47-5001848-00," June 15, 1998. [9806220211]
1730. Memorandum to A. Thadani from S. Collins, "Potential Need to Reprioritize/Reopen Aspects of Generic Safety Issue (GSI) 22 Pertaining to Boron Dilution Following Loss-of-Coolant Accidents," February 1, 1999. [9902160085]

1731. Memorandum for W. Russell from D. Morrison, "Prioritization of the NRR Action Plans Submitted to RES on February 13, 1996," June 24, 1996. [9606260260]
1732. Memorandum for W. Minners from E. Beckjord, "Generic Issue No. 165, 'Spring-Actuated Safety and Relief Valve Reliability,'" November 26, 1993. [9312090116]
1733. Memorandum to W. Travers from A. Thadani, "Closeout of Generic Safety Issue 165, Spring-Actuated Safety and Relief Valve Reliability," June 18, 1999.
1734. Memorandum for W. Travers from A. Thadani, "Resolution of Generic Safety Issue B-61, 'Analytically Derived Allowable Equipment Outage Periods,'" March 2, 1999. [9904050209]
1735. Regulatory Guide 1.177, "An Approach for Plant-Specific, Risk-Informed Decisionmaking: Technical Specifications," U.S. Nuclear Regulatory Commission, August 1998. [9809110028]
1736. Memorandum to M. Knapp from L. Shao, "Generic Issue No. 169, 'BWR MSIV Common Mode Failure Due to Loss of Accumulator Pressure,'" March 10, 1998. [9804070430]
1737. Memorandum to NRR Division Directors from D. Matthews, "Director's Quarterly Status Report," January 26, 1999. [9902040247]
1738. Memorandum to L. Shao from D. Morrison, "Generic Issue No. 171, 'ESF Failure from LOOP Subsequent to LOCA,'" June 16, 1995. [9507030081]
1739. Memorandum for J. Murphy from E. Beckjord, "Generic Issue No. 158, 'Performance of Safety-Related Power-Operated Valves Under Design Basis Conditions,'" January 26, 1994. [9402040031]
1740. Memorandum to J. Murphy from E. Beckjord, "Generic Issue No. 167, 'Hydrogen Storage Facility Separation,'" September 29, 1994. [9410250044]
1741. NUREG/CP-0123, "Proceedings of the Second NRC/ASME Symposium on Pump and Valve Testing," U.S. Nuclear Regulatory Commission, July 1992.
1742. AEOD/C603, "A Review of Motor-Operated Valve Performance," Office for Analysis and Evaluation of Operational Data, U.S. Nuclear Regulatory Commission, December 1986. [8612150167]
1743. Memorandum for Chairman Zech, et al., from V. Stello, "Case Study Report - A Review of Motor-Operated Valve Performance (AEOD/C603)," December 10, 1986.
1744. Memorandum to W. Travers from A. Thadani, "Closeout of Generic Safety Issue (GSI) - 158, 'Performance of Safety-Related Power-Operated Valves Under Design Basis Conditions,'" August 2, 1999. [9910040224]
1745. Memorandum for W. Minners from E. Beckjord, "Generic Issue No. 148, 'Smoke Control and Manual Fire-Fighting Effectiveness,'" August 26, 1992. [9208310325]

1746. Memorandum to A. Thadani from T. King, "Staff Review Guidance for Generic Safety Issue (GSI) 148, 'Smoke Control and Manual Fire-Fighting Effectiveness,'" July 22, 1999. [9907270312]
1747. Memorandum to G. Holahan from J. Wermeil, "Closeout of Core Performance Action Plan (TAC Nos. M91256, M91602)," February 16, 1999. [9902190260]
1748. Memorandum to Chairman Jackson, et al., from L. Callan, "Agency Program Plan for High-Burnup Fuel," July 6, 1998. [9808060096]
1749. Memorandum to D. Morrison from W. Russell, "Periodic Review of Low-Priority Generic Issues," April 11, 1996. [9604240169]
1750. Memorandum to B. Sheron from J. Craig, "Periodic Review of Low-Priority Generic Safety Issues," March 5, 1999. [9904060275]
1751. Letter to Seismic Qualification Advisory Committee (SQAC) and Meeting Attendees from G. Sliter and R. Vasudevan (EPRI), "Summary of the EPRI Seismic Equipment Qualification Research Coordination Meeting at ANCO Engineers, Inc., Los Angeles, California, September 19 & 20, 1984," October 10, 1984.
1752. NUREG/CR-5500, "Reliability Study: Westinghouse Reactor Protection System, 1984-1995," U.S. Nuclear Regulatory Commission, (Vol. 2) April 1999.
1753. Memorandum to D. Matthews from E. Rossi, "Issue of Final Report - System Reliability: Westinghouse Reactor Protection System, 1984-1995 (NUREG/CR-5500, Volume 2)," March 24, 1999. [9904060063]
1754. Memorandum for W. Minners from E. Beckjord, "Generic Issue No. 145, 'Actions to Reduce Common Cause Failures,'" February 11, 1992. [9203170332]
1755. NUREG/CR-6268, "Common-Cause Failure Database and Analysis System," U.S. Nuclear Regulatory Commission, (Vols. 1, 2, 3, and 4) June 1998.
1756. NRC Administrative Letter 98-04, "Availability of Common-Cause Failure Database," July 30, 1998. [9807240296]
1757. NRC Regulatory Issue Summary 99-03, "Resolution of Generic Issue 145, Actions to Reduce Common-Cause Failures," October 13, 1999. [9910060044]
1758. Memorandum to W. Travers from A. Thadani, "Resolution of Generic Safety Issue 145, 'Actions to Reduce Common Cause Failures,'" October 18, 1999.
1759. NUREG/CR-5640, "Overview and Comparison of U.S. Commercial Nuclear Power Plants," U.S. Nuclear Regulatory Commission, September 1990.
1760. NUREG/CR-5750, "Rates of Initiating Events at U.S. Nuclear Power Plants: 1987-1995," U.S. Nuclear Regulatory Commission, February 1999.

1761. Memorandum to A. Thadani from E. Beckjord, "Generic Issue 156-6.1, 'Pipe Break Effects on Systems and Components,'" October 31, 1994. [9412070254]
1762. Memorandum for J. Murphy from E. Beckjord, "Generic Issue No. 156.6.1, 'Pipe Break Effects on Systems and Components,'" April 29, 1994. [9406200193]
1763. Memorandum to W. Travers from A. Thadani, "Closeout of Generic Safety Issue 23, 'Reactor Coolant Pump Seal Failure,'" November 8, 1999. [ML993370509]
1764. Memorandum to W. Travers from A. Thadani, "Closeout of Generic Safety Issue 190, 'Fatigue Evaluation of Metal Components for 60-Year Plant Life,'" December 26, 1999.
1765. Memorandum to W. Travers from S. Collins, "Closeout of Generic Safety Issue B-55, 'Improved Reliability of Target Rock Safety Relief Valves,'" December 17, 1999.
1766. Memorandum to W. Travers from A. Thadani, "Proposed Resolution of Generic Issue B-17, 'Criteria for Safety-Related Operator Actions,'" March 27, 2000. [ML003695959]
1767. NRC Regulatory Issue Summary 2000-02, "Closure of Generic Safety Issue 23, Reactor Coolant Pump Seal Failure," February 15, 2000. [ML003680402]
1768. NRC Regulatory Issue Summary 2000-03, "Resolution of Generic Safety Issue 158: Performance of Safety-Related Power-Operated Valves Under Design Basis Conditions," March 15, 2000. [ML003686003]
1769. NRC Regulatory Issue Summary 2000-05, "Resolution of Generic Safety Issue 165, Spring-Actuated Safety and Relief Valve Reliability," March 16, 2000. [ML003689694]
1770. Memorandum for W. Russell from C. Miller, "Licensee Offsite Communication Capabilities; Results of Information Gathering Using Temporary Instruction," September 26, 1996.
1771. Memorandum to C. Rossi from D. Cool, "Status of NMSS Issues in the Generic Issue Management and Control System," June 25, 1999. [9907010194]
1772. Memorandum to C. Rossi from D. Cool, "Closure of NMSS Generic Issue," May 18, 1999.
1773. Memorandum for E. Beckjord from J. Milhoan, "Periodic Review of Low-Priority Generic Safety Issues," June 4, 1993.
1774. Memorandum to A. Thadani from S. Bahadur, "Reprioritization of GSI-71, 'Failure of Resin Demineralizer Systems and Their Effects on Nuclear Power Plant Safety,'" December 20, 2000. [ML003779066]
1775. Memorandum to A. Thadani from M. Mayfield, "Closeout of GSI-152, 'Design Basis for Valves That Might Be Subjected to Significant Blowdown Loads,'" April 4, 2001. [ML010740024]
1776. Memorandum to A. Thadani from J. Rosenthal, "Initial Screening of Candidate Generic Issue 187, 'The Potential Impact of Postulated Cesium Concentration on Equipment Qualification in the Containment Sump,'" April 30, 2001. [ML011210348]

1777. Memorandum to A. Thadani from B. Sheron, "Proposed Generic Safety Issue - The Potential Impact of Postulated Cesium Concentration on Equipment Qualification in the Containment Sump," December 16, 1999. [ML993610109]
1778. Memorandum to W. Travers from A. Thadani, "Closure of Generic Issue 170, Reactivity Transients and Fuel Damage Criteria for High Burnup Fuel," May 4, 2001. [ML011280414]
1779. NUREG/CR-6395, "Enhanced Prioritization of Generic Safety Issue 156-6.1 Pipe Break Effects on Systems and Components Inside Containment," (Draft) September 1999. [ML010460480]
1780. Memorandum to F. Eltawila from D. Cool, "Submittal of Generic Issues for Tracking in the Generic Issue Management Control System (GIMCS)," November 14, 2000. [ML003763127]
1781. NRC Information Notice 99-26, "Safety and Economic Consequences of Misleading Marketing Information," August 24, 1999. [9908180183]
1782. NRC Information Notice 99-09, "Problems Encountered When Manually Editing Treatment Data on the Nucletron MicroSelectron-HDR (New) Model 105.999," March 24, 1999. [9903190227]
1783. NRC Information Notice 99-23, "Safety Concerns Related to Repeated Control Unit Failures of the Nucletron Classic Model High-Dose-Rate Remote Afterloading Brachytherapy Devices," July 6, 1999. [9907010001]
1784. Memorandum to W. Travers from W. Kane, "Closure of Two NMSS Generic Issues," January 26, 2001. [ML010240165]
1785. Memorandum to W. Travers from W. Kane, "Closure of NMSS Generic Issue Relating to Gamma Stereotactic Radiosurgery," February 12, 2001. [ML010390357]
1786. NRC Information Notice 2000-22, "Medical Misadministrations Caused by Human Errors Involving Gamma Stereotactic Radiosurgery (Gamma Knife)," December 18, 2000. [ML003761619]
1787. Memorandum to F. Eltawila from D. Cool, "NMSS Input for Second Quarter FY-2001 Update of the Generic Issue Management Control System," April 12, 2001. [ML011000117]

APPENDIX B  
APPLICABILITY OF NUREG-0933 ISSUES TO OPERATING AND FUTURE REACTOR PLANTS

This appendix contains a listing of those residual GSIs that are applicable to operating and future reactor plants and includes: issues that have been resolved with requirements [I, NOTE 3(a)]; USI, HIGH- and MEDIUM-priority issues scheduled for resolution; nearly-resolved issues scheduled for resolution (NOTES 1 and 2); and issues that are scheduled for prioritization (NOTE 4). The priority designations for all issues are consistent with those listed in Table II of the Introduction. In accordance with 10 CFR 52.47(a)(1)(iv), any future application for design certification must contain proposed technical resolutions for the issues in this listing that are designated USI, HIGH, MEDIUM, NOTE 1, and NOTE 2. (In July 1998, the priority categories NOTES 1 and 2 were eliminated and all GSIs in these categories were given a HIGH priority ranking.<sup>1718</sup>) Also included in this listing are those GSIs that were either prioritized or resolved with no impact on operating reactor plants but contain recommendations for future reactor plants (NOTE 6).

