Posted
amdt. 133
to DPR-47

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

January 9, 1985

Dockets Nos. 50-269, 50-270
and 50-287

Mr. H. B. Tucker
Vice President - Steam Production
Duke Power Company
P. O. Box 33189
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

The Commission has issued the enclosed Amendments Nos. 133, 133, and 130 to Facility Operating Licenses Nos. DPR-38, DPR-47 and DPR-55 for the Oconee Nuclear Station, Units Nos. 1, 2 and 3. These amendments consist of changes to the Station's common Technical Specifications (TSs) in response to your request dated April 30, 1984.

These amendments revise the Administrative Controls Section of the TSs to reflect the current regulations governing licensee event reports as required by the Commission. Other changes requested in the April 30, 1984 submittal are still under staff review and will be addressed by separate Safety Evaluation and license amendment.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance of the enclosed amendments will be included in the Commission's monthly notice.

Sincerely,

Helen Nicolaras

Helen Nicolaras, Project Manager
Operating Reactors Branch #4
Division of Licensing

Enclosures:
1. Amendment No. 133 to DPR-38
2. Amendment No. 133 to DPR-47
3. Amendment No. 130 to DPR-55
4. Safety Evaluation

cc w/enclosures:
See next page

Duke Power Company

cc w/enclosure(s):

Mr. William L. Porter
Duke Power Company
P. O. Box 33189
422 South Church Street
Charlotte, North Carolina 28242

Office of Intergovernmental Relations
116 West Jones Street
Raleigh, North Carolina 27603

Honorable James M. Phinney
County Supervisor of Oconee County
Walhalla, South Carolina 29621

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission, Region II
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30323

Heyward G. Shealy, Chief
Bureau of Radiological Health
South Carolina Department of Health
and Environmental Control
2600 Bull Street
Columbia, South Carolina 29201

Regional Radiation Representative
EPA Region IV
345 Courtland Street, N.E.
Atlanta, Georgia 30308

Mr. J. C. Bryant
Senior Resident Inspector
U.S. Nuclear Regulatory Commission
Route 2, Box 610
Seneca, South Carolina 29678

Mr. Robert B. Borsum
Babcock & Wilcox
Nuclear Power Generation Division
Suite 220, 7910 Woodmont Avenue
Bethesda, Maryland 20814

Manager, LIS
NUS Corporation
2536 Countryside Boulevard
Clearwater, Florida 33515

J. Michael McGarry, III, Esq.
Bishop, Liberman, Cook, Purcell & Reynolds
1200 17th Street, N.W.
Washington, D. C. 20036

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 133
License No. DPR-38

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Duke Power Company (the licensee) dated April 30, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-38 is hereby amended to read as follows:

   3.B Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No.133 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 9, 1985

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-270

OCONEE NUCLEAR STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 133
License No. DPR-47

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Duke Power Company (the licensee) dated April 30, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-47 is hereby amended to read as follows:

   3.B Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No.133 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 9, 1985

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-287

OCONEE NUCLEAR STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 130
License No. DPR-55

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Duke Power Company (the licensee) dated April 30, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-55 is hereby amended to read as follows:

   3.B Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No.130 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 9, 1985

ATTACHMENTS TO LICENSE AMENDMENTS

AMENDMENT NO. 133 TO DPR-38

AMENDMENT NO. 133 TO DPR-47

AMENDMENT NO. 130 TO DPR-55

DOCKETS NOS. 50-269, 50-270 AND 50-287

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment numbers and contain vertical lines indicating the area of change.

Remove Pages	Insert Pages
v-a	v-a
3.17-1	3.17-1
3.17-2	3.17-2
3.17-3	3.17-3
6.2-1	6.2-1
6.5-1	6.5-1
6.6-4	6.6-4
6.6-5	6.6-5
6.6-6	-
6.6-6a	-
6.6-7	-

3.17 FIRE PROTECTION AND DETECTION SYSTEMS

Applicability

This specification applies to the operability of fire protection and detection systems when equipment protected by those systems is required to be operable.

