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NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

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March 11, 1983

Dockets Nos. 50-269, 50-270  
and 50-287

Posted  
Amdt. 119  
to DPR-47

Mr. H. B. Tucker, Vice President  
Nuclear Production Department  
Duke Power Company  
P. O. Box 33189  
422 South Church Street  
Charlotte, North Carolina 28242

Dear Mr. Tucker:

The Commission has issued the enclosed Amendments Nos. 119, 119, and 116 to Licenses Nos. DPR-38, DPR-47 and DPR-55 for the Oconee Nuclear Station, Units Nos. 1, 2 and 3. These amendments consist of changes to the Station's common Technical Specifications (TSS) in response to your request dated November 12, 1982, as supplemented on February 24, 1983.

These amendments revise the TSS concerning the heatup, cooldown and inservice test limitations for the reactor coolant systems of each Oconee unit.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

*Eben L. Conner*

Eben L. Conner, Project Manager  
Operating Reactors Branch #4  
Division of Licensing

Enclosures:

1. Amendment No. 119 to DPR-38
2. Amendment No. 119 to DPR-47
3. Amendment No. 116 to DPR-55
4. Safety Evaluation
5. Notice

cc w/enclosures:  
See next page

Duke Power Company

cc w/enclosure(s):

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 119  
License No. DPR-38

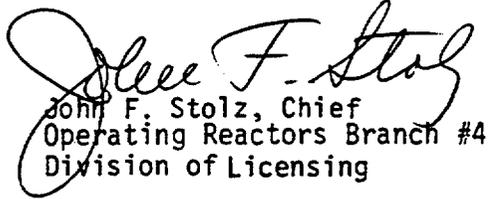
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Duke Power Company (the licensee) dated November 12, 1982, as supplemented February 24, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-38 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 119, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment becomes effective on March 14, 1983.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: **MAR 11 1983**



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-270

OCONEE NUCLEAR STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 119  
License No. DPR-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Duke Power Company (the licensee) dated November 12, 1982, as supplemented February 24, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-47 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 119, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment becomes effective on March 14, 1983.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: MAR 11 1983



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50- 287

OCONEE NUCLEAR STATION, UNIT NO.3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 116  
License No. DPR-55

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Duke Power Company (the licensee) dated November 12, 1982, as supplemented February 24, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-55 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 116, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment becomes effective on March 14, 1983.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: MAR 11 1983

ATTACHMENTS TO LICENSE AMENDMENTS

AMENDMENT NO. 119 TO DPR-38

AMENDMENT NO. 119 TO DPR-47

AMENDMENT NO. 116 TO DPR-55

DOCKETS NOS. 50-269, 50-270 AND 50-287

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment numbers and contain vertical lines indicating the area of change.

<u>Remove Pages</u>	<u>Insert Pages</u>
vi	vi
3.1-3	3.1-3
3.1-3a	3.1-3a
3.1-5	3.1-5
--	3.1-5a
--	3.1-5b
3.1-6	3.1-6
3.1-6a	3.1-6a
3.1-6b	3.1-6b
3.1-7	3.1-7
3.1-7a	3.1-7a
3.1-7b	3.1-7b
3.1-7c	3.1-7c
3.1-7d	3.1-7d
3.1-7e	3.1-7e

LIST OF TABLES

<u>Table No.</u>		<u>Page</u>
2.3-1A	Reactor Protective System Trip Setting Limits - Unit 1	2.3-11
2.3-1B	Reactor Protective System Trip Setting Limits - Unit 2	2.3-12
2.3-1C	Reactor Protective System Trip Setting Limits - Unit 3	2.3-13
3.1-1	Operational Guidance for Plant Heatup	3.1-5a
3.1-2	Operational Guidance for Plant Cooldown	3.1-5b
3.5-1-1	Instruments Operating Conditions	3.5-4
3.5-1	Quadrant Power Tilt Limits	3.5-14
3.7-1	Operability Requirements for the Emergency Power Switching Logic Circuits	3.7-13
3.17-1	Fire Protection & Detection Systems	3.17-5
4.1-1	Instrument Surveillance Requirements	4.1-3
4.1-2	Minimum Equipment Test Frequency	4.1-9
4.1-3	Minimum Sampling Frequency	4.1-10
4.2-1	Oconee Nuclear Station Capsule Assembly Withdrawal Schedule at Crystal River Unit No. 3	4.2-3
4.4-1	List of Penetrations with 10CFR50 Appendix J Test Requirements	4.4-6
4.11-1	Oconee Environmental Radioactivity Monitoring Program	4.11-3
4.11-2	Offsite Radiological Monitoring Program	4.11-4
4.11-3	Analytical Sensitivities	4.11-5
4.17-1	Steam Generator Tube Inspection	4.17-6
6.1-1	Minimum Operating Shift Requirements with Fuel in Three Reactor Vessels	6.1-6
6.6-1	Report of Radioactive Effluents	6.6-8

3.1.2 Pressurization, Heatup, and Cooldown Limitation

Specification

- 3.1.2.1 The reactor coolant pressure and the system heatup and cooldown rates (with the exception of the pressurizer) shall be limited as follows:

Heatup:

Heatup rates and allowable combinations of pressure and temperature shall be limited in accordance with Table 3.1-1 and Figure

3.1.2-1A Unit 1

3.1.2-1B Unit 2

3.1.2-1C Unit 3

Cooldown:

Cooldown rates and allowable combinations of pressure and temperature shall be limited in accordance with Table 3.1-2 and Figure

3.1.2-2A Unit 1

3.1.2-2B Unit 2

3.1.2-2C Unit 3

- 3.1.2.2 Leak tests required by Specification 4.3 and ASME Section XI shall be limited to the heatup and cooldown rates and allowable combinations of pressure and temperature provided in Tables 3.1-1, 3.1-2 and Figure 3.1.2-3A Unit 1  
3.1.2-3B Unit 2  
3.1.2-3C Unit 3
- 3.1.2.3 For thermal steady state system hydro tests required by ASME Section XI the system may be pressurized to the limits set forth in Specification 2.2 and 3.1.2.2.
- 3.1.2.4 The secondary side of the steam generator shall not be pressurized above 237 psig if the temperature of the vessel shell is below 110°F.
- 3.1.2.5 The pressurizer heatup and cooldown rates shall not exceed 100°F/hr. The spray shall not be used if the temperature difference between the pressurizer and the spray fluid is greater than 410°F.

