

NRC 2001-048

10 CFR 50.90

July 27, 2001

Document Control Desk
U.S. Nuclear Regulatory Commission
Mail Stop P1-137
Washington, DC 20555

Ladies/Gentlemen:

DOCKETS 50-266 AND 50-301
SUPPLEMENT 14 TO APPLICATION FOR AMENDMENT TO
FACILITY OPERATING LICENSE APPENDIX A:
TECHNICAL SPECIFICATIONS IMPROVEMENT PROJECT
RESPONSE TO REQUEST FOR ADDITIONAL INFORMATION ON SPECIFICATION 5.6.5.b
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

On November 15, 1999, Wisconsin Electric Power Company (WE), then licensee for the Point Beach Nuclear Plant (PBNP), submitted an application to amend Appendix A, Technical Specifications, for Facility Operating Licenses DPR-24 and DPR-27 for Point Beach Nuclear Power Plant, Units 1 and 2, respectively (reference letter NPL 99-0669). The application proposed to convert the Point Beach Current Technical Specifications (CTS) to the Point Beach Improved Technical Specifications (ITS). That application contained documentation for ITS Chapters 1.0 and 2.0 and Sections 3.0 through 3.9. Documentation for ITS Chapters 4.0 and 5.0 was enclosed with Supplement 1 to the PBNP ITS submittal dated March 15, 2000 (reference letter NPL 2000-0142).

During a telephone conversation between the NRC and plant staff on July 20, 2001, the NRC staff requested that proposed ITS Specification 5.6.5.b be revised to include a reference to the NRC letter (July 23, 2001) that documents the NRC review and approval of the analytical methods used to determine the RCS pressure and temperature limits for Point Beach.

Attachment 1 of this letter includes the NMC response to the staff's request. The response includes changes that are required to the original submittal, including changes to the Current Technical Specification (CTS) markups, NUREG markups, and proposed ITS. These changes are discussed in the response to the NRC's request and are included in the attachment.

The changes required to the CTS, NUREG, and ITS markups are identified as follows (example):



The revision bar identifies the section that has been revised and the G in the triangle identifies revision G. The old pages in the original submittal should be replaced with the new pages enclosed with this letter, following the instructions of attachment 2.

NMC has determined that this supplement does not involve a significant hazards consideration, authorize a significant change in the types or total amounts of effluent release, or result in any significant increase in individual or cumulative occupational radiation exposure. Therefore, NMC concludes that the proposed supplement meets the categorical exclusion requirements of 10 CFR 51.22(c)(9) and that an environmental impact appraisal need not be prepared.

NMC is notifying the State of Wisconsin of this supplement by transmitting a copy of this letter, and its attachments, to the Public Service Commission of Wisconsin.

Other supplements to the PBNP ITS submittal, in response to previous RAIs, are listed for reference:

- Supplement 2 dated June 15, 2000 (ITS sections 2.0, 3.1, 3.2, 3.5; letter NPL 2000-0260)
- Supplement 3 dated June 19, 2000 (ITS section 3.6; letter NPL 2000-0271)
- Supplement 4 dated July 28, 2000 (ITS section 3.8; letter NPL 2000-0341)
- Supplement 5 dated August 17, 2000 (ITS sections 3.4, 3.9; letter NPL 2000-0371)
- Supplement 6 dated September 14, 2000 (ITS section 5.5; letter NPL 2000-0411)
- Supplement 7 dated October 19, 2000 (ITS sections 3.6, 3.7.4, 3.7.5; letter NPL 2000-0465)
- Supplement 8 dated December 21, 2000 (ITS section 1.0; letter NPL 2000-0549)
- Supplement 9 dated February 6, 2001 (ITS sections 3.3.1 and 5.0; letter NPL 2001-0032)
- Supplement 10 dated February 23, 2001 (ITS section 3.7; letter NRC 2001-0004)
- Supplement 11 dated March 19, 2001 (ITS sections 3.3.2-3.3.5; letter NRC 2001-0010)
- Supplement 12 dated May 11, 2001 (ITS sections 3.3, 3.7, 3.8 followup; letter NRC 2001-032)
- Supplement 13 dated June 13, 2001 (Comments on Draft Safety Evaluation; letter NRC 2001-039)

To the best of my knowledge and belief, the statements contained in this document are true and correct. In some respects, these statements are not based entirely on my personal knowledge, but on information furnished by cognizant NMC employees, contractor employees, and/or consultants. Such information has been reviewed in accordance with company practice, and I believe it to be reliable.

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Should you have any questions on this submittal or require additional information, please contact me.

Sincerely,



Mark Reddemann
Site Vice President

Subscribed to and sworn before me
on this 27th day of July, 2001



Notary Public, State of Wisconsin

My Commission expires on September 16, 2001.

