

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

Orket

January 11, 1979

Dockets Nos.:

Mr. William O. Parker, Jr. Vice President - Steam-Production Duke Power Company 422 South Church Street P. O. Box 2178 Charlotte, North Carolina 28242

Dear Mr. Parker:

The Commission has issued the enclosed Amendments Nos. 67 , 67 , and 64for Licenses Nos. DPR-38, DPR-47 and DPR-55 for the Oconee Nuclear Station, Units Nos. 1, 2 and 3. These amendments consist of changes to the Station's common Technical Specifications and are in response to your request dated December 22, 1978.

These amendments revise the Technical Specifications by authorizing a change in the operability requirements of the reactor building spray pumps.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Robert W. Reid. Chief

Operating Reactors Branch #4 Division of Operating Reactors

Enclosures:

Amendment No. 67 to DPR-38

Amendment No. 67 to DPR-47

3. Amendment No. 64 to DPR-55

4. Safety Evaluation

Notice

ccw/enclosures: See next page

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Duke Power Company

cc w/enclosure(s):
Mr. William L. Porter
Duke Power Company
P. O. Box 2178
422 South Church Street
Charlotte, North Carolina 28242

J. Michael McGarry, III, Esquire DeBevoise & Liberman 700 Shorenam Building 806-15th Street, NW., Washington, D.C. 20005

Oconee Public Library 201 South Spring Street Walhalla, South Carolina 29691

Honorable James M. Phinney County Supervisor of Oconee County Walhalla, South Carolina 29621

Chief, Energy Systems
Analyses Branch (AW-459)
Office of Radiation Programs
U. S. Environmental Protection Agency
Room 645, East Tower
401 M Street, S. W.
Washington, D. C. 20460

U. S. Environmental Protection Agency Region IV Office ATTN: EIS COORDINATOR 345 Coutland Street, N. E. Atlanta, Georgia 30308

cc w/enclosures & incoming dtd: 12/22/78 Office of Intergovernmental Relations 116 West Jones Street Raleigh, North Carolina 27603



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 67 License No. DPR-33

- The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duke Power Company (the licensee) dated December 22, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license arendment and paragraph 3.B of Facility Operating License No. DPR-38 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 67 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Robert W. Reid, Chief

Operating Reactors Branch #4
Division of Operating Reactors

Attachment: Changes to the Technical Specifications

Date of Issuance: January 11, 1979



NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-270

OCONEE NUCLEAR STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 67 License No. DPR-47

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duke Power Company (the licensee) dated December 22, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

 Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-47 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 67 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

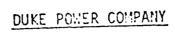
Robert W. Reid, Chief

Operating Reactors Branch #4 Division of Operating Reactors

Attachment: Changes to the Technical Specifications

Date of Issuance: January 11, 1979

UNITED STATES LEAR REGULATORY COMMISSION WASHINGTON, D. C. 20535



DOCKET NO.

OCONEE NUCLEAR STATION, UNIT NO. 3

50-287

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 64 License No. DPR-55

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duke Power Company (the licensee) dated December 2.2, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-55 is hereby amended to read as follows:

3.B <u>Technical Specifications</u>

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 64 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert W. Reid, Chief

Operating Reactors Branch #4 Division of Operating Reactors

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Attachment: Changes to the Technical Specifications

Date of Issuance: January 11, 1979

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 67 TO DPR-38

AMENDMENT NO. 67 TO DPR-47

AMENDMENT NO. 64 TO DPR-55

DOCKETS NOS. 50-269, 50-270, AND 50-287

Revise Appendix A by removing page 3.3-3 and inserting the attached revised identically numbered page.

Changes on the revised page are indicated by a marginal line.

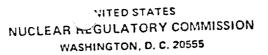
Exceptions to 3.3.5 shall be as follows: 3.3.6