3.1.2.6 Prior to exceeding fifteen (Unit 1)  
fifteen (Unit 2)  
fifteen (Unit 3)

effective full power years of operation.

Figures 3.1.2-1A (Unit 1), 3.1.2-2A (Unit 1)  
3.1.2-1B (Unit 2), 3.1.2-2B (Unit 2)  
3.1.2-1C (Unit 3), 3.1.2-2C (Unit 3)

and 3.1.2-3A (Unit 1)  
3.1.2-3B (Unit 2)  
3.1.2-3C (Unit 3)

and Technical Specification 3.1.2.1, 3.1.2.2 and 3.1.2.3 shall be updated for the next service period in accordance with 10 CFR 50, Appendix G, Section V.B and V.E.

3.1.2.7 The updated proposed technical specification referred to in 3.1.2.6 shall be submitted for NRC review at least 90 days prior to the end of the service period for Units 1, 2 and 3.

limitations of 110°F and 237 psig are based on the highest estimated RT<sub>NDT</sub> of +40°F and the preoperational system hydrostatic test pressure of 1312 psig. The average metal temperature is assumed to be equal to or greater than the coolant temperature. The limitations include margins of 25 psi and 10°F for possible instrument error.

The spray temperature difference is imposed to maintain the thermal stresses at the pressurized spray line nozzle below the design limit.

#### REFERENCES

- (1) Analysis of Capsule OCII-A from Duke Power Company Oconee Unit 2 Reactor Vessel Materials Surveillance Program, BAW-1699, December 1981.
- (2) Analysis of Capsule OCIII-B from Duke Power Company Oconee Unit 3 Reactor Vessel Materials Surveillance Program, BAW-1697, October 1981.
- (3) Analysis of Capsule OCI-E from Duke Power Company Oconee Unit 1 Reactor Vessel Materials Surveillance Program, BAW-1436, September, 1977.

TABLE 3.1-1

OPERATIONAL GUIDANCE FOR PLANT HEATUP

I. RC Temperature Constraints

RC Temperature	Maximum Heatup Rate
T < 280°F	50°F/HR
T > 280°F	100°F/HR

II. RC Pump Constraints

None

TABLE 3.1-2

OPERATIONAL GUIDANCE FOR PLANT COOLDOWN

I. RC Temperature Constraints

RC Temperature <sup>(1)</sup>	Maximum Cooldown Rate <sup>(2)</sup>
T > 280°F	≤ 50°F in any ½ hour period
150°F < T < 280°F	≤ 25°F in any ½ hour period
T < 150°F	≤ 10°F in any 1 hour period
RCS depressurized <sup>(3)</sup>	≤ 50°F in any 1 hour period

(1) RC temperature is cold leg temperature if one or more RC pumps are in operation or if on natural circulation cooldown; otherwise it is the LPI cooler outlet temperature.

(2) These rate limits must be applied to the change in temperature indication from cold leg temperature to LPI cooler outlet temperature per Note (1).

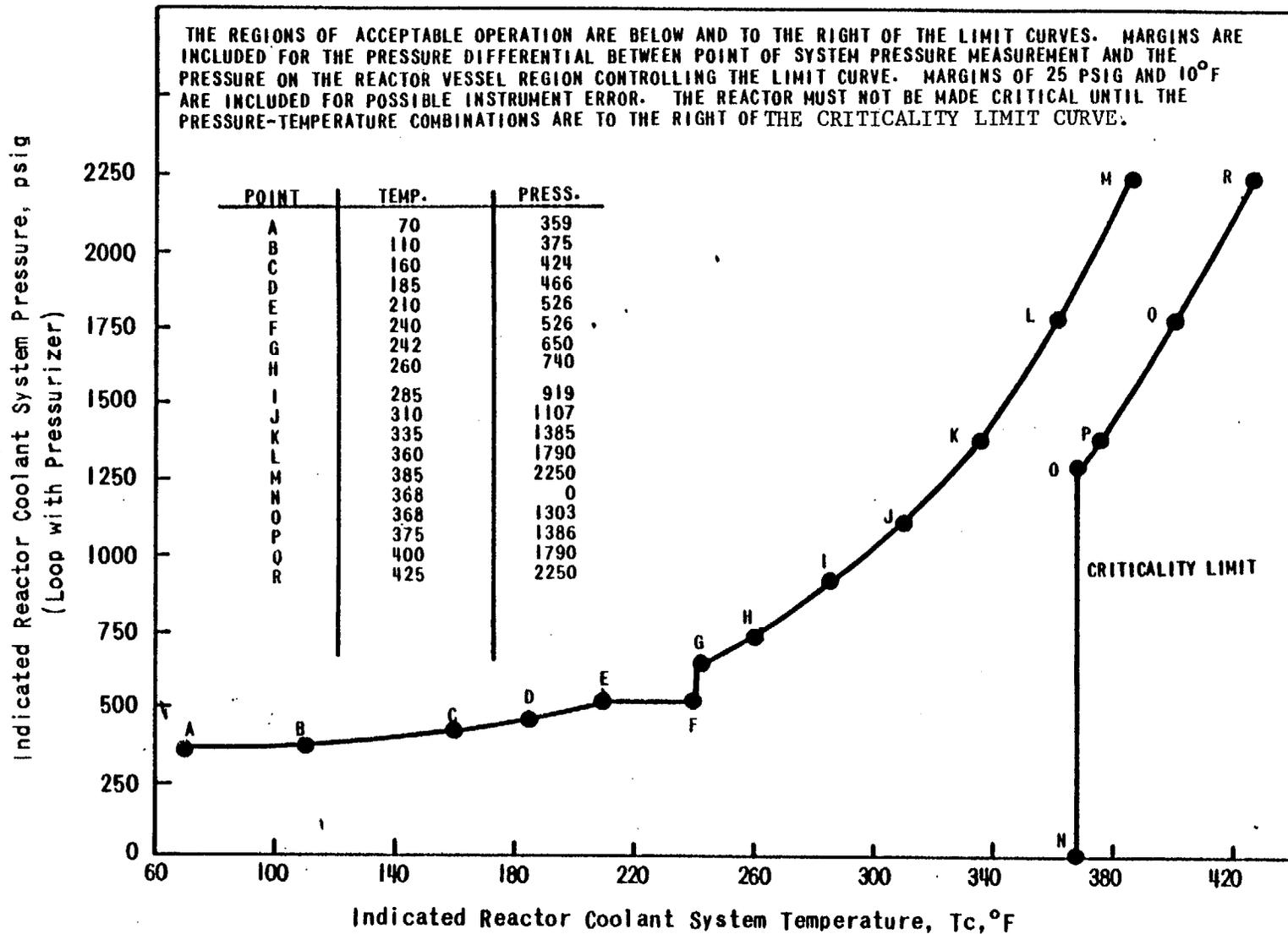
(3) When the RCS is depressurized such that all three of the following conditions exist:

- a) RCS temperature < 200°F,
- b) RCS pressure < 50 psig,
- c) All RC Pumps off,

the maximum cooldown rate shall be relaxed to ≤ 50°F in any 1 hour period.

II. RC Pump Constraints For Validity of Guidance

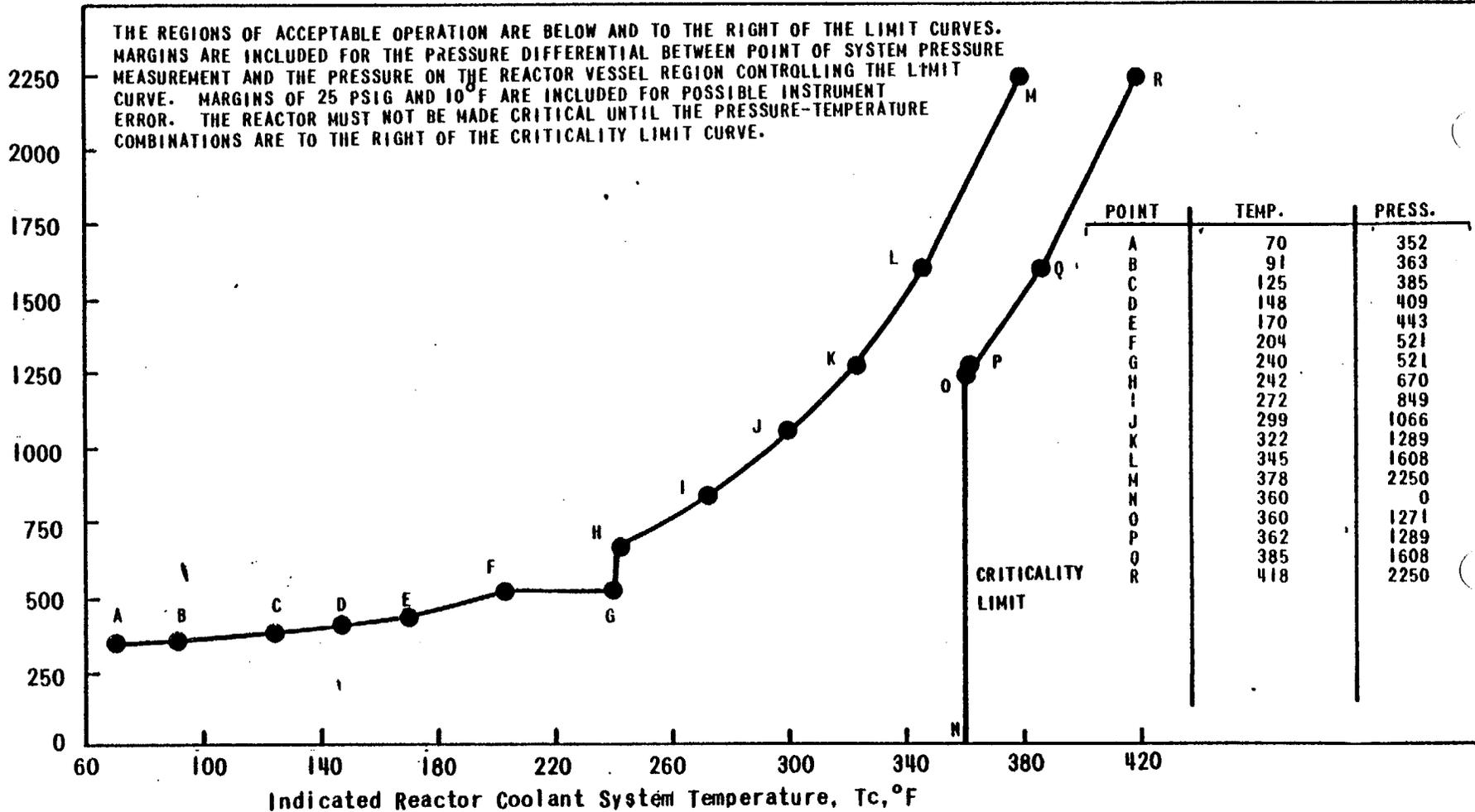
RC Temperature	Allowed Pump Combinations
> 270°F	Any
270-200°F	No more than 1 pump per loop
< 200°F	No more than 1 pump



UNIT 1 OCONEE NUCLEAR STATION  
 REACTOR COOLANT SYSTEM NORMAL OPERATION-HEATUP  
 LIMITATIONS APPLICABLE FOR THE FIRST 15 EPY