Legend

|          |   |
|----------|---|
| NOTES: 1 | - Possible Resolution Identified for Evaluation (Discontinued 07-06-98)   |
| 2        | - Resolution Available [Documented in NUREG, NRC Memorandum, SER or equivalent] (Discontinued 07-06-98)                       |
| 3(a)     | - Resolution Resulted in the Establishment of New Regulatory Requirements [Rule, Regulatory Guide, SRP Change, or equivalent] |
| 4        | - Issue to be Prioritized in the Future   |
| 6        | - New Requirements for Future Plants Recommended  |
| B&W      | - Babcock & Wilcox Company  |
| CE       | - Combustion Engineering Company  |
| GE       | - General Electric Company  |
| HIGH     | - High Safety Priority  |
| I        | - Resolved TMI Action Plan Item with Implementation of Resolution Mandated by NUREG-0737                                      |
| MEDIUM   | - Medium Safety Priority  |
| MPA      | - Multiplant Action   |
| NA       | - Not Applicable  |
| TBD      | - To Be Determined  |
| USI      | - Unresolved Safety Issue   |
| <u>W</u> | - Westinghouse Electric Corporation   |

Appendix B (Continued)

06/30/01

| Action Plan Item/Issue No. | Title | Safety Priority/Status | Affected NSSS Vendor |     | Operating Plants- MPA No | Operating Plants - Effective Date | Future Plants- Effective Date |
|----------------------------|-------|------------------------|----------------------|-----|--------------------------|-----------------------------------|-------------------------------|
|                            |       |                        | BWR                  | PWR |                          |                                   |                               |

TMI ACTION PLAN ITEMS

I.A. OPERATING PERSONNEL

I.A.1 Operating Personnel and Staffing

|         |  |           |     |     |      |          |          |
|---------|--|-----------|-----|-----|------|----------|----------|
| I.A.1.2 | Shift Technical Advisor                | I         | All | All | F-01 | 09/13/79 | 09/27/79 |
| I.A.1.2 | Shift Supervisor Administrative Duties | I         | All | All |      | 09/13/79 | 09/27/79 |
| I.A.1.3 | Shift Manning                          | I         | All | All | F-02 | 07/31/80 | 06/26/80 |
| I.A.1.4 | Long-Term Upgrading                    | NOTE 3(a) | All | All |      | 04/28/83 | 04/28/83 |

I.A.2 Training and Qualifications of Operating Personnel

|            |  |           |     |     |      |          |          |
|------------|--|-----------|-----|-----|------|----------|----------|
| I.A.2.1    | Immediate Upgrading of Operator and Senior Operator Training and Qualifications                          | -         | -   | -   | -    | -        | -        |
| I.A.2.1(1) | Qualifications - Experience  | I         | All | All | F-03 | 03/28/80 | 03/28/80 |
| I.A.2.1(2) | Training   | I         | All | All | F-03 | 03/28/80 | 03/28/80 |
| I.A.2.1(3) | Facility Certification of Competence and Fitness of Applicants for Operator and Senior Operator Licenses | I         | All | All | F-03 | 03/28/80 | 03/28/80 |
| I.A.2.3    | Administration of Training Programs  | I         | All | All |      | 03/28/80 | 03/28/80 |
| I.A.2.6    | Long-Term Upgrading of Training and Qualifications   | -         | -   | -   | -    | -        | -        |
| I.A.2.6(1) | Revise Regulatory Guide 1.8  | NOTE 3(a) | All | All |      | TBD      | 05--/87  |

I.A.3 Licensing and Requalification of Operating Personnel

|         |   |   |     |     |  |          |          |
|---------|---|---|-----|-----|--|----------|----------|
| I.A.3.1 | Revise Scope of Criteria for Licensing Examinations | I | All | All |  | 03/28/80 | 03/28/80 |
|---------|---|---|-----|-----|--|----------|----------|

I.A.4 Simulator Use and Development

|            |   |           |     |     |   |          |          |
|------------|---|-----------|-----|-----|---|----------|----------|
| I.A.4.1    | Initial Simulator Improvement                 | -         | -   | -   | - | -        | -        |
| I.A.4.1(2) | Interim Changes in Training Simulators        | NOTE 3(a) | All | All |   | 04--/81  | 03/28/81 |
| I.A.4.2    | Long-Term Training Simulator Upgrade          | -         | -   | -   | - | -        | -        |
| I.A.4.2(1) | Research on Training Simulators               | NOTE 3(a) | All | All |   | 04--/87  | 04--/87  |
| I.A.4.2(2) | Upgrade Training Simulator Standards          | NOTE 3(a) | All | All |   | 04--/81  | 04--/81  |
| I.A.4.2(3) | Regulatory Guide on Training Simulators       | NOTE 3(a) | All | All |   | 04--/81  | 04--/81  |
| I.A.4.2(4) | Review Simulators for Conformance to Criteria | NOTE 3(a) | All | All |   | 03/25/87 | 03/25/87 |

A.B-2

NUREG-0933

Revision 16

Appendix B (Continued)

06/30/01

| Action Plan Item/Issue No. | Title | Safety Priority/Status | Affected NSSS Vendor |     | Operating Plants- MPA No | Operating Plants - Effective Date | Future Plants- Effective Date |
|----------------------------|-------|------------------------|----------------------|-----|--------------------------|-----------------------------------|-------------------------------|
|                            |       |                        | BWR                  | PWR |                          |                                   |                               |

I.C OPERATING PROCEDURES

|          |  |           |     |     |      |          |          |
|----------|--|-----------|-----|-----|------|----------|----------|
| I.C.1    | Short-Term Accident Analysis and Procedures Revision   | -         | -   | -   | -    | -        | -        |
| I.C.1(1) | Small Break LOCAs  | I         | All | All |      | 09/13/79 | 09/13/79 |
| I.C.1(2) | Inadequate Core Cooling  | I         | All | All | F-04 | 09/13/79 | 09/13/79 |
| I.C.1(3) | Transients and Accidents   | I         | All | All | F-05 | 09/13/79 | 09/27/79 |
| I.C.2    | Shift and Relief Turnover Procedures   | I         | All | All |      | 09/13/79 | 09/27/79 |
| I.C.3    | Shift Supervisor Responsibilities  | I         | All | All |      | 09/13/79 | 09/27/79 |
| I.C.4    | Control Room Access  | I         | All | All |      | 09/13/79 | 09/27/79 |
| I.C.5    | Procedures for Feedback of Operating Experience to Plant Staff                               | I         | All | All | F-06 | 05/07/80 | 06/26/80 |
| I.C.6    | Procedures for Verification of Correct Performance of Operating Activities                   | I         | All | All | F-07 | 10/31/80 | 10/31/80 |
| I.C.7    | NSSS Vendor Review of Procedures   | I         | All | All |      | NA       | 06/26/80 |
| I.C.8    | Pilot Monitoring of Selected Emergency Procedures for Near-Term Operating License Applicants | I         | All | All |      | NA       | 06/26/80 |
| I.C.9    | Long-Term Program Plan for Upgrading of Procedures   | NOTE 3(a) | All | All |      | 09/13/79 | 06/1-85  |

A.B-3

I.D CONTROL ROOM DESIGN

|          |  |           |     |     |      |          |          |
|----------|--|-----------|-----|-----|------|----------|----------|
| I.D.1    | Control Room Design Reviews                    | I         | All | All | F-08 | 06/26/80 | 06/26/80 |
| I.D.2    | Plant Safety Parameter Display Console         | I         | All | All | F-09 | 06/26/80 | 06/26/80 |
| I.D.5    | Improved Control Room Instrumentation Research | -         | -   | -   | -    | -        | -        |
| I.D.5(2) | Plant Status and Post-Accident Monitoring      | NOTE 3(a) | All | All |      | NA       | 12/1-80  |

I.F QUALITY ASSURANCE

|          |   |           |     |     |   |    |         |
|----------|---|-----------|-----|-----|---|----|---------|
| I.F.2    | Develop More Detailed QA Criteria   | -         | -   | -   | - | -  | -       |
| I.F.2(2) | Include QA Personnel in Review and Approval of Plant Procedures                                   | NOTE 3(a) | All | All |   | NA | 07/1-81 |
| I.F.2(3) | Include QA Personnel in All Design, Construction, Installation, Testing, and Operation Activities | NOTE 3(a) | All | All |   | NA | 07/1-81 |
| I.F.2(6) | Increase the Size of Licensees' QA Staff  | NOTE 3(a) | All | All |   | NA | 07/1-81 |
| I.F.2(9) | Clarify Organizational Reporting Levels for the QA Organization                                   | NOTE 3(a) | All | All |   | NA | 07/1-81 |

I.G PREOPERATIONAL AND LOW-POWER TESTING

|       |                       |           |     |     |  |    |          |
|-------|-----------------------|-----------|-----|-----|--|----|----------|
| I.G.1 | Training Requirements | I         | All | All |  | NA | 06/26/80 |
| I.G.2 | Scope of Test Program | NOTE 3(a) | All | All |  | NA | 07/1-81  |

NUREG-0933

Revision 16

Appendix B (Continued)