RS/jlk

Attachments

Enclosure

cc: NRC Regional Administrator
NRC Resident Inspector

NRC Project Manager
PSCW

bcc: (w/o enclosures)

R. G. Mende

R. P. Pulec

T. J. Webb

B. J. Onesti (OSRC)

File

A. J. Cayia

J. Gadzala

D. F. Johnson

J. L. Kudick (3)

M. E. Reddemann

R. A. Anderson

R. R. Grigg

D. Weaver

DOCKETS 50-266 AND 50-301
RESPONSE TO REQUEST FOR ADDITIONAL INFORMATION
TECHNICAL SPECIFICATIONS IMPROVEMENT PROJECT
FOLLOWUP STAFF QUESTIONS
POINT BEACH NUCLEAR PLANT UNITS 1 AND 2

The following information is provided in response to the Nuclear Regulatory Commission staff's followup requests as discussed during a telephone conversation between NRC and plant staff on July 20, 2001.

The NRC request is restated below with the NMC's response following.

ITS 5.5.6.b, Reactor Coolant System (RCS) PRESSURE AND TEMPERATURE LIMITS REPORT (PTLR)

CTS Specification 5.6 markup Insert 5.6-2, item b. states that information will be submitted to the NRC for approval under Technical Specification Change Request (TSCR) 219. Furthermore, a statement is made that after NRC approval of TSCR 219, a supplement to this section will be submitted to identify the necessary approval amendments.

Proposed ITS Specification 5.6.5.b does not currently include reference to the NRC's approval of the analytical methods used by Point Beach to determine the RCS pressure and temperature limits. Comment: Provide a reference in ITS Specification 5.6.5.b to NRC Letter dated July 23, 2001, which documents approval of the analytical methods used by Point Beach to determine the RCS pressure and temperature limits.

Response:

The NRC documented approval of the analytical methods used by Point Beach to determine the pressure and temperature limits in a letter dated July, 23, 2001. This reference has been included in the proposed ITS Specification 5.6.5.b. This change also results in a change to the marked up CTS (page 15 of 16) and marked up NUREG Insert 5.0-11. The associated Description of Change, Justification for Deviation and No Significant Hazards Considerations still bound the above changes and do not require revision.

**ATTACHMENT 2
DISCARD AND INSERTION INSTRUCTIONS**

VOLUME 11	
SECTION 5.0	
DISCARD	INSERT
CTS markup page 15 of 16	CTS markup page 15 of 16
ISTS markup page 5.0-21	ISTS markup page 5.0-21
ISTS markup, Insert 5.0-11	ISTS markup, Insert 5.0-11
ITS page 5.6-4	ITS page 5.6-4

ENCLOSURE

Insert 5.6-2:

Reactor Coolant System (RCS) PRESSURE AND TEMPERATURE LIMITS REPORT (PTLR)