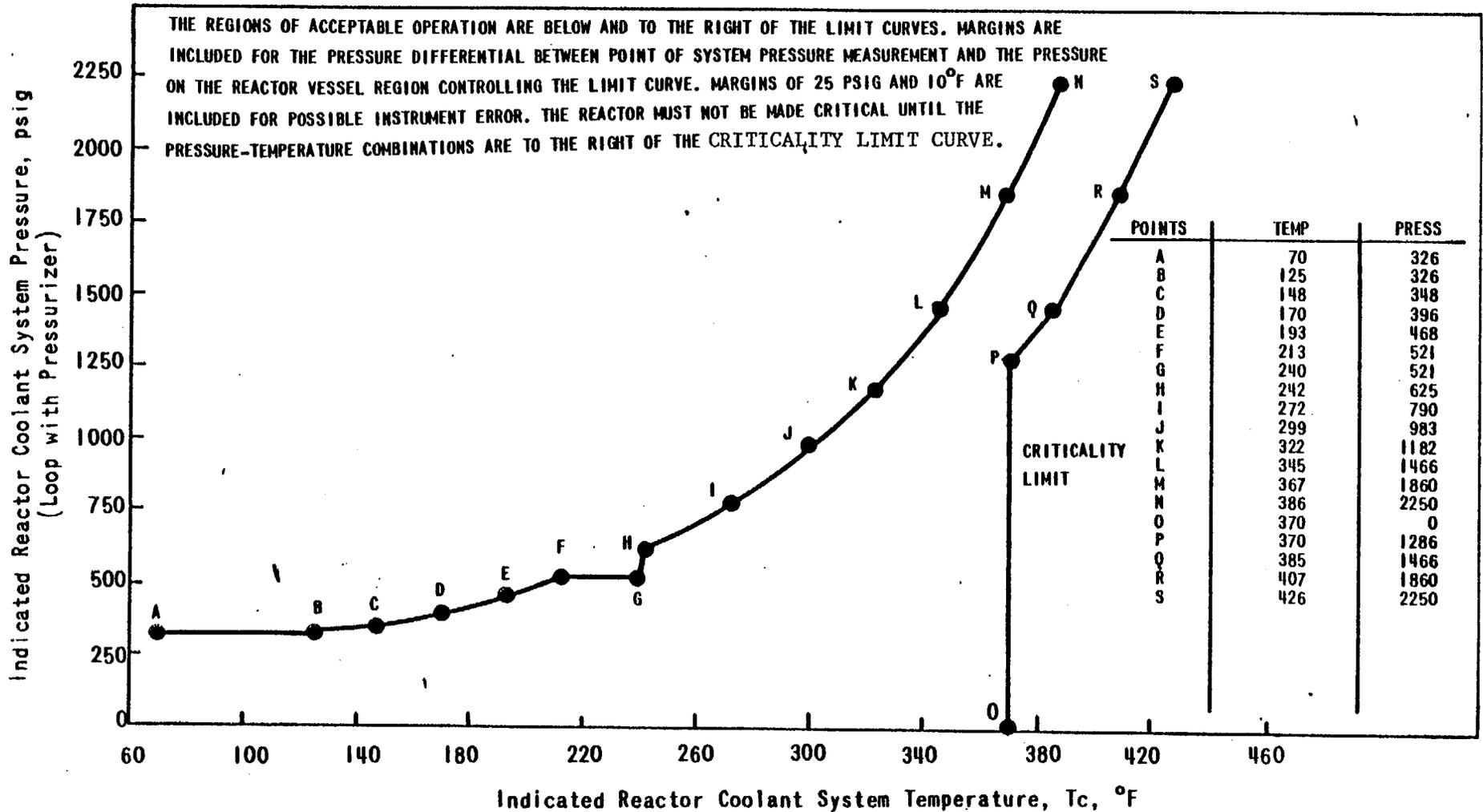
Figure 3.1.2-1A

Indicated Reactor Coolant System Pressure, psig  
(Loop with Pressurizer)



UNIT 2 OCONEE NUCLEAR STATION  
REACTOR COOLANT SYSTEM NORMAL OPERATION-  
HEATUP LIMITATIONS APPLICABLE FOR FIRST 15.0 EFY

Figure 3.1.2-1B



UNIT 3 OCONEE NUCLEAR STATION  
 REACTOR COOLANT SYSTEM NORMAL OPERATION-HEATUP  
 LIMITATIONS APPLICABLE FOR FIRST 15.0 EFPY

Figure 3.1.2-1C

Indicated Reactor Coolant System Pressure, psig  
(Loop with Pressurizer)