06/30/01

| Action Plan Item/Issue No.   | Title   | Safety Priority/Status | Affected NSSS Vendor |     | Operating Plants- MPA No | Operating Plants - Effective Date | Future Plants- Effective Date |
|--|---|------------------------|----------------------|-----|--------------------------|-----------------------------------|-------------------------------|
|  |   |                        | BWR                  | PWR |                          |                                   |                               |
| <b><u>II.B</u></b>   |   |                        |                      |     |                          |                                   |                               |
| <b><u>CONSIDERATION OF DEGRADED OR MELTED CORES IN SAFETY REVIEW</u></b> |   |                        |                      |     |                          |                                   |                               |
| II.B.1   | Reactor Coolant System Vents  | I                      | All                  | All | F-10                     | 09/13/79                          | 09/27/79                      |
| II.B.2   | Plant Shielding to Provide Access to Vital Areas and Protect Safety Equipment for Post-Accident Operation | I                      | All                  | All | F-11                     | 09/13/79                          | 09/27/79                      |
| II.B.3   | Post-Accident Sampling  | I                      | All                  | All | F-12                     | 09/13/79                          | 09/27/79                      |
| II.B.4   | Training for Mitigating Core Damage   | I                      | All                  | All | F-13                     | 03/28/80                          | 03/28/80                      |
| ii.B.6   | Risk Reduction for Operating Reactors at Sites with High Population Densities                             | NOTE 3(a)              | All                  | All |                          | TBD                               | NA                            |
| II.B.8   | Rulemaking Proceeding on Degraded Core Accidents  | NOTE 3(a)              | All                  | All |                          | TBD                               | 01/25/85                      |
| <b><u>II.D</u></b>   |   |                        |                      |     |                          |                                   |                               |
| <b><u>REACTOR COOLANT SYSTEM RELIEF AND SAFETY VALVES</u></b>            |   |                        |                      |     |                          |                                   |                               |
| II.D.1   | Testing Requirements  | I                      | All                  | All | F-14                     | 09/13/79                          | 09/27/79                      |
| II.D.3   | Relief and Safety Valve Position Indication   | I                      | All                  | All |                          | 07/21/79                          | 09/27/79                      |
| <b><u>II.E</u></b>   |   |                        |                      |     |                          |                                   |                               |
| <b><u>SYSTEM DESIGN</u></b>  |   |                        |                      |     |                          |                                   |                               |
| <b><u>II.E.1</u></b>   |   |                        |                      |     |                          |                                   |                               |
| <b><u>Auxiliary Feedwater System</u></b>                                 |   |                        |                      |     |                          |                                   |                               |
| II.E.1.1   | Auxiliary Feedwater System Evaluation   | I                      | NA                   | All | F15                      | 03/10/80                          | 03/10/80                      |
| II.E.1.2   | Auxiliary Feedwater System Automatic Initiation and Flow Indication                                       | I                      | NA                   | All | F-16, F-17               | 09/13/79                          | 09/27/79                      |
| II.E.1.3   | Update Standard Review Plan and Develop Regulatory Guide  | NOTE 3(a)              | All                  | All |                          | NA                                | 07/--/81                      |
| <b><u>II.E.3</u></b>   |   |                        |                      |     |                          |                                   |                               |
| <b><u>Decay Heat Removal</u></b>   |   |                        |                      |     |                          |                                   |                               |
| II.E.3.1   | Reliability of Power Supplies for Natural Circulation   | I                      | NA                   | All |                          | 09/13/79                          | 09/27/79                      |
| <b><u>II.E.4</u></b>   |   |                        |                      |     |                          |                                   |                               |
| <b><u>Containment Design</u></b>   |   |                        |                      |     |                          |                                   |                               |
| II.E.4.1   | Dedicated Penetrations  | I                      | All                  | All | F-18                     | 09/13/79                          | 09/27/79                      |
| II.E.4.2   | Isolation Dependability   | I                      | All                  | All | F-19                     | 09/13/79                          | 09/27/79                      |
| II.E.4.4   | Purging   | -                      | -                    | -   | -                        | -                                 | -                             |
| II.E.4.4(1)  | Issue Letter to Licensees Requesting Limited Purging  | NOTE 3(a)              | All                  | All |                          | 11/28/78                          | NA                            |
| II.E.4.4(2)  | Issue Letter to Licensees Requesting Information on Isolation Letter                                      | NOTE 3(a)              | All                  | All |                          | 10/22/79                          | NA                            |
| II.E.4.4(3)  | Issue Letter to Licensees on Valve Operability  | NOTE 3(a)              | All                  | All |                          | 09/27/79                          | NA                            |

A.B-4

NUREG-0933

Revision 16

Appendix B (Continued)

06/30/01

| Action Plan Item/Issue No. | Title  | Safety Priority/Status | Affected NSSS Vendor |     | Operating Plants- MPA No               | Operating Plants - Effective Date | Future Plants- Effective Date |
|----------------------------|--|------------------------|----------------------|-----|--|-----------------------------------|-------------------------------|
|                            |  |                        | BWR                  | PWR |  |                                   |                               |
| <u>II.E.5</u>              | <u>Design Sensitivity of B&amp;W Reactors</u>  |                        |                      |     |  |                                   |                               |
| II.E.5.1                   | Design Evaluation  | NOTE 3(a)              | NA                   | B&W |  |                                   |                               |
| II.E.5.2                   | B&W Reactor Transient Response Task Force  | NOTE 3(a)              | NA                   | B&W |  |                                   |                               |
| <u>II.E.6</u>              | <u>In Situ Testing of Valve</u>  |                        |                      |     |  |                                   |                               |
| II.E.6.1                   | Test Adequacy Study  | NOTE 3(a)              | All                  | All |  | 06/--/89                          | 06/--/89                      |
| <u>II.F</u>                | <u>INSTRUMENTATION AND CONTROLS</u>  |                        |                      |     |  |                                   |                               |
| II.F.1                     | Additional Accident Monitoring Instrumentation   | I                      | All                  | All | F-20, F-21<br>F-22, F-23<br>F-24, F-25 | 09/13/79                          | 09/27/79                      |
| II.F.2                     | Identification of and Recovery from Conditions Leading to Inadequate Core Cooling                                  | I                      | All                  | All | F-26                                   | 07/02/79                          | 09/27/79                      |
| II.F.3                     | Instruments for Monitoring Accident Conditions   | NOTE 3(a)              | All                  | All |  | NA                                | 12/--/80                      |
| <u>II.G</u>                | <u>ELECTRICAL POWER</u>  |                        |                      |     |  |                                   |                               |
| II.G.1                     | Power Supplies for Pressurizer Relief Valves, Block Valves, and Level Indicators                                   | I                      | NA                   | All |  | 09/13/79                          | 09/27/79                      |
| <u>II.J</u>                | <u>GENERAL IMPLICATIONS OF TMI FOR DESIGN AND CONSTRUCTION ACTIVITIES</u>  |                        |                      |     |  |                                   |                               |
| <u>II.J.4</u>              | <u>Revise Deficiency Reporting Requirements</u>  |                        |                      |     |  |                                   |                               |
| II.J.4.1                   | Revise Deficiency Reporting Requirements   | NOTE 3(a)              | All                  | All |  | 07/31/91                          | 07/31/91                      |
| <u>II.K</u>                | <u>MEASURES TO MITIGATE SMALL-BREAK LOSS-OF-COOLANT ACCIDENTS AND LOSS-OF-FEEDWATER ACCIDENTS</u>                  |                        |                      |     |  |                                   |                               |
| II.K.1                     | IE Bulletins   | -                      | -                    | -   | -                                      | -                                 | -                             |
| II.K.1(1)                  | Review TMI-2 PNs and Detailed Chronology of the TMI-2 Accident   | NOTE 3(a)              | All                  | All |  | 03/31/80                          | NA                            |
| II.K.1(2)                  | Review Transients Similar to TMI-2 That Have Occurred at Other Facilities and NRC Evaluation of Davis-Besse Event  | NOTE 3(a)              | NA                   | B&W |  | 03/31/80                          | NA                            |
| II.K.1(3)                  | Review Operating Procedures for Recognizing, Preventing, and Mitigating Void Formation in Transients and Accidents | NOTE 3(a)              | NA                   | All |  | 03/31/80                          | NA                            |

A.B-5

NUREG-0933

Revision 16

Appendix B (Continued)

06/30/01

| Action Plan Item/Issue No. | Title   | Safety Priority/Status | Affected NSSS Vendor |              | Operating Plants- MPA No | Operating Plants - Effective Date | Future Plants- Effective Date |
|----------------------------|---|------------------------|----------------------|--------------|--------------------------|-----------------------------------|-------------------------------|
|                            |   |                        | BWR                  | PWR          |                          |                                   |                               |
| II.K.1(4)                  | Review Operating Procedures and Training Instructions   | NOTE 3(a)              | All                  | All          |                          | 03/31/80                          | NA                            |
| II.K.1(5)                  | Safety-Related Valve Position Description   | NOTE 3(a)              | All                  | All          |                          | 03/31/80                          | 03/31/80                      |
| II.K.1(6)                  | Review Containment Isolation Initiation Design and Procedures   | NOTE 3(a)              | All                  | All          |                          | 03/31/80                          | NA                            |
| II.K.1(7)                  | Implement Positive Position Controls on Valves That Could Compromise or Defeat AFW Flow   | NOTE 3(a)              | NA                   | B&W          |                          | 03/31/80                          | NA                            |
| II.K.1(8)                  | Implement Procedures That Assure Two Independent 100% AFW Flow Paths  | NOTE 3(a)              | NA                   | B&W          |                          | 03/31/80                          | NA                            |
| II.K.1(9)                  | Review Procedures to Assure That Radioactive Liquids and Gases Are Not Transferred out of Containment Inadvertently   | NOTE 3(a)              | All                  | All          |                          | 03/31/80                          | NA                            |
| II.K.1(10)                 | Review and Modify Procedures for Removing Safety-Related Systems from Service   | NOTE 3(a)              | All                  | All          |                          | 03/31/80                          | 03/31/80                      |
| II.K.1(11)                 | Make All Operating and Maintenance Personnel Aware of the Seriousness and Consequences of the Erroneous Actions Leading up to, and in Early Phases of, the TMI-2 Accident | NOTE 3(a)              | All                  | All          |                          | 03/31/80                          | NA                            |
| II.K.1(12)                 | One Hour Notification Requirement and Continuous Communications Channels  | NOTE 3(a)              | All                  | All          |                          |                                   | NA                            |
| II.K.1(13)                 | Propose Technical Specification Changes Reflecting Implementation of All Bulletin Items   | NOTE 3(a)              | All                  | All          |                          | 01/01/81                          | 01/01/81                      |
| II.K.1(14)                 | Review Operating Modes and Procedures to Deal with Significant Amounts of Hydrogen  | NOTE 3(a)              | GE                   | CE, <u>W</u> |                          | 03/31/80                          | NA                            |
| II.K.1(15)                 | For Facilities with Non-Automatic AFW Initiation, Provide Dedicated Operator in Continuous Communication with CR to Operate AFW   | NOTE 3(a)              | NA                   | CE, <u>W</u> |                          | NA                                |                               |
| II.K.1(16)                 | Implement Procedures That Identify PRZ PORV "Open" Indications and That Direct Operator to Close Manually at "Reset" Setpoint   | NOTE 3(a)              | NA                   | CE, <u>W</u> |                          | NA                                |                               |
| II.K.1(17)                 | Trip PZR Level Bistable so That PZR Low Pressure Will Initiate Safety Injection   | NOTE 3(a)              | NA                   | <u>W</u>     |                          |                                   |                               |
| II.K.1(18)                 | Develop Procedures and Train Operators on Methods of Establishing and Maintaining Natural Circulation   | NOTE 3(a)              | NA                   | B&W          |                          | NA                                |                               |
| II.K.1(19)                 | Describe Design and Procedure Modifications to Reduce Likelihood of Automatic PZR PORV Actuation in Transients  | NOTE 3(a)              | NA                   | B&W          |                          | 03/31/80                          | NA                            |
| II.K.1(20)                 | Provide Procedures and Training to Operators for Prompt Manual Reactor Trip for LOFW, TT, MSIV Closure, LOOP, LOSG Level, and LO PZR Level                                | NOTE 3(a)              | NA                   | B&W          |                          | 03/31/80                          | 03/31/80                      |