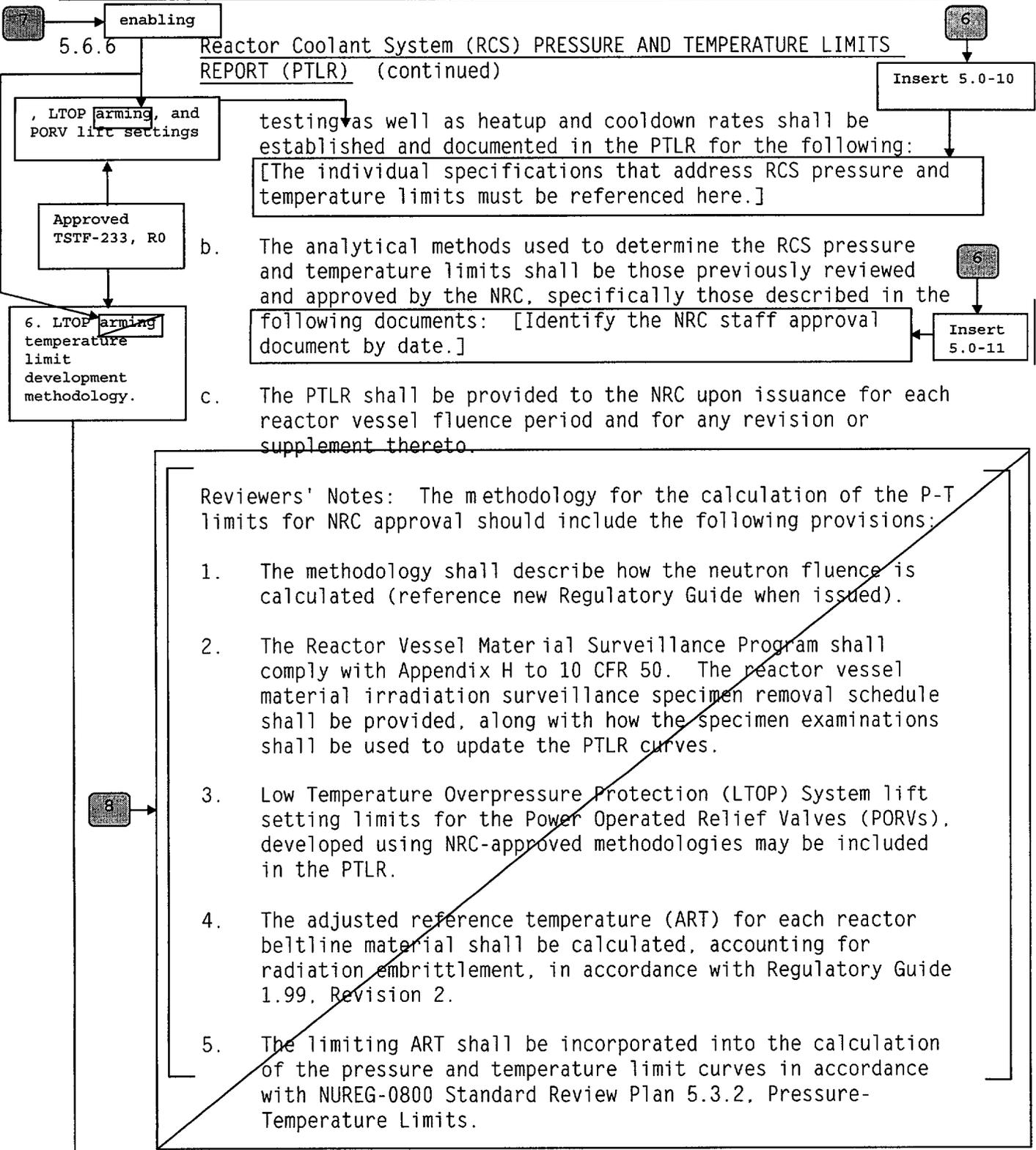
- a. RCS pressure and temperature limits for heat up, cooldown, low temperature operation, criticality, hydrostatic testing, LTOP enabling, and PORV lift settings as well as heatup and cooldown rates shall be established and documented in the PTLR for the following:
 - (1) LCO 3.4.3, "RCS Pressure and Temperature (P/T) Limits"
 - (2) LCO 3.4.6, "RCS Loops-MODE 4"
 - (3) LCO 3.4.7, "RCS Loops-MODE 5, Loops Filled"
 - (4) LCO 3.4.10, "Pressurizer Safety Valves"
 - (5) LCO 3.4.12, "Low Temperature Overpressure Protection (LTOP)"

- b. The analytical methods used to determine the RCS pressure and temperature limits shall be those previously reviewed and approved by the NRC, specifically those described in the NRC Letters dated October 6, 2000 and July 23, 2001.

- c. The PTLR shall be provided to the NRC upon issuance for each reactor vessel fluence period and for any revision or supplement thereto.



5.6 Reporting Requirements



Section 5.0 Inserts

Insert 5.0-11:

NRC letters dated October 6, 2000 and July 23, 2001.



5.6 Reporting Requirements

5.6.4 CORE OPERATING LIMITS REPORT (COLR) (continued)

- c. The core operating limits shall be determined such that all applicable limits (e.g., fuel thermal mechanical limits, core thermal hydraulic limits, Emergency Core Cooling Systems (ECCS) limits, nuclear limits such as SDM, transient analysis limits, and accident analysis limits) of the safety analysis are met.
- d. The COLR, including any midcycle revisions or supplements, shall be provided upon issuance for each reload cycle to the NRC.

5.6.5 Reactor Coolant System (RCS) PRESSURE AND TEMPERATURE LIMITS REPORT (PTLR)

- a. RCS pressure and temperature limits for heat up, cooldown, low temperature operation, criticality, hydrostatic testing, LTOP enabling, and PORV lift settings as well as heatup and cooldown rates shall be established and documented in the PTLR for the following:
 - (1) LCO 3.4.3, "RCS Pressure and Temperature (P/T) Limits"
 - (2) LCO 3.4.6, "RCS Loops-MODE 4"
 - (3) LCO 3.4.7, "RCS Loops-MODE 5, Loops Filled"
 - (4) LCO 3.4.10, "Pressurizer Safety Valves"
 - (5) LCO 3.4.12, "Low Temperature Overpressure Protection (LTOP)"
- b. The analytical methods used to determine the RCS pressure and temperature limits shall be those previously reviewed and approved by the NRC, specifically those described in the NRC Letters dated October 6, 2000 and July 23, 2001.
- c. The PTLR shall be provided to the NRC upon issuance for each reactor vessel fluence period and for any revision or supplement thereto.