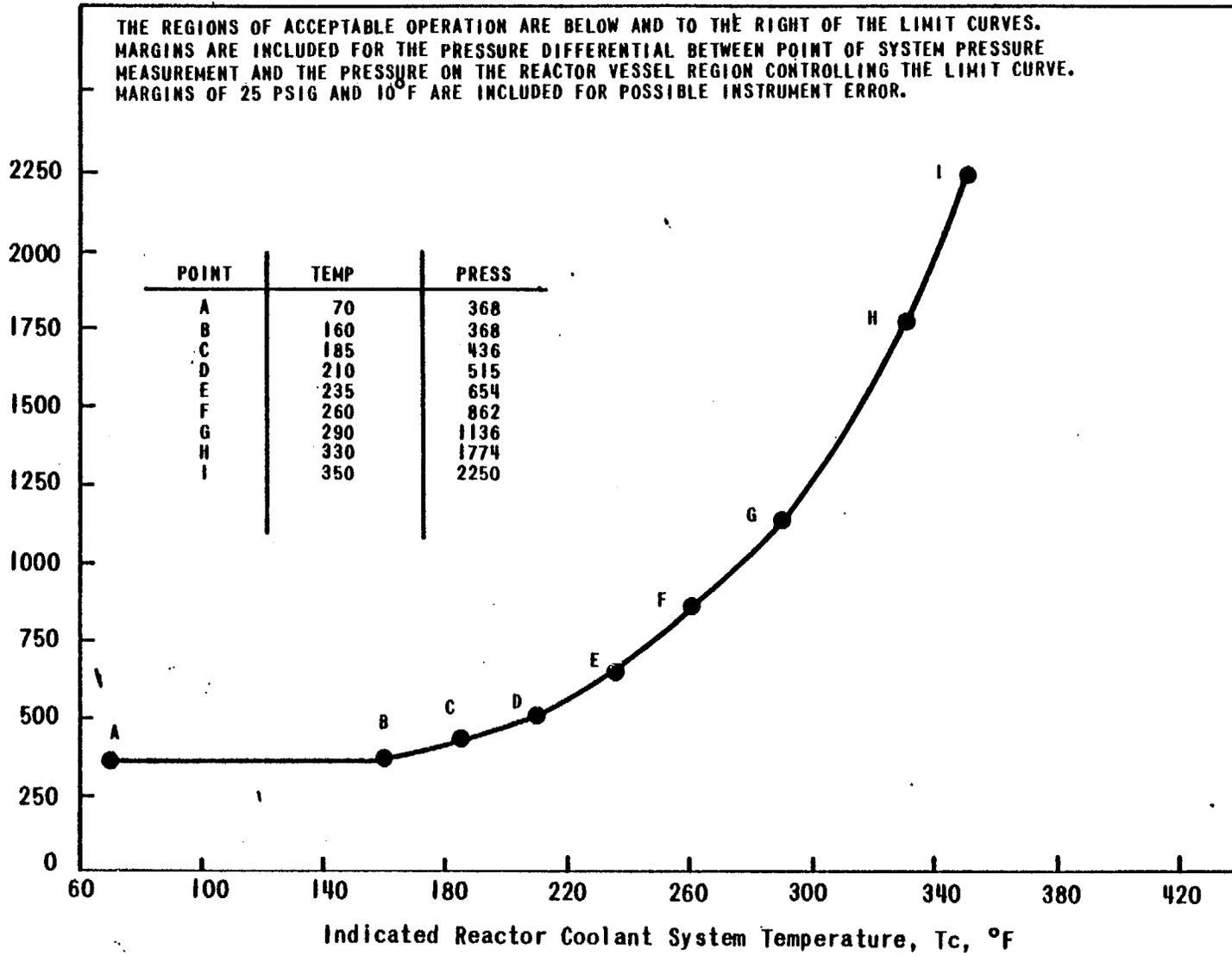
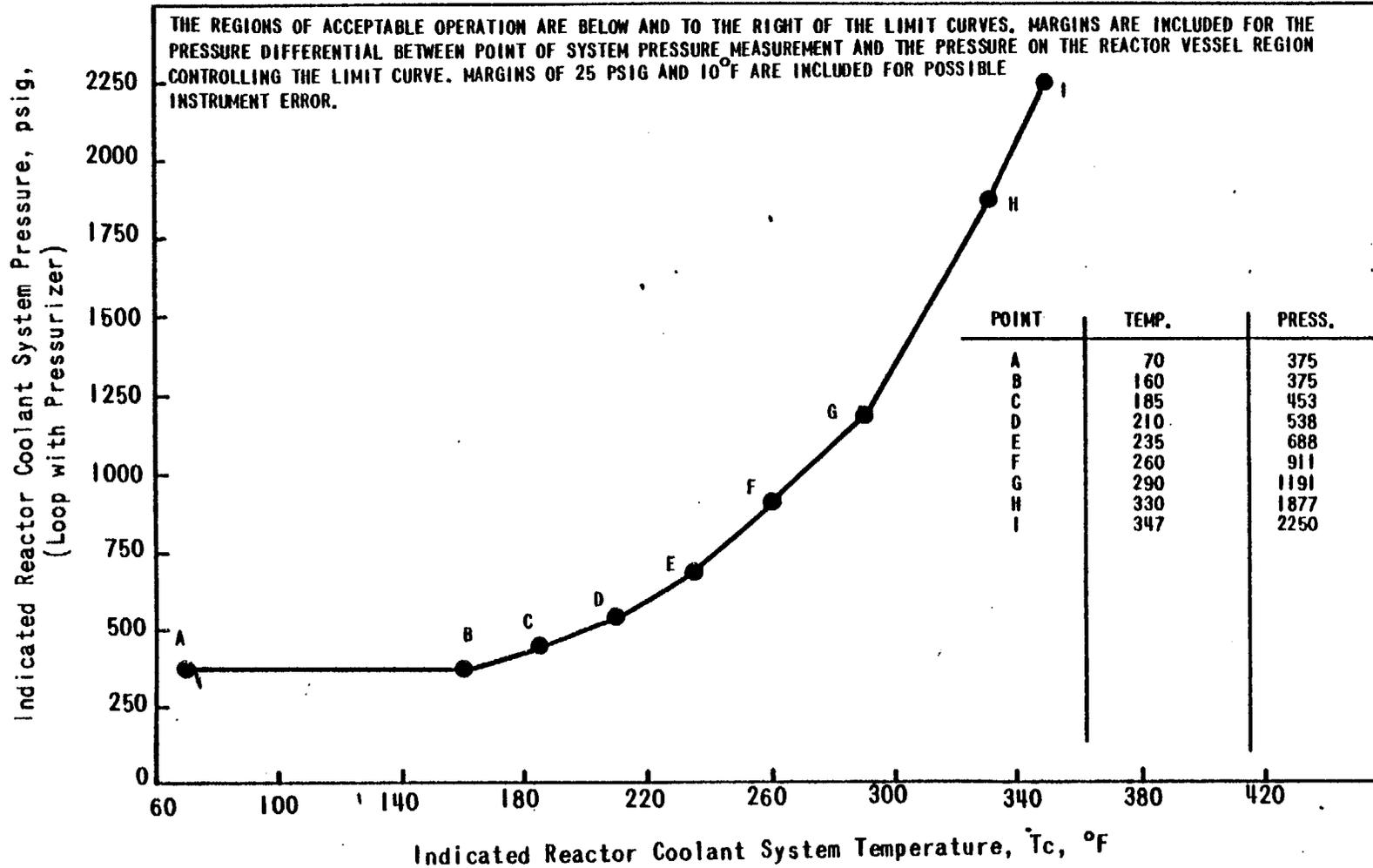