A.B-6

NUREG-0933

Revision 16

Appendix B (Continued)

| Action Plan Item/Issue No. | Title  | Safety Priority/Status | Affected NSSS Vendor |     | Operating Plants- MPA No | Operating Plants - Effective Date | Future Plants- Effective Date |
|----------------------------|--|------------------------|----------------------|-----|--------------------------|-----------------------------------|-------------------------------|
|                            |  |                        | BWR                  | PWR |                          |                                   |                               |
| II.K.1(21)                 | Provide Automatic Safety-Grade Anticipatory Reactor Trip for LOFW, TT, or Significant Decrease in SG Level                 | NOTE 3(a)              | NA                   | B&W |                          | 03/31/80                          | 03/31/80                      |
| II.K.1(22)                 | Describe Automatic and Manual Actions for Proper Functioning of Auxiliary Heat Removal Systems When FW System Not Operable | NOTE 3(a)              | All                  | NA  |                          | 03/31/80                          | 03/31/80                      |
| II.K.1(23)                 | Describe Uses and Types of RV Level Indication for Automatic and Manual Initiation Safety Systems                          | NOTE 3(a)              | All                  | NA  |                          | 03/31/80                          | 03/31/80                      |
| II.K.1(24)                 | Perform LOCA Analyses for a Range of Small-Break Sizes and a Range of Time Lapses Between Reactor Trip and RCP Trip        | NOTE 3(a)              | NA                   | All |                          | NA                                |                               |
| II.K.1(25)                 | Develop Operator Action Guidelines   | NOTE 3(a)              | NA                   | All |                          | NA                                |                               |
| II.K.1(26)                 | Revise Emergency Procedures and Train ROs and SROs   | NOTE 3(a)              | NA                   | All |                          | NA                                |                               |
| II.K.1(27)                 | Provide Analyses and Develop Guidelines and Procedures for Inadequate Core Cooling Conditions                              | NOTE 3(a)              | NA                   | All |                          | NA                                |                               |
| II.K.1(28)                 | Provide Design That Will Assure Automatic RCP Trip for All Circumstances Where Required                                    | NOTE 3(a)              | NA                   | All |                          | 01/01/81                          | 01/01/82                      |
| II.K.2                     | Commission Orders on B&W Plants  | -                      | -                    | -   |                          | -                                 | -                             |
| II.K.2(1)                  | Upgrade Timeliness and Reliability of AFW System   | NOTE 3(a)              | NA                   | B&W |                          | NA                                |                               |
| II.K.2(2)                  | Procedures and Training to Initiate and Control AFW Independent of Integrated Control System                               | NOTE 3(a)              | NA                   | B&W |                          | NA                                |                               |
| II.K.2(3)                  | Hard-Wired Control-Grade Anticipatory Reactor Trips  | NOTE 3(a)              | NA                   | B&W |                          | NA                                |                               |
| II.K.2(4)                  | Small-Break LOCA Analysis, Procedures and Operator Training  | NOTE 3(a)              | NA                   | B&W |                          | NA                                |                               |
| II.K.2(5)                  | Complete TMI-2 Simulator Training for All Operators  | NOTE 3(a)              | NA                   | B&W |                          | NA                                |                               |
| II.K.2(6)                  | Reevaluate Analysis for Dual-Level Setpoint Control  | NOTE 3(a)              | NA                   | B&W |                          | NA                                |                               |
| II.K.2(7)                  | Reevaluate Transient of September 24, 1977   | NOTE 3(a)              | NA                   | B&W |                          | NA                                |                               |
| II.K.2(9)                  | Analysis and Upgrading of Integrated Control System  | I                      | NA                   | B&W | F-27                     | 01/01/81                          | 01/01/81                      |
| II.K.2(10)                 | Hard-Wired Safety-Grade Anticipatory Reactor Trips   | I                      | NA                   | B&W | F-28                     | 01/01/81                          | 01/01/81                      |
| II.K.2(11)                 | Operator Training and Drilling   | I                      | NA                   | B&W | F-29                     | 01/01/81                          | 01/01/81                      |
| II.K.2(13)                 | Thermal-Mechanical Report on Effect of HPI on Vessel Integrity for Small-Break LOCA With No AFW                            | I                      | NA                   | B&W | F-30                     | 01/01/81                          | 01/01/81                      |
| II.K.2(14)                 | Demonstrate That Predicted Lift Frequency of PORVs and SVs Is Acceptable   | I                      | NA                   | B&W | F-31                     | 01/01/81                          | 01/01/81                      |
| II.K.2(15)                 | Analysis of Effects of Slug Flow on Once-Through Steam Generator Tubes After Primary System Voiding                        | I                      | NA                   | B&W |                          | 06/01/80                          | 06/01/80                      |
| II.K.2(16)                 | Impact of RCP Seal Damage Following Small-Break LOCA With Loss of Offsite Power  | I                      | NA                   | B&W | F-32                     | 06/01/80                          | 06/01/80                      |

06/30/01

A.B-7

NUREG-0933

Revision 16

Appendix B (Continued)

| 06/30/01   | Action Plan Item/Issue No. | Title   | Safety Priority/Status | Affected NSSS Vendor |          | Operating Plants- MPA No | Operating Plants - Effective Date | Future Plants- Effective Date |
|------------|----------------------------|---|------------------------|----------------------|----------|--------------------------|-----------------------------------|-------------------------------|
|            |                            |   |                        | BWR                  | PWR      |                          |                                   |                               |
|            | II.K.2(17)                 | Analysis of Potential Voiding in RCS During Anticipated Transients  | I                      | NA                   | B&W      | F-33                     | NA                                |                               |
|            | II.K.2(19)                 | Benchmark Analysis of Sequential AFW Flow to Once-Through Steam Generator                                       | I                      | NA                   | B&W      | F-34                     | 01/01/81                          | NA                            |
|            | II.K.2(20)                 | Analysis of Steam Response to Small-Break LOCA That Causes System Pressure to Exceed PORV Setpoint              | I                      | NA                   | B&W      | F-35                     | 01/01/81                          | NA                            |
|            | II.K.2(21)                 | LOFT L3-1 Predictions   | NOTE 3(a)              | NA                   | B&W      |                          | NA                                |                               |
|            | II.K.3                     | Final Recommendations of Bulletins and Orders Task Force  | -                      | -                    | -        | -                        | -                                 | -                             |
|            | II.K.3(1)                  | Install Automatic PORV Isolation System and Perform Operational Test  | I                      | NA                   | All      | F-36                     | 07/01/81                          | 07/01/81                      |
|            | II.K.3(2)                  | Report on Overall Safety Effect of PORV Isolation System  | I                      | NA                   | All      | F-37                     | 01/01/81                          | 01/01/81                      |
|            | II.K.3(3)                  | Report Safety and Relief Valve Failures Promptly and Challenges Annually  | I                      | All                  | All      | F-38                     | 04/01/80                          | 04/01/80                      |
|            | II.K.3(5)                  | Automatic Trip of Reactor Coolant Pumps   | I                      | NA                   | All      | F-39, G-01               | 01/01/81                          | 01/01/81                      |
| A.B-8      | II.K.3(7)                  | Evaluation of PORV Opening Probability During Overpressure Transient  | I                      | NA                   | B&W      |                          | 01/01/81                          | 01/01/81                      |
|            | II.K.3(9)                  | Proportional Integral Derivative Controller Modification  | I                      | NA                   | <u>W</u> | F-40                     | 07/01/80                          | 07/01/80                      |
|            | II.K.3(10)                 | Anticipatory Trip Modification Proposed by Some Licensees to Confine Range of Use to High Power Levels          | I                      | NA                   | <u>W</u> | F-41                     |                                   |                               |
|            | II.K.3(11)                 | Control Use of PORV Supplied by Control Components, Inc. Until Further Review Complete                          | I                      | All                  | All      |                          |                                   |                               |
|            | II.K.3(12)                 | Confirm Existence of Anticipatory Trip Upon Turbine Trip  | I                      | NA                   | <u>W</u> | F-42                     | 07/01/80                          | 07/01/80                      |
|            | II.K.3(13)                 | Separation of HPCI and RCIC System Initiation Levels  | I                      | GE                   | NA       | F-43                     | 10/01/80                          | 10/01/80                      |
|            | II.K.3(14)                 | Isolation of Isolation Condensers on High Radiation   | I                      | GE                   | NA       | F-44                     | 01/01/81                          | NA                            |
|            | II.K.3(15)                 | Modify Break Detection Logic to Prevent Spurious Isolation of HPCI and RCIC Systems                             | I                      | GE                   | NA       | F-45                     | 01/01/81                          | 01/01/81                      |
|            | II.K.3(16)                 | Reduction of Challenges and Failures of Relief Valves - Feasibility Study and System Modification               | I                      | GE                   | NA       | F-46                     | 01/01/81                          | 01/01/81                      |
|            | II.K.3(17)                 | Report on Outage of ECC Systems - Licensee Report and Technical Specification Changes                           | I                      | GE                   | NA       | F-47                     | 01/01/81                          | 01/01/81                      |
|            | II.K.3(18)                 | Modification of ADS Logic - Feasibility Study and Modification for Increased Diversity for Some Event Sequences | I                      | GE                   | NA       | F-48                     | 01/01/81                          | 01/01/81                      |
|            | II.K.3(19)                 | Interlock on Recirculation Pump Loops   | I                      | GE                   | NA       | F-49                     | 01/01/81                          | NA                            |
| NUREG-0933 | II.K.3(20)                 | Loss of Service Water for Big Rock Point  | I                      | GE                   | NA       |                          | 01/01/81                          | NA                            |

Appendix B (Continued)

