

April 21, 1989

DO NOT REMOVE

Docket Nos.: 50-269, **50-270**
and 50-287

P O S T E D

**50-270
OCONEE 2
AMENDMENT NO. 173
TO DPR-47**

Mr. H. B. Tucker, Vice President
Nuclear Production Department
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

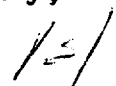
SUBJECT: ISSUANCE OF AMENDMENT NOS. 173, 173, AND 170 TO FACILITY OPERATING LICENSES DPR-38, DPR-47, and DPR-55 - OCONEE NUCLEAR STATION, UNITS 1, 2 AND 3 (TACS 63553/63554/63555)

The Nuclear Regulatory Commission has issued the enclosed Amendment Nos. 173, 173, and 170 to Facility Operating Licenses Nos. DPR-38, DPR-47 and DPR-55 for the Oconee Nuclear Station, Units 1, 2 and 3. These amendments consist of changes to the Station's Technical Specifications (TS) in response to your request dated October 13, 1986, as supplemented December 22, 1988.

The amendments require a Type C local leak test for penetration no. 22, low pressure service water from the reactor coolant pump motors and lube oil coolers outlet, in accordance with Appendix J, "Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors."

A copy of our Safety Evaluation is also enclosed. Notice of issuance of the enclosed amendments will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,



Darl S. Hood, Acting Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.173 to DPR-38
2. Amendment No.173 to DPR-47
3. Amendment No.170 to DPR-55
4. Safety Evaluation

cc w/enclosures:
See next page

OFFICIAL RECORD COPY

PDII-3/DRP-I/II*
MRood
04/17/89

PDII-3/DRP-I/II*
DHood:ls
04/17/89

OGC*
04/18/89

PDII-3/DRP-I/II
DMatthews
4/20/89

*See previous concurrence



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 21, 1989

Docket Nos.: 50-269, 50-270
and 50-287

Mr. H. B. Tucker, Vice President
Nuclear Production Department
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

SUBJECT: ISSUANCE OF AMENDMENT NOS. 173, 173, AND 170 TO FACILITY OPERATING
LICENSES DPR-38, DPR-47, and DPR-55 - OCONEE NUCLEAR STATION,
UNITS 1, 2 AND 3 (TACS 63553/63554/63555)

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A copy of our Safety Evaluation is also enclosed. Notice of issuance of the enclosed amendments will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

A handwritten signature in dark ink that reads "Darl S. Hood".

Darl S. Hood, Acting Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 173 to DPR-38
2. Amendment No. 173 to DPR-47
3. Amendment No. 170 to DPR-55
4. Safety Evaluation

cc w/enclosures:
See next page

Mr. H. B. Tucker
Duke Power Company

Oconee Nuclear Station
Units Nos. 1, 2 and 3

cc:

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Raleigh, North Carolina 27603

Honorable James M. Phinney
County Supervisor of Oconee County
Walhalla, South Carolina 29621



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 173
License No. DPR-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 1 (the facility) Facility Operating License No. DPR-38 filed by the Duke Power Company (the licensee) dated October 13, 1986, as supplemented December 22, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 3.B. of Facility Operating License No. DPR-38 is hereby amended to read as follows:

3.B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 173, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: April 21, 1989

3.B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 173, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes


Date of Issuance: April 21, 1989

OFFICIAL RECORD COPY

LA:RDII-3
MR60d
04/17/89

^{DSH}
PM:PDII-3
DHood:ls
04/17/89

OGC-WF
OGC
04/15/89


D:RDII-3
DMatthews
04/20/89



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-270

OCONEE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 173
License No. DPR-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 2 (the facility) Facility Operating License No. DPR-47 filed by the Duke Power Company (the licensee) dated October 13, 1986, as supplemented December 22, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 3.B. of Facility Operating License No. DPR-47 is hereby amended to read as follows:

3.B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 173, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: April 21, 1989

3.B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 173, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: April 21, 1989

OFFICIAL RECORD COPY

LA:PDII-3
MRHod
04/17/89

^{DSH}
PM:PDII-3
DHood:ls
04/17/89

see chart 1

OGC-WF
04/ /89

DM

D:PDII-3
DMatthews
04/20/89



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-287

OCONEE NUCLEAR STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 170
License No. DPR-55

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 3 (the facility) Facility Operating License No. DPR-55 filed by the Duke Power Company (the licensee) dated October 13, 1986, as supplemented December 22, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 3.B. of Facility Operating License No. DPR-55 is hereby amended to read as follows:

3.B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.170 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: April 21, 1989

3.B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 170, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: April 21, 1989