Objective

To assure the operability of fire protection and detection systems which protect systems and equipment required for safe shutdown.

Specification

3.17.1 The minimum fire detection instrumentation for each fire detection zone shown in Table 3.17-1 shall be operable. The fire detection instruments located within the containment are not required to be operable during the performance of Type A Containment Leakage Rate Tests. When this specification is determined not to be met, appropriate action shall be taken consisting of one or more of the following:

1. Within 1 hour, a fire watch patrol shall be established to inspect an accessible zone with the inoperable instrumentation at least once per hour.

2. The inoperable instrumentation shall be restored to operable status within 14 days or a report shall be submitted to the Commission within the next 30 days outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the instrumentation to operable status. Continued operation of the affected unit is permitted provided that this condition is met. Operation under this specification is not considered to be in a degraded mode and thus is not reportable under Technical Specification 6.6.2.1.

3.17.2 The Fire Suppression Water System shall be operable. This system shall consist of 2 High Pressure Service Water (HPSW) pumps with a design capacity of 6000 gpm each and automatic initiation logic, and the associated piping and valves supplying water to the sprinkler systems and fire hose stations. The HPSW pumps shall be aligned to the high pressure fire header. When this specification is determined not to be met, appropriate action shall be taken consisting of the following:

1. The inoperable equipment shall be restored to operable status within 7 days or a report shall be submitted to the Commission within the next 30 days outlining the plans and procedures to be used to provide for the loss of redundancy in this system. Continued operation of the affected unit is permitted provided that this condition is met. Operation under this specification is not considered to be in a degraded mode and thus is not reportable under Technical Specification 6.6.2.1.

2. With no Fire Suppression Water System operable, in lieu of the above, the following action shall be taken.

   a. Within 24 hours a backup Fire Suppression Water System shall be established. If a backup Fire Suppression Water System cannot be established within 24 hours, place the reactor in Hot Standby within the next twelve (12) hours and in cold shutdown within the following forty-eight (48) hours.

   b. Within 24 hours the Commission shall be notified by telephone, and in writing no later than the first working day following the event.

   c. Within 14 days of the event, a report shall be submitted to the Commission outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to operable status.

3.17.3 The sprinkler and spray systems in safety related areas listed in Table 3.17-1 shall be operable. If a system is determined to be inoperable, the following corrective action shall be taken.

1. A continuous fire watch with backup fire suppression equipment shall be established in the area within 1 hour.

2. The sprinkler or spray system shall be restored to operable status within 14 days or a report shall be submitted to the Commission within the next 30 days outlining the cause of inoperability and the plans for restoring the system to operable status. Continued operation of the affected unit is permitted provided that this condition is met. Operation under this specification is not considered to be a degraded mode and thus is not reportable under Technical Specification 6.6.2.1.

3.17.4 The automatic CO_2 system provided for the generators at the Keowee Hydro Station shall be operable. If the system is determined to be inoperable the following corrective action shall be taken:

1. A continuous fire watch with backup fire suppression equipment shall be established in the area within 1 hour.

2. The CO_2 system shall be restored to operable status within 14 days or a report shall be submitted to the Commission within the next 30 days outlining the cause of inoperability and the plans for restoring the system to operable status. Continued operation of the affected reactor unit is permitted provided that this condition is met. Operation under this specification is not considered to be in a degraded mode and thus is not reportable under Technical specification 6.6.2.1.

3.17.5 The fire hose stations listed in Table 3.17-1 shall be operable or the following action shall be taken:

1. If a fire hose station listed in Table 3.17.1 is inoperable, an additional equivalent capacity fire hose of length sufficient to reach the unprotected area shall be provided at an operable hose station within 1 hour.

2. If the inoperable fire hose station cannot be restored to operable status within 14 days, continued operation of the affected unit is permitted provided that within the next 30 days a report is submitted to the Commission outlining the cause of the inoperability, actions taken, and the plans for restoring the system to operable status. Operation under this specification is not considered to be a degraded mode and is not reportable under Tech. Spec. 6.6.2.1.

