



Monticello Nuclear Generating Plant
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Monticello, MN 55362-9637

Operated by Nuclear Management
Company LLC

July 27, 2001

US Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Replacement Technical Specification Pages for the Previously Submitted Relocation of
ASME Inservice Testing Requirements to a Licensee Program

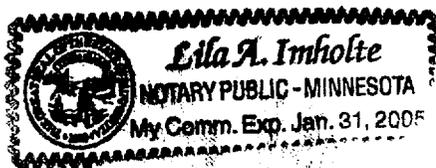
Attached are revised Technical Specification (TS) pages reflecting changes to several TS pages previously submitted to the NRC from the Monticello Nuclear Generating Plant by letter dated May 2, 2001, and supplemented by letter dated June 22, 2001.

Revised pages are being submitted to reflect changes to the Monticello Technical Specifications that have been made since the submittal of the License Amendment Request and Supplement. This submittal contains Table of Contents page iv, and re-pagination of other pages resulting from the NRC approval and issuance of Monticello License Amendment 120.

If you have any questions relating to these revised pages or the previously submitted License Amendment Request regarding the Relocation of ASME Inservice Testing Requirements to a Licensee Program, please contact Doug Neve, Licensing Project Manager (Interim), at (763) 295-1353.

By _____


Jeff S. Forbes
Plant Manager
Monticello Nuclear Generating Plant



Subscribed to and sworn before me this 27th day of July, 2001.


Notary

Attachment: Revised Monticello Technical Specification Pages

cc: (next page)

A047

cc: Regional Administrator-III, NRC
NRR Project Manager, NRC
Sr. Resident Inspector, NRC
Minnesota Department of Commerce
J. Silberg, Esq.

ATTACHMENT

Revised Monticello Technical Specification Pages

Remove pages from IST

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Insert pages in IST

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Bases 3.14/4.14:

The operability of the accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. This capability is consistent with the recommendations of NUREG-0578, "TMI-2 Learned Task Force Status Report and Short Term Recommendations".

8. Limitations on the annual and quarterly air doses resulting from noble gases released in gaseous effluents from the site to areas at or beyond the site boundary, conforming to 10 CFR 50, Appendix I.
9. Limitations on the annual and quarterly doses to a member of the public from iodine-131, iodine-133, tritium, and all radionuclides in particulate form with half lives > 8 days in gaseous effluents released from the site to areas beyond the site boundary, conforming to 10 CFR 50, Appendix I;
10. Limitations on the annual dose or dose commitment to any member of the public, beyond the site boundary, due to releases of radioactivity and to radiation from uranium fuel cycle sources, conforming to 40 CFR 190; and
11. Limitations on venting and purging of the containment through the Standby Gas Treatment System to maintain releases as low as reasonably achievable.

The provisions of Specifications 4.0.B, 4.0.D and 4.0.E are applicable to the Radioactive Effluent Controls Program surveillance frequency.

6.8.E and 6.8.F - RESERVED

G. Inservice Testing Program

This program provides controls for inservice testing of Quality Group A, B, and C pumps and valves which shall be performed in accordance with the requirements of ASME Code Class 1, 2, and 3 pumps and valves, respectively.

1. Testing frequencies specified in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda are as follows:

| ASME Boiler and Pressure Vessel Code and Applicable Addenda Terminology for Inservice Testing Activities | Required Frequencies for Performing Inservice Testing Activities |
|--|---|
| Weekly | At least once per 7 days |
| Monthly | At least once per 31 days |
| Biquarterly | At least once per 46 days |
| Quarterly or every 3 months | At least once per 92 days |
| Semiannually or every 6 months | At least once per 184 days |
| Every 9 months | At least once per 276 days |
| Yearly or annually | At least once per 366 days |
| Biennially or every 2 years | At least once per 731 days |

2. The provisions of Surveillance Requirement 4.0.B are applicable to the Frequencies for performing inservice testing activities;
3. The provisions of Surveillance Requirement 4.0.D and 4.0.E are applicable to inservice testing activities; and
4. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any TS.

6.8.H - RESERVED

I. Explosive Gas and Storage Tank Radioactivity Monitoring Program

This program provides controls for potentially explosive gas mixtures contained in the Offgas Treatment System, the quantity of radioactivity contained in gas storage tanks or fed into the offgas treatment system, and the quantity of radioactivity contained in unprotected outdoor liquid storage tanks. The quantity of radioactivity after 12 hours holdup contained in each gas storage tank shall be limited to $\leq 22,000$ curies of noble gases (considered as dose equivalent Xe-133). The quantity of liquid radioactive material contained in each outside temporary tank shall be limited to ≤ 10 curies, excluding tritium and dissolved or entrained noble gases.

The program shall include:

1. The limits for concentrations of hydrogen and oxygen in the Offgas Treatment System and a surveillance program to ensure the limits are maintained. Such limits shall be appropriate to the system's design criteria (i.e., whether or not the system is designed to withstand a hydrogen explosion);
2. A surveillance program to ensure that the quantity of radioactivity contained in each gas storage tank and fed into the offgas treatment system is less than the amount that would result in a whole body exposure of ≥ 0.5 rem to any individual in an unrestricted area, in the event of an uncontrolled release of the tanks' contents; and
3. A surveillance program to ensure that the quantity of radioactivity contained in all outdoor liquid radwaste tanks that are not surrounded by liners, dikes, or walls, capable of holding the tanks' contents and that do not have tank overflows and surrounding area drains connected to the Liquid Radwaste Treatment System is less than the amount that would result in concentrations less than the limits of 10 CFR 20, Appendix B, Table 2, Column 2, at the nearest potable water supply and the nearest surface water supply in an unrestricted area, in the event of an uncontrolled release of the tanks' contents.

The provisions of Specifications 4.0.B, 4.0.D and 4.0.E are applicable to the Explosive Gas and Storage Tank Radioactivity Monitoring Program surveillance frequencies.

6.8.J - RESERVED

K. Technical Specifications (TS) Bases Control Program

This program provides a means for processing changes to the Bases of these Technical Specifications.

1. Changes to the Bases of the TS shall be made under appropriate administrative controls and reviews.
2. Changes to Bases may be made without prior NRC approval provided the changes do not involve either of the following:
 - a. a change in the TS incorporated in the license; or
 - b. a change to the USAR or Bases that requires NRC approval pursuant to 10 CFR 50.59.
3. The Bases Control Program shall contain provisions to ensure that the Bases are maintained consistent with the USAR.
4. Proposed changes to the Bases that involve changes as described in a. or b. of Specification 6.8.K.2 above shall be reviewed and approved by the NRC prior to implementation. Changes to the Bases implemented without prior NRC approval shall be provided to the NRC on a frequency consistent with 10 CFR 50.71(e).