06/30/01

A.B-9

NUREG-0933

| Action Plan Item/Issue No. | Title  | Safety Priority/Status | Affected NSSS Vendor |     | Operating Plants- MPA No | Operating Plants - Effective Date | Future Plants- Effective Date |
|----------------------------|--|------------------------|----------------------|-----|--------------------------|-----------------------------------|-------------------------------|
|                            |  |                        | BWR                  | PWR |                          |                                   |                               |
| II.K.3(21)                 | Restart of Core Spray and LPCI Systems on Low Level - Design and Modification                  | I                      | GE                   | NA  | F-50                     | 01/01/81                          | 01/01/81                      |
| II.K.3(22)                 | Automatic Switchover of RCIC System Suction - Verify Procedures and Modify Design              | I                      | GE                   | NA  | F-51                     | 01/01/81                          | 01/01/81                      |
| II.K.3(24)                 | Confirm Adequacy of Space Cooling for HPCI and RCIC Systems                                    | I                      | GE                   | NA  | F-52                     | 01/01/82                          | 01/01/82                      |
| II.K.3(25)                 | Effect of Loss of AC Power on Pump Seals   | I                      | GE                   | NA  | F-53                     | 01/01/82                          | 01/01/82                      |
| II.K.3(27)                 | Provide Common Reference Level for Vessel Level Instrumentation                                | I                      | GE                   | NA  | F-54                     | 10/01/80                          | 10/01/80                      |
| II.K.3(28)                 | Study and Verify Qualification of Accumulators on ADS Valves                                   | I                      | GE                   | NA  | F-55                     | 01/01/82                          | 01/01/82                      |
| II.K.3(29)                 | Study to Demonstrate Performance of Isolation Condensers with Non-Condensibles                 | I                      | GE                   | NA  | F-56                     | 04/01/81                          | NA                            |
| II.K.3(30)                 | Revised Small-Break LOCA Methods to Show Compliance with 10 CFR 50, Appendix K                 | I                      | All                  | All | F-57                     | 01/01/83                          | 01/01/83                      |
| II.K.3(31)                 | Plant-Specific Calculations to Show Compliance with 10 CFR 50.46                               | I                      | All                  | All | F-58                     | 01/01/83                          | 01/01/83                      |
| II.K.3(44)                 | Evaluation of Anticipated Transients with Single Failure to Verify No Significant Fuel Failure | I                      | GE                   | NA  | F-59                     | 01/01/81                          | 01/01/81                      |
| II.K.3(45)                 | Evaluate Depressurization with Other Than Full ADS   | I                      | GE                   | NA  | F-60                     | 01/01/81                          | 01/01/81                      |
| II.K.3(46)                 | Response to List of Concerns from ACRS Consultant  | I                      | GE                   | NA  | F-61                     | 07/01/80                          | 07/01/80                      |
| II.K.3(57)                 | Identify Water Sources Prior to Manual Activation of ADS                                       | I                      | GE                   | NA  | F-62                     | 10/01/80                          | NA                            |
| <b>III.A</b>               | <b>EMERGENCY PREPAREDNESS AND RADIATION EFFECTS</b>  |                        |                      |     |                          |                                   |                               |
| <b>III.A.1</b>             | <b>Improve Licensee Emergency Preparedness - Short Term</b>                                    |                        |                      |     |                          |                                   |                               |
| III.A.1.1                  | Upgrade Emergency Preparedness   | -                      | -                    | -   | -                        | -                                 | -                             |
| III.A.1.1(1)               | Implement Action Plan Requirements for Promptly Improving Licensee Emergency Preparedness      | I                      | All                  | All |                          | 10/10/79                          | 08/19/80                      |
| III.A.1.2                  | Upgrade Licensee Emergency Support Facilities  | -                      | -                    | -   | -                        | -                                 | -                             |
| III.A.1.2(1)               | Technical Support Center   | I                      | All                  | All | F-63                     | 09/13/79                          | 09/27/79                      |
| III.A.1.2(2)               | On-Site Operational Support Center   | I                      | All                  | All | F-64                     | 09/13/79                          | 09/27/79                      |
| III.A.1.2(3)               | Near-Site Emergency Operations Facility  | I                      | All                  | All | F-65                     | 09/13/79                          | 09/27/79                      |
| <b>III.A.2</b>             | <b>Improving Licensee Emergency Preparedness-Long Term</b>                                     |                        |                      |     |                          |                                   |                               |
| III.A.2.1                  | Amend 10 CFR 50 and 10 CFR 50, Appendix E  | -                      | -                    | -   | -                        | -                                 | -                             |
| III.A.2.1(1)               | Publish Proposed Amendments to the Rules   | NOTE 3(a)              | All                  | All |                          |                                   |                               |
| III.A.2.1(4)               | Revise Inspection Program to Cover Upgraded Requirements                                       | I                      | All                  | All | F-67                     |                                   |                               |

Revision 16

Appendix B (Continued)

06/30/01

| Action Plan Item/Issue No. | Title  | Safety Priority/Status | Affected NSSS Vendor |     | Operating Plants- MPA No | Operating Plants - Effective Date | Future Plants- Effective Date |
|----------------------------|--|------------------------|----------------------|-----|--------------------------|-----------------------------------|-------------------------------|
|                            |  |                        | BWR                  | PWR |                          |                                   |                               |
| III.A.2.2                  | Development of Guidance and Criteria   | I                      | All                  | All | F-68                     |                                   |                               |
| <u>III.A.3</u>             | <u>Improving NRC Emergency Preparedness</u>  |                        |                      |     |                          |                                   |                               |
| III.A.3.3                  | Communications   | -                      | -                    | -   | -                        | -                                 | -                             |
| III.A.3.3(1)               | Install Direct Dedicated Telephone Lines   | NOTE 3(a)              | All                  | All |                          |                                   |                               |
| III.A.3.3(2)               | Obtain Dedicated, Short-Range Radio Communication Systems  | NOTE 3(a)              | All                  | All |                          |                                   |                               |
| <u>III.D</u>               | <u>RADIATION PROTECTION</u>  |                        |                      |     |                          |                                   |                               |
| <u>III.D.1</u>             | <u>Radiation Source Control</u>  |                        |                      |     |                          |                                   |                               |
| III.D.1.1                  | Primary Coolant Sources Outside the Containment Structure  | -                      | -                    | -   | -                        | -                                 | -                             |
| III.D.1.1(1)               | Review Information Submitted by Licensees Pertaining to Reducing Leakage from Operating Systems      | I                      | All                  | All |                          | 07/02/79                          | 09/27/79                      |
| <u>III.D.3</u>             | <u>Worker Radiation Protection Improvement</u>   |                        |                      |     |                          |                                   |                               |
| III.D.3.3                  | Inplant Radiation Monitoring   | -                      | -                    | -   | -                        | -                                 | -                             |
| III.D.3.3(1)               | Issue Letter Requiring Improved Radiation Sampling Instrumentation                                   | I                      | All                  | All | F-69                     | 09/13/79                          | 09/27/79                      |
| III.D.3.3(2)               | Set Criteria Requiring Licensees to Evaluate Need for Additional Survey Equipment                    | NOTE 3(a)              | All                  | All |                          | 09/13/79                          | 09/27/79                      |
| III.D.3.3(3)               | Issue a Rule Change Providing Acceptable Methods for Calibration of Radiation-Monitoring Instruments | NOTE 3(a)              | All                  | All |                          | 09/13/79                          | 09/27/79                      |
| III.D.3.3(4)               | Issue a Regulatory Guide   | NOTE 3(a)              | All                  | All |                          | 09/13/79                          | 09/27/79                      |
| III.D.3.4                  | Control Room Habitability  | I                      | All                  | All | F-70                     | 05/07/80                          | 06/26/80                      |

TASK ACTION PLAN ITEMS

A.B-10

NUREG-0933

|     |   |           |     |     |      |          |          |
|-----|---|-----------|-----|-----|------|----------|----------|
| A-1 | Water Hammer (former USI)   | NOTE 3(a) | All | All |      | NA       | 03/15/84 |
| A-2 | Asymmetric Blowdown Loads on Reactor Primary Coolant Systems (former USI) | NOTE 3(a) | NA  | All | D-10 | 01--/81  | 01--/81  |
| A-3 | Westinghouse Steam Generator Tube Integrity (former USI)                  | NOTE 3(a) | NA  | W   |      | 04/17/85 | 04/17/85 |
| A-4 | CE Steam Generator Tube Integrity (former USI)                            | NOTE 3(a) | NA  | CE  |      | 04/17/85 | 04/17/85 |
| A-5 | B&W Steam Generator Tube Integrity (former USI)                           | NOTE 3(a) | NA  | B&W |      | 04/17/85 | 04/17/85 |
| A-6 | Mark I Short-Term Program (former USI)                                    | NOTE 3(a) | GE  | NA  |      | 12--/77  | NA       |
| A-7 | Mark I Long-Term Program (former USI)                                     | NOTE 3(a) | GE  | NA  | D-01 | 08--/82  | 08--/82  |
| A-8 | Mark II Containment Pool Dyanmic Loads - Long Term Program (former USI)   | NOTE 3(a) | GE  | NA  |      | 08--/81  | 08--/81  |
| A-9 | ATWS (former USI)   | NOTE 3(a) | All | All |      | 06/26/84 | 06/26/84 |

Revision 16

Appendix B (Continued)