3.17.6 All fire barrier penetrations (including cable penetration barriers, fire doors, fire dampers) protecting safety related areas shall be operable.

If a fire barrier protecting a safety-related area is determined to be inoperable, the operability status of the fire detection instrumentation for the affected safety related area(s) shall be determined within 1 hour, and the following action shall be taken:

1. If the fire detection instrumentation for the affected area(s) is operable, a fire watch patrol shall be established to inspect the area at least once per hour.

2. If the fire detection instrumentation is inoperable, a continuous fire watch shall be established within the next hour on at least one side of the affected penetration fire barrier. The non-functional fire barrier penetration(s) shall be restored to functional status within 7 days.

3. If the non-functional fire barrier penetration(s) cannot be restored to functional status within 7 days, continued operation of the affected unit is permitted provided that within the next 30 days, a report is submitted to the Commission outlining the cause of the inoperability and the plans for restoring the system to operable status. Operation under this specification is not considered to be a degraded mode and is not reportable under Technical Specification 6.6.2.1.

Bases

Operability of the fire detection instrumentation ensures that adequate warning capability is available for the prompt detection of fires. This capability is required in order to detect and locate fires in their early stages. Prompt detection of fires will reduce the potential for damage to safety related equipment and is an integral element in the overall facility fire protection program.

6.2 ACTION TO BE TAKEN IN THE EVENT OF A REPORTABLE EVENT

6.2.1 Any reportable event shall be investigated promptly by the station Manager.

6.2.2 The Vice President, Nuclear Production Department shall be notified of any reportable event. A written report shall be prepared which describes the circumstances leading up to and resulting from the incident and shall recommend appropriate action to prevent or minimize the probability of a recurrence. The report shall be submitted to the Vice President, Nuclear Production Department, and the Director of the Nuclear Safety Review Board.

6.2.3 The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.6.2.

6.5 STATION OPERATING RECORDS

Specification

6.5.1 The following records shall be prepared and permanently retained in a manner convenient for review:

a. Records of modifications to the station as described in the FSAR.

b. Special nuclear material physical inventory records.

c. Special nuclear material isotopic inventory records.

d. Radiation monitoring records, including records of radiation and contamination surveys.

e. Records of off-site environmental surveys.

f. Personnel radiation exposure records as required by 10CFR20.

g. Records of radioactive releases and waste disposal.

h. Records of reactor coolant system in-service inspections.

i. Preoperational testing records.

j. Records of special reactor tests or experiments.

k. Records of changes to safety-related operating procedures.

l. Records for Environmental Qualification which are covered under the provisions of paragraph 6.7.

m. Records of the seal service lives of hydraulic snubbers.

6.5.2 The following records shall be prepared and retained for a minimum of six (6) years in a manner convenient for review:

a. Switchboard Record.

b. Reactor Operations Logbook.

c. Shift Supervisor Logbook.

d. Maintenance histories for station safety-related structures, systems and components.

e. Records of safety-related inspections, other than reactor coolant system in-service inspections.

f. Records of reportable events.

g. Periodic testing records and records of other periodic checks, calibrations, etc. performed in accordance with surveillance requirements for safety-related parameters, structures, systems and components.

6.6.2 Non-Routine Reports

6.6.2.1 Reportable Events

Reporting requirements for Licensee Event Reports are contained in 10 CFR 50, §50.73.

6.6.2.2 Environmental Monitoring

a. If individual milk samples show I-131 concentrations of 10 picocuries per liter or greater, a plan shall be submitted within one week advising the NRC of the proposed action to ensure the plant related annual doses will be within the design objective of 45 mrem/yr to the thyroid of any individual.

b. If milk samples collected over a calendar quarter show average concentrations of 4.8 picocuries per liter or greater, a plan shall be submitted within 30 days advising the NRC of the proposed action to ensure the plant related annual doses will be within the design objective of 45 mrem/yr to the thyroid of any individual.