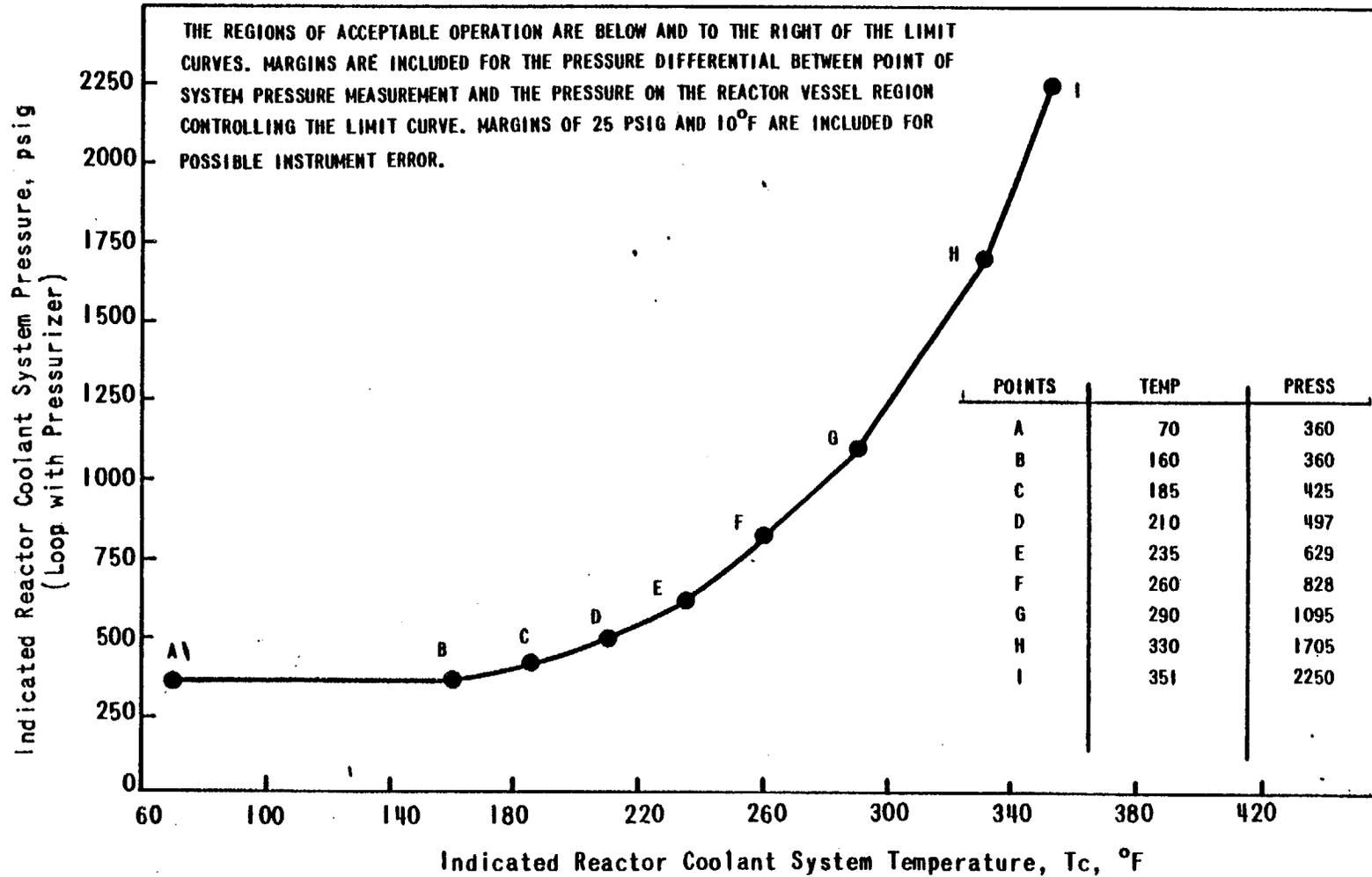
Figure 3.1.2-2A

UNIT 1 OCONEE NUCLEAR STATION  
REACTOR COOLANT SYSTEM NORMAL OPERATION-  
COOLDOWN LIMITATIONS APPLICABLE FOR  
FIRST 15 EFPY



UNIT 2 OCONEE NUCLEAR STATION  
 REACTOR COOLANT SYSTEM NORMAL OPERATION-COOLDOWN  
 LIMITATIONS APPLICABLE FOR FIRST 15.0 EFPY

Figure 3.1.2-2B



UNIT 3 OCONEE NUCLEAR STATION  
 REACTOR COOLANT SYSTEM NORMAL OPERATION-COOLDOWN  
 LIMITATIONS APPLICABLE FOR FIRST 15.0 EFY