06/30/01

A.B-11

NUREG-0933

| Action Plan Item/Issue No. | Title  | Safety Priority/Status | Affected NSSS Vendor |          | Operating Plants- MPA No | Operating Plants - Effective Date | Future Plants- Effective Date |
|----------------------------|--|------------------------|----------------------|----------|--------------------------|-----------------------------------|-------------------------------|
|                            |  |                        | BWR                  | PWR      |                          |                                   |                               |
| A-10                       | BWR Feedwater Nozzle Cracking (former USI)   | NOTE 3(a)              | All                  | NA       | B-25                     | 11/--/80                          | 11/--/80                      |
| A-11                       | Reactor Vessel Materials Toughness (former USI)  | NOTE 3(a)              | All                  | All      |                          | 10/--/82                          | NA                            |
| A-12                       | Fracture Toughness of Steam Generator and Reactor Coolant Pump Supports (former USI)   | NOTE 3(a)              | NA                   | All      |                          | NA                                | TBD                           |
| A-13                       | Snubber Operability Assurance  | NOTE 3(a)              | All                  | All      | B-17, B-22               | 1980                              | 1980                          |
| A-16                       | Steam Effects on BWR Core Spray Distribution   | NOTE 3(a)              | GE                   | NA       | D-12                     | NA                                |                               |
| A-24                       | Qualification of Class 1E Safety Related Equipment (former USI)  | NOTE 3(a)              | All                  | All      | B-60                     | 08/--/81                          | 08/--/81                      |
| A-25                       | Non-Safety Loads on Class 1E Power Sources   | NOTE 3(a)              | All                  | All      |                          | 09/--/78                          |                               |
| A-26                       | Reactor Vessel Pressure Transient Protection (former USI)  | NOTE 3(a)              | NA                   | All      | B-04                     | 09/--/78                          | 09/--/78                      |
| A-28                       | Increase in Spent Fuel Pool Storage Capacity   | NOTE 3(a)              | All                  | All      |                          | 04/17/78                          | NA                            |
| A-31                       | RHR Shutdown Requirements (former USI)   | NOTE 3(a)              | All                  | All      |                          | 05/--/78                          | 10/01/78                      |
| A-35                       | Adequacy of Offsite Power Systems  | NOTE 3(a)              | All                  | All      | B-23                     | 06/02/77                          | 1980                          |
| A-36                       | Control of Heavy Loads Near Spent Fuel (former USI)  | NOTE 3(a)              | All                  | All      | C-10, C-15               | 07/--/80                          | 07/--/80                      |
| A-39                       | Determination of Safety Relief Valve Pool Dynamic Loads and Temperature Limits (former USI)  | NOTE 3(a)              | GE                   | NA       |                          | 02/29/80                          | 09/30/80                      |
| A-40                       | Seismic Design Criteria (former USI)   | NOTE 3(a)              | All                  | All      |                          | TBD                               | 09/--/89                      |
| A-42                       | Pipe Cracks in Boiling Water Reactors (former USI)   | NOTE 3(a)              | All                  | NA       | B-05                     | 02/--/81                          | 02/--/81                      |
| A-43                       | Containment Emergency Sump Performance (former USI)  | NOTE 3(a)              | NA                   | All      |                          | NA                                | 11/--/85                      |
| A-44                       | Station Blackout (former USI)  | NOTE 3(a)              | All                  | All      |                          | TBD                               | 06/--/88                      |
| A-46                       | Seismic Qualification of Equipment in Operating Plants (former USI)  | NOTE 3(a)              | All                  | All      |                          | 02/--/87                          | NA                            |
| A-47                       | Safety Implications of Control Systems (former USI)  | NOTE 3(a)              | All                  | All      |                          | 09/20/89                          | 09/20/89                      |
| A-48                       | Hydrogen Control Measures and Effects of Hydrogen Burns on Safety Equipment  | NOTE 3(a)              | All                  | <u>W</u> |                          | 12/--/81                          | 12/--/81                      |
| A-49                       | Pressurized Thermal Shock (former USI)   | NOTE 3(a)              | NA                   | All      | A-21                     | TBD                               | 07/--/85                      |
| B-10                       | Behavior of BWR Mark III Containments  | NOTE 3(a)              | GE                   | NA       |                          | NA                                | 09/--/84                      |
| B-36                       | Develop Design, Testing, and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Feature Systems and for Normal Ventilation Systems | NOTE 3(a)              | All                  | All      |                          | 03/--/78                          |                               |
| B-56                       | Diesel Reliability   | NOTE 3(a)              | All                  | All      | D-19                     | 06/--/93                          | 06/--/93                      |
| B-63                       | Isolation of Low Pressure Systems Connected to the Reactor Coolant Pressure Boundary   | NOTE 3(a)              | All                  | All      | B-45                     | 04/20/81                          |                               |
| B-64                       | Decommissioning of Reactors  | NOTE 3(a)              | All                  | All      |                          | 06/27/88                          | NA                            |
| B-66                       | Control Room Infiltration Measurements   | NOTE 3(a)              | All                  | All      |                          | NA                                | 07/--/81                      |
| C-1                        | Assurance of Continuous Long Term Capability of Hermetic Seals on Instrumentation and Electrical Equipment   | NOTE 3(a)              | All                  | All      |                          | 05/27/80                          | 05/27/80                      |

Appendix B (Continued)

06/30/01

| Action Plan Item/Issue No. | Title   | Safety Priority/Status | Affected NSSS Vendor |              | Operating Plants- MPA No   | Operating Plants - Effective Date | Future Plants- Effective Date |
|----------------------------|---|------------------------|----------------------|--------------|--|-----------------------------------|-------------------------------|
|                            |   |                        | BWR                  | PWR          |  |                                   |                               |
| C-10                       | Effective Operation of Containment Sprays in a LOCA                                     | NOTE 3(a)              | All                  | All          |  | NA                                |                               |
| C-17                       | Interim Acceptance Criteria for Solidification Agents for Radioactive Solid Wastes      | NOTE 3(a)              | All                  | All          |  | 12/27/82                          | 12/27/82                      |
| <b>NEW GENERIC ISSUES</b>  |   |                        |                      |              |  |                                   |                               |
| 25.                        | Automatic Air Header Dump on BWR Scram System   | NOTE 3(a)              | All                  | NA           |  | 01/09/81                          | 01/09/81                      |
| 40.                        | Safety Concerns Associated with Pipe Breaks in the BWR Scram System                     | NOTE 3(a)              | All                  | NA           | B-65   | 08/31/81                          | 08/31/81                      |
| 41.                        | BWR Scram Discharge Volume Systems  | NOTE 3(a)              | All                  | NA           | B-58   | 12/09/80                          | NA                            |
| 43.                        | Reliability of Air Systems  | NOTE 3(a)              | All                  | All          | B-107  | 08/08/88                          | 08/08/88                      |
| 45                         | Inoperability of Instrumentation Due to Extreme Cold Weather                            | NOTE 3(a)              | All                  | All          |  | NA                                | 09/01/83                      |
| 51.                        | Proposed Requirements for Improving the Reliability of Open Cycle Service Water Systems | NOTE 3(a)              | All                  | All          | L-913  | 07/18/89                          | 07/18/89                      |
| 67.                        | <u>Steam Generator Staff Actions</u>  | -                      | -                    | -            | -  | -                                 | -                             |
| 67.3.3                     | Improved Accident Monitoring  | NOTE 3(a)              | All                  | All          | A-17   | 12/17/82                          | 12/17/82                      |
| 70.                        | PORV and Block Valve Reliability  | NOTE 3(a)              | NA                   | All          |  | 06/25/90                          | 06/25/90                      |
| 73.                        | Detached Thermal Sleeves  | NOTE 3(a)              | NA                   | <u>W</u>     |  | NA                                |                               |
| 75.                        | Generic Implications of ATWS Events at the Salem Nuclear Plant                          | NOTE 3(a)              | All                  | All          | B-76, B-77, B-78, B-79, B-80, B-81, B-82, B-85, B-86, B-87, B-88, B-89, B-90, B-91, B-92, B-93 | 07/08/83                          | TBD                           |
| 86.                        | Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping                | NOTE 3(a)              | All                  | NA           | B-84   | TBD                               | TBD                           |
| 87.                        | Failure of HPCI Steam Line Without Isolation  | NOTE 3(a)              | All                  | All          |  | 06/28/89                          | 06/28/89                      |
| 89.                        | Stiff Pipe Clamps   | NOTE 6                 | All                  | All          | NA   | NA                                | TBD                           |
| 93                         | Steam Binding of Auxiliary Feedwater Pumps  | NOTE 3(a)              | NA                   | All          | B-98   | 10/-/85                           | 10/-/85                       |
| 94                         | Additional Low Temperature Overpressure Protection for Light Water Reactors             | NOTE 3(a)              | NA                   | CE, <u>W</u> |  | 06/25/90                          | 06/25/90                      |
| 99                         | RCS/RHR Suction Line Valve Interlock on PWRs  | NOTE 3(a)              | NA                   | All          | L-817  | 10/17/88                          | 10/17/88                      |
| 103                        | Design for Probable Maximum Precipitation   | NOTE 3(a)              | All                  | All          |  | 10/19/89                          | 10/19/89                      |
| 118                        | Tendon Anchorage Failure  | NOTE 3(a)              | All                  | All          | NA   | NA                                | 07/-/90                       |
| 124                        | Auxiliary Feedwater System Reliability  | NOTE 3(a)              | All                  | All          |  | TBD                               | TBD                           |
| 128                        | Electrical Power Reliability  | NOTE 3(a)              | All                  | All          |  | 04/29/91                          | 04/29/91                      |
| 130                        | Essential Service Water Pump Failures at Multiplant                                     | NOTE 3(a)              | NA                   | All          |  | 09/19/91                          | 09/19/91                      |

A.B-12

NUREG-0933

Revision 16



APPENDIX F

NUCLEAR MATERIAL SAFETY AND SAFEGUARDS GSIs

This appendix documents those non-reactor GSIs identified, prioritized, and resolved by NMSS. As stated in SECY-98-001,<sup>1724</sup> the prioritization procedure for these issues is contained in NMSS Policy and Procedures Letter 1-57,<sup>1725</sup> "NMSS Generic Issues Program."

**TABLE F.1**  
**LISTING OF NMSS GSIs**

This table contains the priority designations for all NMSS GSIs listed in Appendix F.

Legend

NOTES: 3(a) - Resolution Resulted in the Establishment of New Requirements  
 3(b) - Resolution Resulted in the Establishment of No New Requirements  
 4 - Issue to be Prioritized in the Future  
 HIGH - High Safety Priority  
 MEDIUM - Medium Safety Priority  
 LOW - Low Safety Priority

| Issue No. | Title   | Priority Engineer | LeadOffice/<br>Division/Branch | Safety<br>Priority<br>Ranking | Latest<br>Revision | Latest<br>Issuance<br>Date |
|-----------|---|-------------------|--------------------------------|-------------------------------|--------------------|----------------------------|
| NMSS-0001 | Door Interlock Failure Resulting from Faulty MicroSelectron-High Dose Rate Remote Afterloader                   | Ramsey            | NMSS/IMNS/IMOB                 | NOTE 3(b)                     |                    | 12/31/1998                 |
| NMSS-0002 | Significant Quantities of Fixed Contamination Remain in Krypton-85 Leak-Detection Devices After Venting         | Ramsey            | NMSS/IMNS/IMOB                 | NOTE 3(b)                     |                    | 12/31/1998                 |
| NMSS-0003 | Corrosion of Sealed Sources Caused by Sensitization of Stainless Steel Source Capsules During Shipment          | Ramsey            | NMSS/IMNS/IMOB                 | NOTE 3(b)                     |                    | 12/31/1998                 |
| NMSS-0004 | Overexposures Caused by Sources Stolen from Facility of Bankrupt Licensee                                       | Ramsey            | NMSS/IMNS/IMOB                 | NOTE 3(b)                     |                    | 12/31/1998                 |
| NMSS-0005 | Potential for Erroneous Calibration, Dose Rate, or Radiation Exposure Measurements With Victoreen Electrometers | Ramsey            | NMSS/IMNS/IMOB                 | NOTE 3(a)                     |                    | 12/31/1998                 |
| NMSS-0006 | Criticality in Low-Level Waste  | Ramsey            | NMSS/IMNS/IMOB                 | NOTE 3(b)                     | 1                  | 06/30/2000                 |
| NMSS-0007 | Criticality Benchmarks Greater Than 5% Enrichment   | Ramsey            | NMSS/FCSS                      | HIGH                          | 1                  | 06/30/2001                 |
| NMSS-0008 | Year 2000 Computer Problem - Non-Reactor Licensees  | Ramsey            | NMSS/IMNS                      | NOTE 3(b)                     | 1                  | 06/30/2000                 |
| NMSS-0009 | Amersham Radiography Source Cable Failures  | Ramsey            | NMSS/IMNS                      | NOTE 3(b)                     |                    | 12/31/1998                 |