6.6.3 Special Reports

Special reports shall be submitted to the Regional Administrator, Region II, within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

a. Single Loop Restrictions, Specification 3.1.8

b. Auxiliary Electrical Systems, Specification 3.7

c. Radioactive Liquid Effluents,
   Dose, Specification 3.9.2
   Liquid Waste Treatment, Specification 3.9.3
   Chemical Treatment Ponds, Specification 3.9.4

d. Radioactive Gaseous Effluents,
   Dose, Specification 3.10.2
   Gaseous Radwaste Treatment, Specification 3.10.3

e. Fire Protection and Detection Systems, Specification 3.17

f. Reactor Coolant System Surveillance,
   Inservice Inspection, Specification 4.2.1
   Reactor Vessel Specimen, Specification 4.2.4

g. Reactor Building Surveillance,
   Containment Leakage Tests, Specification 4.4.1

h. Structural Integrity Surveillance,
   Tendon Surveillance, Specification 4.4.2.2

i. Radiological Environmental Monitoring
   Program, Specification 4.11.1
   Land Use Census, Specification 4.11.2

j. Dose Calculations (40 CFR 190), Specification 4.21

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 133 TO FACILITY OPERATING LICENSE NO. DPR-38

AMENDMENT NO.133 TO FACILITY OPERATING LICENSE NO. DPR-47

AMENDMENT NO. 130TO FACILITY OPERATING LICENSE NO. DPR-55

DUKE POWER COMPANY

OCONEE NUCLEAR STATION, UNITS NOS. 1, 2, AND 3

DOCKETS NOS. 50-269, 50-270 AND 50-287

1.0 Introduction

By letter dated April 30, 1984, Duke Power Company (the licensee) proposed changes to the Technical Specifications (TSs) of Facility Operating Licenses Nos. DPR-38, DPR-47 and DPR-55 for the Oconee Nuclear Station, Units Nos. 1, 2 and 3. These amendments would consist of changes to the Station's common TSs and revise the "Administrative Controls" section of the TSs to reflect current regulations governing licensee event reports as required by the Commission. Other changes requested in the April 30, 1984 submittal are still under staff review and will be addressed by separate Safety Evaluation and license amendment.

2.0 Evaluation

The proposed amendment requests modification to the TSs to reflect changes in the reporting requirements as outlined in the Commission's regulations 10 CFR 50.72 and 10 CFR 50.73 and in accordance with the guidance provided in the Commission's Generic Letter (GL) 83-43. The licensee's request for amendment was reviewed against the guidance and model TSs attached to the GL 83-43 that show the appropriate revisions which should be made in the "Administrative Controls" section.

Pages 3.17-1, 3.17-2 and 3.17-3 are included in this revision package to update TS 6.6.2.1 Reportable Event references. Page v-a is being revised to reference the new page numbering. Pages 6.2-1 and 6.5-1 delete the term "Reportable Occurrence" and add the term "Reportable Event." Page number 6.6-7 has been changed to 6.6-5 because the following material has been deleted from:

Page 6.6-4: blank page;
6.6-5: prompt notification with written followup;
6.6-6
and
6.6-6a: thirty-day written reports

We have reviewed the proposed changes to the TSs and find that they conform to the changes of the Commission's regulations of 10 CFR 50.72 and 10 CFR 50.73 and have been revised in accordance with the guidance provided in the Commission's GL 83-43. Inasmuch as these amendments reflect changes in nomenclature and in the scope of licensee reporting obligations, we have determined that the proposed changes are administrative in nature. They revise the TSs only in format to meet the regulations, and therefore do not affect the operation of the plant or the health and safety of the public. Therefore, we find the proposed changes to the TSs to be acceptable.

3.0 Environmental Consideration

The amendments relate to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

4.0 Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: January 9, 1985

Principal Contributor: H. Nicolaras