Figure 3.1.2-2C

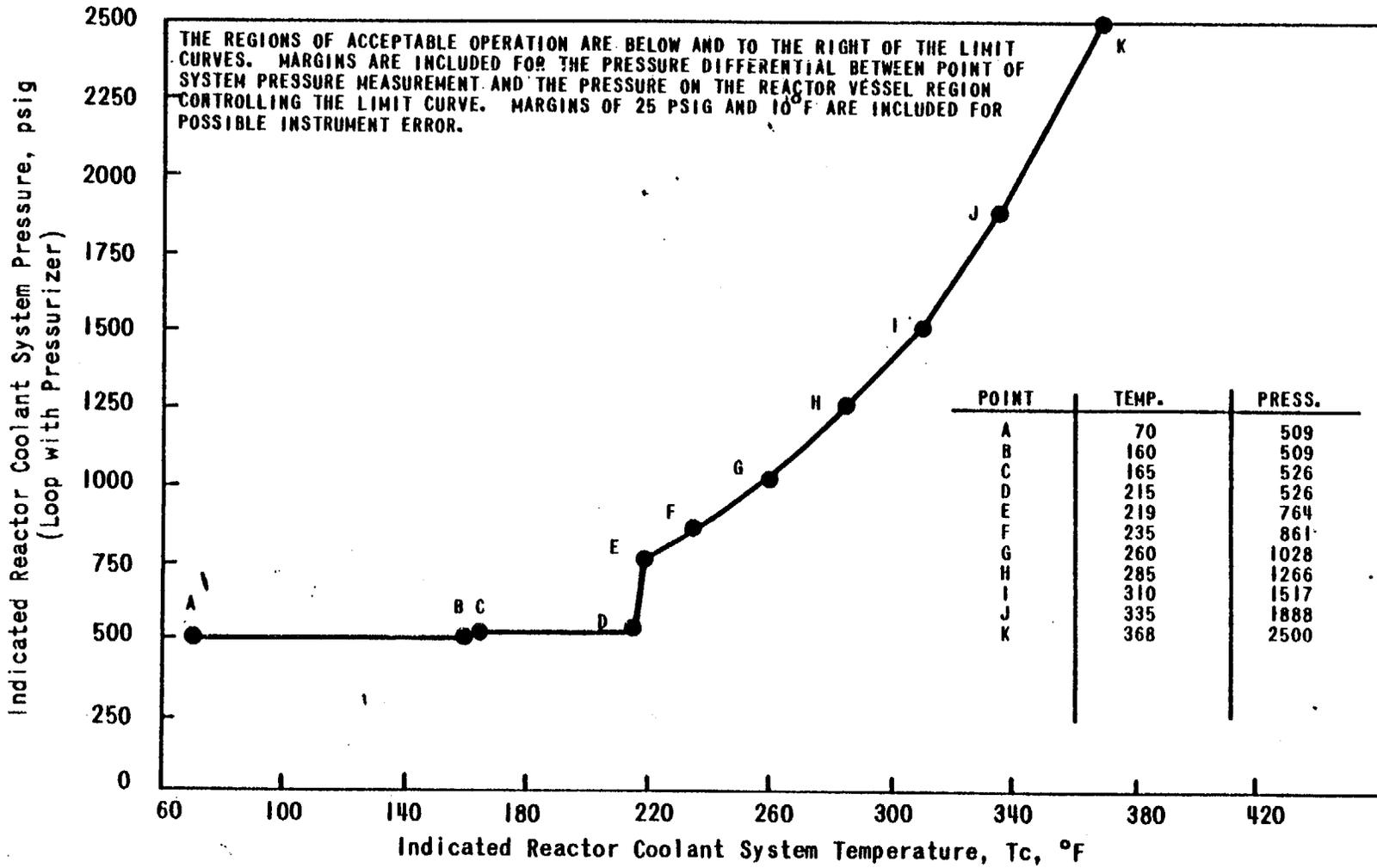
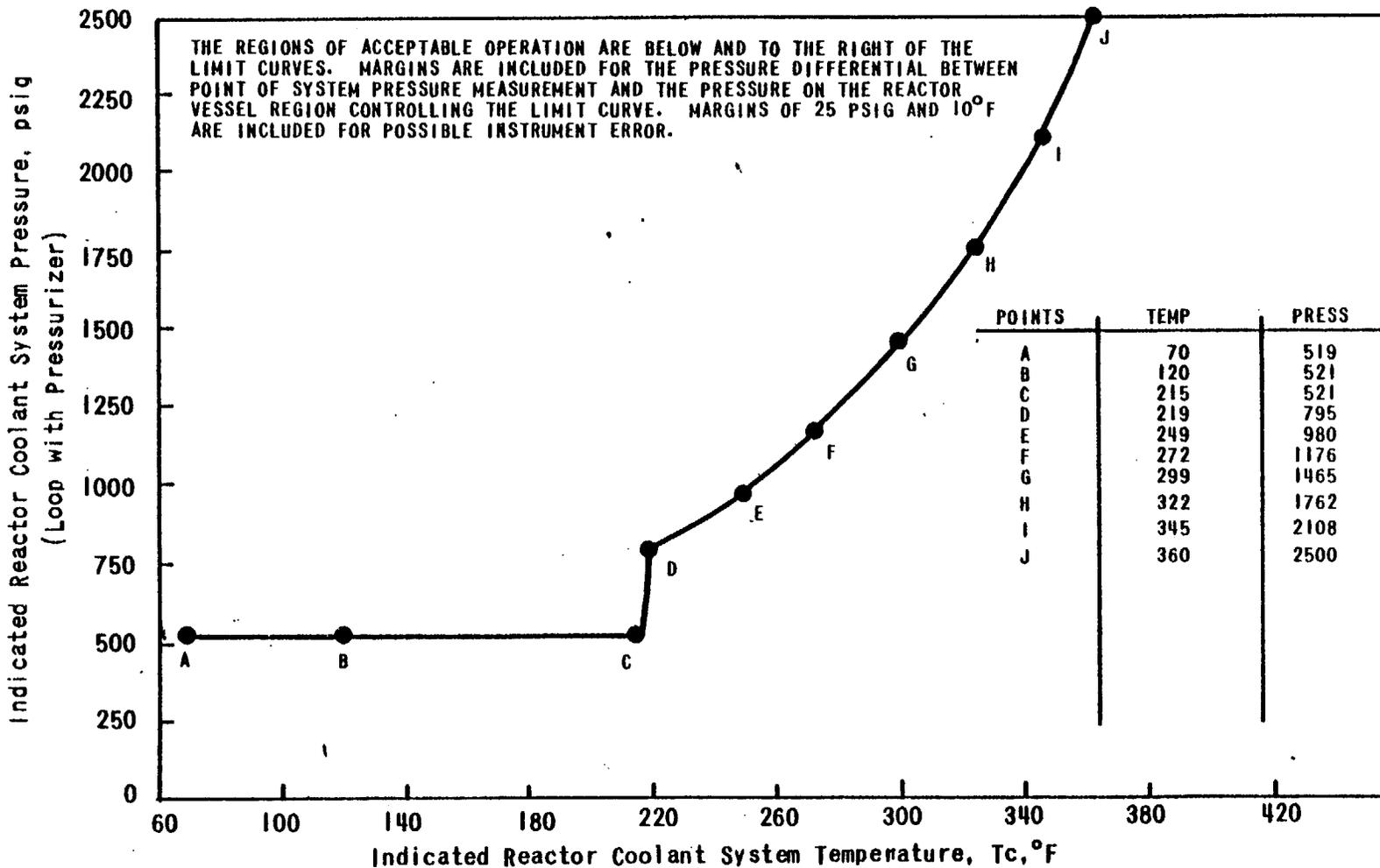