06/30/01

A.F.0-3

NUREG-0933

| Issue No. | Title  | Priority Engineer | LeadOffice/<br>Division/Branch | Safety<br>Priority<br>Ranking | Latest<br>Revision | Latest<br>Issuance<br>Date |
|-----------|--|-------------------|--------------------------------|-------------------------------|--------------------|----------------------------|
| NMSS-0010 | Troxler Gauge Source Rod Weld Failures   | Ramsey            | NMSS/IMNS                      | MEDIUM                        |                    | 12/31/1998                 |
| NMSS-0011 | Spent Fuel Dry Cask Weld Cracks  | Ramsey            | NMSS/SFPO                      | NOTE 3(b)                     |                    | 12/31/1998                 |
| NMSS-0012 | Inadequate Transportation Packaging Puncture Tests   | Ramsey            | NMSS/SFPO                      | NOTE 3(b)                     | 1                  | 06/30/2000                 |
| NMSS-0013 | Use of Different Dose Equivalent Models to Show Compliance   | Ramsey            | NMSS/IMNS                      | NOTE 3(b)                     | 1                  | 06/30/2000                 |
| NMSS-0014 | Surety Estimates for Groundwater Restoration at In-Situ Leach Fields   | Ramsey            | NMSS/DWM                       | MEDIUM                        |                    | 12/31/1998                 |
| NMSS-0015 | Adequacy of 10 CFR 150 Criticality Requirements  | Ramsey            | NMSS/DWM                       | NOTE 3(b)                     | 1                  | 06/30/2000                 |
| NMSS-0016 | Adequacy of 0.05 Weight Percent Limit in 10 CFR 40   | Ramsey            | NMSS/IMNS                      | MEDIUM                        |                    | 12/31/1998                 |
| NMSS-0017 | Misleading Marketing Information to General Licensees  | C. Mattsen        | NMSS/IMNS                      | NOTE 3(a)                     |                    | 06/30/2001                 |
| NMSS-0018 | Problems Encountered When Manually Editing Treatment Planning Data on Nucletron MicroSelectron-HDR Model 105.999 | B. Ayres          | NMSS/IMNS                      | NOTE 3(b)                     |                    | 06/30/2001                 |
| NMSS-0019 | Control Unit Failures of Classic Nucletron HDR Units   | B. Ayres          | NMSS/IMNS                      | NOTE 3(b)                     |                    | 06/30/2001                 |
| NMSS-0020 | Leaking Pools  | M. Sitek          | NMSS/IMNS                      | DROP                          |                    | 06/30/2001                 |
| NMSS-0021 | Unlikely Events  | M. Sitek          | NMSS/IMNS                      | DROP                          |                    | 06/30/2001                 |
| NMSS-0022 | Gamma Stereotactic Radiosurgery  | M. Sitek          | NMSS/IMNS                      | DROP                          |                    | 06/30/2001                 |

NMSS-0007: CRITICALITY BENCHMARKS GREATER THAN 5% ENRICHMENTDESCRIPTION

The importance of computer software (methods and data) in establishing the criticality safety of systems with fissile material is increasing as licensees work to optimize facilities and storage/transport packages at the same time that access to experimental data is decreasing. Available experimental data are insufficient to validate nuclear criticality safety evaluations for all required configurations at U<sup>235</sup> enrichments in the range of 5% to 20%. This issue was identified<sup>1709</sup> by NMSS to develop and confirm the adequacy of methods, analytical tools, and guidance for criticality safety software to be used in licensing nuclear facilities.

CONCLUSION

The issue was initially given a low priority ranking<sup>1709</sup> which was later changed to a high priority.<sup>1787</sup>

REFERENCES

1709. Memorandum to J. Craig from F. Combs, "Submittal of New Generic Issues for Tracking in the Generic Issues Management and Control System," June 4, 1998.
1787. Memorandum to F. Eltawila from D. Cool, "NMSS Input for Second Quarter FY-2001 Update of the Generic Issue Management Control System," April 12, 2001.

## NMSS-0017: MISLEADING MARKETING INFORMATION TO GENERAL LICENSEES

### DESCRIPTION

Radioactive products to be distributed under a general license are required to be inherently safe so that they can be used by untrained people. However, licensed devices containing radioactive materials have not always been disposed of, nor handled properly, particularly those authorized by general licenses. This has, on some occasions, resulted in radiation exposure to the public and subsequent costly decontamination of property.

NRC's routine inspections of licensees indicate that some distributors and manufacturers appear not to completely understand the regulations and their responsibilities. The general licensees that possess these types of products have responsibilities, but many times do not clearly understand them until an accident forces them to do so. Many times the manufacturers and distributors of these types of radioactive products portray them to prospective customers as non-maintenance items that can be readily bought off the shelf, installed without any consideration given to special circumstances or safe location, and forgotten about. Product-related literature may provide misleading information, or information that can easily be misconstrued by omission or commission. In order to address this issue,<sup>1780</sup> the staff proposed to: (1) issue an information notice to all distributors and manufacturers of generally licensed devices; and (2) revise 10 CFR Part 32 to require manufacturers/distributors to provide more information on the general licensees' responsibilities under the regulations.

### CONCLUSION

Information Notice 99-26<sup>1781</sup> was issued to all distributors and/or manufacturers of generally licensed products to alert them of the possible threat to public safety caused by misleading marketing information and lack of end-user understanding of the regulatory requirements in terms of the transfer and disposal of generally licensed devices. This issue was ultimately resolved with the approval of a final rule by the Commission on July 10, 2000. The final rule requires distributors to provide copies of § 31.5 to general licensees prior to time of transfer of the device rather than at the time of transfer of the device. The distributor is also required to provide copies of additional applicable sections of the regulations; a listing of the services that can only be performed by a specific licensee; in the case of NRC general licensees, a statement concerning high civil penalties for improper disposal of sources; and information regarding disposal options for the devices being transferred.

### REFERENCES

1780. Memorandum to F. Eltawila from D. Cool, "Submittal of Generic Issues for Tracking in the Generic Issue Management Control System (GIMCS)," November 14, 2000.
1781. NRC Information Notice 99-26, "Safety and Economic Consequences of Misleading Marketing Information," August 24, 1999.

NMSS-0018: PROBLEMS ENCOUNTERED WHEN MANUALLY EDITING TREATMENT  
PLANNING DATA ON NUCLETRON MICROSELECTRON-HDR MODEL 105.999

DESCRIPTION

Two misadministrations resulted when a licensee manually edited treatment data for the Nucletron HDR unit. While attempting to manually edit one parameter, unintended and unnoticed changes to the source step size occurred which caused the misadministration. The unintended changes occurred when the licensee attempted to select the dwell positions to be edited, by the inappropriate use of the "Tab" and "arrow" keys, the other (unrelated) treatment plan parameters were changed. The licensee did not expect those parameters to change. This expectation was reinforced by the lack of any cautions or other instructions, in the Nucletron MicroSelectron-HDR v1.2X manual, that would indicate otherwise. More importantly, the licensee expected to see a confirmation message that would have required it to acknowledge any changes to the treatment plan step size, in accordance with the information on page 4-23 of the users manual. Secondly, since the licensee did not anticipate that any of the other treatment plan parameters would change it did not check any parameters other than the source dwell times on the final printed treatment plan.

In order to address this issue,<sup>1780</sup> the staff proposed to issue an information notice to all medical licensees authorized to conduct high-dose-rate (HDR) remote after loading brachytherapy treatments. Furthermore, the staff proposed to work with the FDA and the State of Maryland to have the manufacturer modify the software. Rulemaking or other actions were to be considered if the effort to have the software modified was unsuccessful.

CONCLUSION

Information Notice 99-09<sup>1782</sup> was issued to inform medical licensees of the potential problems. The issue was resolved when NRC verified that the manufacturer corrected the software problem.

REFERENCES

1780. Memorandum to F. Eltawila from D. Cool, "Submittal of Generic Issues for Tracking in the Generic Issue Management Control System (GIMCS)," November 14, 2000.
1782. NRC Information Notice 99-09, "Problems Encountered When Manually Editing Treatment Data on the Nucletron MicroSelectron-HDR (New) Model 105.999," March 24, 1999.

## NMSS-0019: CONTROL UNIT FAILURES OF CLASSIC NUCLETRON HDR UNITS

### DESCRIPTION

Several control unit failures in Nucletron Classic Model HDR units occurred over the span of three years: two were reported in 1997; four in 1998; and three in 1999. These nine control unit failures indicated an ongoing problem with unexplained control unit failures. Nucletron continued to investigate the ongoing control unit failures and believes it found the root cause of the failures and subsequently developed appropriate corrective measures. Affected licensees were to be informed and any necessary additional actions were to be taken to ensure that safety was maintained.

### CONCLUSION

This issue<sup>1780</sup> was resolved with the issuance of Information Notice 99-23.<sup>1783</sup> Additional follow-up with Nucletron on their corrective measures was deemed unnecessary following an analysis of the failures that deemed them to be of low safety significance.

### REFERENCES

1780. Memorandum to F. Eltawila from D. Cool, "Submittal of Generic Issues for Tracking in the Generic Issue Management Control System (GIMCS)," November 14, 2000.
1783. NRC Information Notice 99-23, "Safety Concerns Related to Repeated Control Unit Failures of the Nucletron Classic Model High-Dose-Rate Remote Afterloading Brachytherapy Devices," July 6, 1999.

## NMSS-0020: LEAKING POOLS

### DESCRIPTION

On September 19, 2000, BWX Technologies determined that a water leak existed in the Cask Handling Area Pool at their Lynchburg, VA facility. The pool of water is used to store irradiated commercial reactor hardware along with irradiated fuel rods. The average concentrations of radionuclides (Cs-137 and Co-60) leaking from the pool significantly exceeded the limits in 10 CFR Part 20, Appendix B, Table 2, for effluent releases to unrestricted areas (10 CFR 20.1301 and 20.1302). The leak was not identified for an extended period of time. The contaminated water, however, did not actually reach an unrestricted area.

The licensee did not have a formal program for tracking and trending pool level data or additions to the pool, or for monitoring groundwater quality near the pool. Furthermore, NRC regulations do not specifically require these measures, nor does its NRC license. As a result, the inspection program for this facility did not evaluate the licensee's ability to identify and correct pool leaks. Based on the leaking pool at BWX Technologies, the potential for unrecognized pool leaks with long term environmental or health effects was identified as a generic issue.<sup>1784</sup>

The existing regulations for irradiator pools were promulgated in 1994 and were developed, in part, in response to a specific type of sealed source that was susceptible to leaking. Thus, for irradiator pools, undetected pool leakage is not a concern because the current regulations require the sealed sources used in these pools to meet certain performance criteria that are designed to significantly minimize the possibility that radioactivity will ever even reach the pool water. Any radioactivity that leaks into the pool water will be detected due to the requirement on licensees to monitor their pools for elevated levels of radioactivity. The regulations further require licensees to cease operations and immediately determine the cause of any detected elevated activity in their pools. Furthermore, irradiator pools licensed after 1994 must be designed and constructed based on criteria that prevent the likelihood of pool leakage.