- (a) Both core flooding tanks shall be operational above 800 psig.
- (b) Both motor-operated valves associated with the core flooding tanks shall be fully open above 800 psig.
- One pressure instrument channel and one level instrument channel per core flood tank shall be operable above 800 psig.
- One reactor building cooling fan and associated cooling unit shall be permitted to be out of service for seven days provided both reactor building spray pumps and associated spray nozzle headers are in service at the same time.
- (e) If the requirements of Specification 3.3.1(f) are not met, the borated water storage tank shall be considered unavailable and action shall be initiated in accordance with Specification 3.2.
- (f) One reactor building spray pump and its associated spray nozzle header shall be permitted to be out of service for 72 hours provided all Reactor Building Cooling Units are operable.
- Prior to initiating maintenance on any of the components, the dupli-3.3.7 cate (redundant) component shall be tested to assure operability.
- (a) Reactor power shall not be increased above 60% FP unless three 3.3.8 HPI pumps and two HPI flow paths are operable.
 - (b) During power operation above 60% FP, tests or maintenance shall be allowed on any one HPI pump, provided two trains of the HPI system are operable. If the inoperable HPI pump is not restored to operable status within 72 hours, reactor power shall be reduced below 60% FP within an additional 12 hours.
 - (c) If during power operation above 60% FP a high pressure injection flow path becomes inoperable, reactor power shall be reduced below 60% FP within 12 hours.

The requirements of Specification 3.3 assure that, before the reactor can be made critical, adequate engineered safety features are operable. Two high pressure injection pumps and two low pressure injection pumps are required (except as specified in Specification 3.3.8 and as discussed further on in these bases.) However, only one of each is necessary to supply emergency coolant to the reactor in the event of a loss-of-coolant accident. Both core flooding tanks are required as a single core flood tank has insufficient inventory to reflood the core. (1)

The borated water storage tanks are used for two purposes:

- (a) As a supply of borated water for accident conditions.
- (b) As a supply of borated water for flooding the fuel transfer canal during refueling operation. (2)



SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION SUPPORTING AMENDMENT NO. 67 TO FACILITY OPERATING LICENSE NO. DPR-38

AMENDMENT NO. 67 TO FACILITY OPERATING LICENSE NO. DPR-47

AMENDMENT NO. 64 TO FACILITY OPERATING LICENSE NO. DPR-55

DUKE POWER COMPANY

OCONEE NUCLEAR STATION, UNITS NOS. 1, 2 AND 3

DOCKETS NOS. 50-269, 50-270 AND 50-287

Introduction

By letter dated December 22, 1978, Duke Power Company (the licensee) requested a change to the Oconee Nuclear Station, Units Nos. 1, 2 and 3, Technical Specifications (TS). This change would extend the period of inoperability of a reactor building spray train from 24 hours to 72 hours before the reactor must proceed with a cooldown to the hot shutdown condition.

Evaluation

The reactor building spray system is a post loss-of-coolant accident (LOCA) heat removal system; no credit is taken for iodine removal. The spray system consists of two independent trains of pumps, spray nozzle headers, and associated piping, valves and controls. Either independent spray train provides sufficient heat removal to perform the post LOCA reactor building heat removal function as described in the Final Safety Analysis Report (FSAR). For similar systems in other operating nuclear plants, we have found that a period of inoperability of up to seven days for a single train is acceptable; the licensee requests a period of only 72 hours of inoperability. In addition to the heat removal capability of the spray system, the reactor building is equipped with a fan cooling system. The fan coolers provide sufficient heat removal capability so that even with both spray trains inoperable, the fan coolers alone maintain the containment pressure within the post-LOCA design limits of the Accident Analysis section of the FSAR. For the reasons stated above, we find the extension of the period of inoperability for a single spray train from 24 hours to 72 hours to be acceptable.

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Environmental Consideration

We have determined that these amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: January 11, 1979

UNITED STATES NUCLEAR REGULATORY COMMISSION DOCKETS NOS. 50-269, 50-270 AND 50-287 DUKE POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 67, 67, and 64 to Facility Operating Licenses Nos. DPR-38, DPR-47 and DPR-55, respectively, issued to Duke Power Company for operation of the Oconee Nuclear Station, Units Nos. 1, 2 and 3, located in Oconee County, South Carolina. The amendments are effective as of the date of issuance.

These amendments revise the Technical Specifications by authorizing a change in the operability requirements of the reactor building spray pumps.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

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The Commission has determined that the issuance of these americants will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated December 22, 1978, (2) Amendments Nos. 67, and 64 to Licenses Nos. DPR-38, EPR-47 and DPR-55; respectively. and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. and at the Oconee County Litrary, 201 South Spring Street, Walhalla, South Carolina. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 11th day of January 1979.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert W. Reid, Chief

Operating Reactors Branch #4
Division of Operating Reactors