OFFICIAL RECORD COPY

LA: RDII-3
MRood
04/17/89

DSH for
PM: PDII-3
HPastis:pw
04/17/89

OGC-WF
W. P. ...
04/18/89

D: PD II-3
DMatthews
04/20/89

conditioned on changes noted to SER + letter of transmittal

ATTACHMENT TO LICENSE AMENDMENT NO. 173

FACILITY OPERATING LICENSE NO. DPR-38

DOCKET NO. 50-269

AND

TO LICENSE AMENDMENT NO. 173

FACILITY OPERATING LICENSE NO. DPR-47

DOCKET NO. 50-270

AND

TO LICENSE AMENDMENT NO. 170

FACILITY OPERATING LICENSE NO. DPR-55

DOCKET NO. 50-287

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

Amended Page

4.4-8

TABLE 4.4-1
LIST OF PENETRATIONS WITH 10CFR50,
APPENDIX J TEST REQUIREMENTS

PENETRATION NUMBER	SYSTEM	TYPE A TEST SYSTEM CONDITION	LOCAL LEAK TEST	REMARKS
22	LPSW from RC Pump motors and lube oil coolers outlet	Not Vented	Type C	Note 7b
23	RC Pump seal injection	Not Vented	Type C	Note 5, 7d, 9
24	RB H ₂ Analyzer Train A	Note 1	Type C	Note 7c
25	OTSG B Feedwater line	Not Vented	None required	Note 5
26	OTSG A Main steam line	Not Vented	None required	Note 5, MS Stop valve leak test performed
27	OTSG A Feedwater line	Not Vented	None required	Note 5
28	OTSG B Main steam line	Not Vented	None required	Note 5, MS Stop valve leak test performed
29	Quench tank drain line	Note 1	Type C	Note 3, 7b, 9
30	LPSW for RB	Not Vented	None required	Note 5
31	Cooling units			
32	inlet line			
33	LPSW for RB	Not Vented	None required	Note 5
34	cooling units			
35	outlet line			

4.4-8

Amendment No. 173 (Unit 1)
Amendment No. 173 (Unit 2)
Amendment No. 170 (Unit 3)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO.173 TO FACILITY OPERATING LICENSE DPR-38
AMENDMENT NO.173 TO FACILITY OPERATING LICENSE DPR-47
AMENDMENT NO.170 TO FACILITY OPERATING LICENSE DPR-55

DUKE POWER COMPANY

OCONEE NUCLEAR STATION, UNITS 1, 2 AND 3

DOCKET NOS. 50-269, 50-270 AND 50-287

1.0 INTRODUCTION

By letters dated October 13, 1986, and December 22, 1988, Duke Power Company (Duke or the licensee) submitted an application proposing revisions to the common Oconee Nuclear Station (ONS) Technical Specifications (TSs) on the testing requirements of certain containment isolation valves. The application would revise Table 4.4-1, "List of Penetrations with 10 CFR Part 50, Appendix J Requirements." The proposed amendments would require a Type C local leak test for penetration no. 22, low pressure service water (LPSW) from the reactor coolant pump motors and lube oil coolers outlet (inlet valve, no. 21; outlet valve, no. 22) in accordance with Appendix J, "Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors."

Because the December 22, 1988, submittal clarified certain aspects of the original request, the substance of the changes noticed in the FEDERAL REGISTER and the proposed no significant hazards consideration were not affected.

2.0 EVALUATION

By letter dated November 6, 1981, Duke requested a revision to its September 3, 1981 application on local leak test requirements for penetrations 21 and 22. In the November 6, 1981 letter, Duke concluded that Type C testing of valve LPSW-15 was not required because the outboard side of the valve would remain pressurized by the LPSW system during a loss-of-coolant accident (LOCA).

By November 6, 1981 amendment (Nos. 104, 104, and 100, letter from P. Wagner, NRC to W. Parker, Duke), the NRC staff concluded that sufficient assurance existed that the LPSW would be pressurized on the outboard side of penetration no. 22 to preclude leakage of containment atmosphere. Therefore, the current TS Table 4.4-1 does not require a local leak test for penetration no. 22. The conclusion that testing was not required was based on the belief that, with LPSW being in service, the pressure in the line outside valve LPSW-15 would be greater than 60 psig following an engineered safeguards (ES) closure of this

valve. Subsequently, Duke checked the pressure outside LPSW-15 and found it to be 12 psig or less with LPSW-15 closed. This datum invalidated the basis for not requiring a local leak test for penetration no. 22. The licensee has included this penetration in its inservice testing program.

Appendix J to 10 CFR Part 50 requires that licensees identify all containment isolation valves that will be locally (Type C) tested and prescribes methods for conducting the containment isolation valve leak rate tests.

Type C leakage rate test should be performed on valve LPSW-15 to meet Appendix J requirements, and, as noted in the licensee's letter dated December 22, 1988, the test pressure will be applied in the same direction as that when the valve would be required to perform its safety function. This testing will ensure that penetration no. 22 will meet the leak rate criteria of Appendix J during an ES actuation of valve LPSW-15. The revision to the TS would constitute a more stringent requirement by requiring Type C leakage rate testing of penetration no. 22 when no testing was required previously. Therefore, the staff finds this proposed revision to the TSs acceptable.

Based on the above, the staff concludes that the proposed changes to the TS, which add valve LPSW-15 to the Type C testing program, are required by Appendix J and are, therefore, acceptable.

3.0 ENVIRONMENTAL CONSIDERATIONS

These amendments involve a change in the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (52 FR 16943) on May 6, 1987, and consulted with the state of South Carolina. No public comments were received, and the state of South Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities

will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Pulsipher, SPLB/DEST
D. Hood, PD#II-3/DRP-I/II

Dated: April 21, 1989