### CONCLUSION

The NMSS staff evaluated whether regulatory programs were adequate to ensure that leaks of contaminated water from pools would be promptly identified and corrected by licensees. The applicable licensees were those that maintained pools as wet-storage irradiators, along with the specific licensees of GE Morris, GE Vallecitos, and West Valley.

The GE Morris facility (an Independent Spent Fuel Storage Installation) was required to have a well monitoring program to verify that offsite release limits were not exceeded. Furthermore, 10 CFR Part 72 requires the licensee to have a QA program which included a monitoring program that would indicate any pool leakage. The West Valley facility was scheduled for closure in early 2001 and, as a result, the need for pool monitoring at this facility was not necessary. While there were not specific regulations requiring monitoring at GE Vallecitos, there were license conditions which required a pool monitoring and maintenance program that would indicate pool leakage. In January 2001, a series of corrective actions were being implemented at BWXT to ensure that any future leaks would not go undetected.

For all types of licensees, the staff determined that the risks associated with not monitoring pools for leakage are very low and the issue was dropped from further pursuit. This conclusion was reached based on the adequacy of the physical barriers that must be overcome in order for material to escape the pool and reach unrestricted areas. Furthermore, any material that escaped would be greatly diluted before it ever reached an unrestricted area.<sup>1784</sup>



#### REFERENCE

1784. Memorandum to W. Travers from W. Kane, "Closure of Two NMSS Generic Issues," January 26, 2001.



## NMSS-0021: UNLIKELY EVENTS

### DESCRIPTION

Inspections at the gaseous diffusion plants identified that they were not properly applying ANSI/ANS-8.1-1983, "Nuclear Criticality Safety in Operations with Fissionable Materials Outside Reactors," with regard to unlikely events. Many licensees adopt the standards in ANSI/ANS-8.1-1983 as part of their nuclear criticality safety programs. These standards state that criticality safety is achieved by controlling certain parameters within sub-critical limits. The safety concern with the use of unlikely events at the gaseous diffusion plants (GDPs) is that "unlikely events" are being used as one leg of double contingency without establishing specific controls that make the occurrence of the event unlikely. With the exception of external phenomena (earthquakes, floods, and so forth), there are few events that are inherently unlikely without depending on the nature of the operation and specific engineered and administrative features that make the event unlikely. Without specific features of the system being identified and controlled as important to safety, these items cannot be placed under the configuration control program, as required to maintain the plant's safety basis, and the availability and reliability of these features cannot be ensured. This issue<sup>1784</sup> concerned a bank of evaporators at Portsmouth, the failure of which had been considered an unlikely event even though there was a demonstrated history of failure.

The existing regulations and/or license conditions require fuel cycle facilities to have nuclear criticality safety programs. The NRC reviews, approves, and inspects these programs as part of the regulatory process. The failure of the GDPs to correctly apply the ANSI standard was captured by the existing regulatory processes and was an issue that was expected to be resolved through enforcement. Furthermore, while nuclear criticality safety is applicable to all fuel cycle facilities, the incorrect interpretation of ANSI/ANS-8.1-1983 by the GDPs was deemed to be an isolated case. In order to verify that the NRC was properly interpreting the standard, ANSI was consulted by the licensee which in turn supported NRC's interpretation that an unlikely event was not to be used as a contingency.

### CONCLUSION

While the issue was of low safety significance and did not appear to be generic, follow-up action was pursued. In January 2001, a letter to the GDPs was being prepared to clarify the proper use of unlikely events as outlined by ANSI/ANS-8.1-1983. The development of guidance generic to the fuel cycle industry, based on the letter to the GDPs, was also to be considered.

Finally, the requirements in Subpart H of the new 10 CFR Part 70 (particularly those relating to the identification of Items Relied on for Safety (IROFS), and performance of an Integrated Safety Analysis, or ISA) will also serve to significantly increase the likelihood that licensees have an adequately documented criticality safety program. These new requirements will require a more systematic analysis of systems and processes relied on for criticality safety and control. Thus, the issue was dropped from further pursuit.<sup>1784</sup>

REFERENCE

1784. Memorandum to W. Travers from W. Kane, "Closure of Two NMSS Generic Issues," January 26, 2001.



## NMSS-0022: GAMMA STEREOTACTIC RADIOSURGERY

### DESCRIPTION

NRC was informed in July 2000 of a medical misadministration during gamma stereotactic radiosurgery that occurred in the State of California in September 1998. The misadministration was the result of an erroneous coordinate setting which resulted in an unintended site of the brain receiving 10 gray. The licensee's procedure to independently verify the coordinate setting also failed to identify the error prior to treatment. The State requested that the licensee, as part of their corrective action plan, investigate the manner with which coordinate settings are verified at other facilities that perform gamma stereotactic radiosurgery. The licensee discovered that there was a great deal of variability in this verification process, and as a result, a wide range of probabilities of having a coordinate setting error that could lead to a medical misadministration.

As a result of the findings of the California licensee, the State planned to require all of its licensees that perform gamma stereotactic radiosurgery to change their procedures to improve the process by which coordinate settings are verified. Specifically, these licensees will be required to have a "double check" verification procedure. In this procedure, the coordinates are called out by one individual while person A sets them and persons B and C independently verify the setting. This method of setting coordinates was based on a University of Pittsburgh study which indicated that the probability of coordinate setting errors was reduced from 1 in 400 to 1 in 155,000 when a "double check" procedure is used, as opposed to a "single check."

The State of California shared these findings with other Agreement States and NRC because it believed that there were generic implications associated with safely performing gamma stereotactic radiosurgery, particularly with respect to the verification of coordinate settings. As a result, the NMSS staff evaluated whether existing NRC regulatory programs were adequate to prevent medical misadministrations that result from human errors associated with the setting of coordinates on the stereotactic frame during gamma stereotactic radiosurgery.

The existing and pending regulations in 10 CFR Part 35 require licensees to prepare a written directive prior to gamma stereotactic radiosurgery. Prescribed target coordinate settings for each treatment for each distinct treatment site are required elements of the written directive. In addition, licensees are required to have written procedures which provide high confidence that the written directive will be followed as prepared. Finally, the existing guidance on this topic suggested that at least one qualified person independently verify the coordinate settings. The combination of the existing regulations and guidance are adequate, to the extent that licensees are required to have procedures to ensure that coordinates are set in accordance with the physicians' treatment plan and written directive. In those cases where coordinates are not set in accordance with the written directive and the treatment subsequently executed, the existing regulatory process will identify and address the issue<sup>1785</sup> through the appropriate mechanisms such as enforcement and licensee corrective actions.

### CONCLUSION

Examination of existing operating data related to gamma stereotactic radiosurgery revealed that the probability of coordinate setting errors is very low. Of the tens of thousands of gamma

stereotactic radiosurgery procedures that have been performed over the last ten years, only sixteen misadministrations had been reported, with six of these cases involving coordinate setting errors. The misadministrations in these cases did not result in negative health consequences. In fact, the probability that a critical nerve or other sensitive structure will be impacted as a result of a misadministration is small since these structures represent a small portion of the total volume of the brain. The combination of the low probability of misadministration occurrence and the low probability of severe consequences if a misadministration occurs results in the risks associated with coordinate setting errors being very low.

In addition to the staff's evaluation, Information Notice 2000-22<sup>1786</sup> was issued to applicable licensees to remind them of their responsibilities with respect to written directives and to ensuring that the written directives were followed as planned. Furthermore, a series of best practices on how other licensees ensure correct coordinate settings were identified in IN 2000-22.<sup>1786</sup> The staff determined that the low risk associated with coordinate setting errors combined with the current and emerging performance-based regulations to require some form of verification do not justify more prescriptive oversight. Thus, the issue was dropped from further pursuit.<sup>1785</sup>

#### REFERENCES

- 1785. Memorandum to W. Travers from W. Kane, "Closure of NMSS Generic Issue Relating to Gamma Stereotactic Radiosurgery," February 12, 2001.
- 1786. NRC Information Notice 2000-22, "Medical Misadministrations Caused by Human Errors Involving Gamma Stereotactic Radiosurgery (Gamma Knife)," December 18, 2000.

**BIBLIOGRAPHIC DATA SHEET**

*(See instructions on the reverse)*

1. REPORT NUMBER  
(Assigned by NRC, Add Vol., Supp., Rev.,  
and Addendum Numbers, if any.)

NUREG-0933  
Supplement 25

2. TITLE AND SUBTITLE

A Prioritization of Generic Safety Issues

3. DATE REPORT PUBLISHED

MONTH | YEAR

July | 2001

4. FIN OR GRANT NUMBER

5. AUTHOR(S)

R. Emrit, et al.

6. TYPE OF REPORT

7. PERIOD COVERED *(Inclusive Dates)*

07-01-2000 to 06-30-2001

8. PERFORMING ORGANIZATION - NAME AND ADDRESS *(If NRC, provide Division, Office or Region, U.S. Nuclear Regulatory Commission, and mailing address; if contractor, provide name and mailing address.)*

Division of Systems Analysis and Regulatory Effectiveness  
Office of Nuclear Regulatory Research  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555-0001

9. SPONSORING ORGANIZATION - NAME AND ADDRESS *(If NRC, type "Same as above"; if contractor, provide NRC Division, Office or Region, U.S. Nuclear Regulatory Commission, and mailing address.)*

Division of Systems Analysis and Regulatory Effectiveness  
Office of Nuclear Regulatory Research  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555-0001

SUPPLEMENTARY NOTES

11. ABSTRACT *(200 words or less)*

The report presents the safety priority rankings for generic safety issues related to nuclear power plants. The purpose of these rankings is to assist in the timely and efficient allocation of NRC resources for the resolution of those safety issues that have a significant potential for reducing risk. The safety priority rankings are HIGH, MEDIUM, LOW, and DROP, and have been assigned on the basis of risk significance estimates, the ratio of risk to costs and other impacts estimated to result if resolution of the safety issues were implemented, and the consideration of uncertainties and other quantitative and qualitative factors. To the extent practical, estimates are quantitative.

12. KEY WORDS/DESCRIPTORS *(List words or phrases that will assist researchers in locating the report.)*

generic safety issues

13. AVAILABILITY STATEMENT

unlimited

14. SECURITY CLASSIFICATION

*(This Page)*

unclassified

*(This Report)*

unclassified

15. NUMBER OF PAGES

16. PRICE



Federal Recycling Program