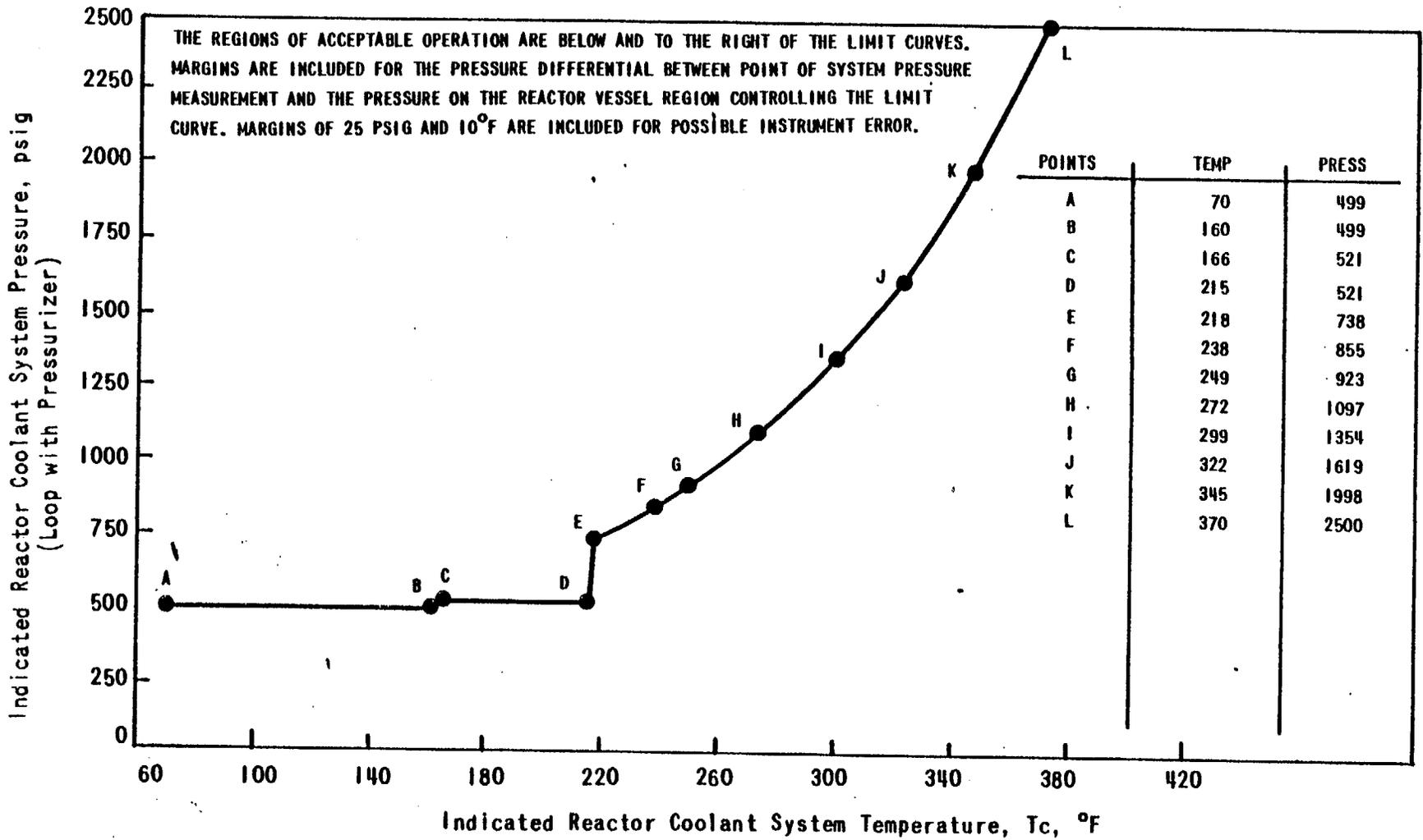
Figure 3.1.2-3A

UNIT 1 OCONEE NUCLEAR STATION  
 REACTOR COOLANT SYSTEM INSERVICE LEAK AND  
 HYDROSTATIC TEST HEATUP AND COOLDOWN LIMITATION  
 APPLICABLE FOR FIRST 15.0 EFYP



UNIT 2 OCONEE NUCLEAR STATION  
 REACTOR COOLANT SYSTEM INSERVICE LEAK AND  
 HYDROSTATIC TEST HEATUP AND COOLDOWN  
 LIMITATIONS APPLICABLE FOR 15.0 EFPY

Figure 3.1.2-3B



Indicated Reactor Coolant System Temperature, Tc, °F

UNIT 3 OCONEE NUCLEAR STATION  
 REACTOR COOLANT SYSTEM INSERVICE LEAK AND  
 HYDROSTATIC TEST HEATUP & COOLDOWN LIMITATIONS FOR FIRST  
 15.0 EFPY

Figure 3.1.2-3C



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 119 TO FACILITY OPERATING LICENSE NO. DPR-38

AMENDMENT NO. 119 TO FACILITY OPERATING LICENSE NO. DPR-47

AMENDMENT NO. 116 TO FACILITY OPERATING LICENSE NO. DPR-55

DUKE POWER COMPANY

OCONEE NUCLEAR STATION, UNITS NOS. 1, 2 AND 3

DOCKETS NOS. 50-269, 50-270 AND 50-287

Introduction

By letter dated November 12, 1982, as revised on February 24, 1983, Duke Power Company (DPC or the licensee) proposed a change to the Oconee Nuclear Station, Units 1, 2 and 3 Technical Specifications (TSs). This change is a revision to the reactor vessel pressure-temperature limits.

Background

The licensee indicated that the bases for the proposed pressure-temperature limits were the material properties data in Babcock & Wilcox (B&W) Reports BAW-1697 and BAW-1699. The curves for each Oconee reactor vessel are to be valid for 15 effective full power years (EFPY).

The B&W Reports BAW-1697 and BAW-1699 contain the B&W analysis of reactor vessel material surveillance capsules OC III-B and OC II-A, respectively. These capsules are part of the B&W Owners Group Integrated Surveillance Program. As a result, the capsules were irradiated in both the Oconee and Crystal River 3 reactor vessels.

Evaluation

A comparison of the materials in the Oconee 1, 2 and 3 reactor vessels and the OC III-B and OC II-A capsules indicates that the limiting weld material in the Oconee 1, 2 and 3 reactor vessels is not contained in the OC III-B and OC II-A capsules. The limiting material in the Oconee 1, 2 and 3 reactor vessels is weld material SA 1430, WF 24, and WF 67, respectively. The weld materials in OC III-B and OC II-A are WF 209-1B and WF 209-1A, respectively. Although the weld materials in the vessel and the capsules are not identical, they were prepared by the same manufacturer, using the same type of wire and flux and heat treated to an equivalent metallurgical condition. As a result, the fracture toughness data from capsules OC III-B and OC II-A may be utilized for evaluating the proposed pressure-temperature limits.

The change in upper shelf energy (USE) and reference temperature resulting from neutron irradiation damage of the limiting materials in the OC III-B and OC II-A capsules are compared in Table 1 to the values predicted by Regulatory Guide 1.99, Rev. 1, "Effects of Residual Elements on Predicted Radiation Damage to Reactor Vessel Materials", and the values predicted by B&W Report BAW-1511P dated October 1980. This comparison indicates that the Regulatory Guide 1.99 method for predicting change in  $RT_{NDT}$  resulting from neutron irradiation damage is conservative. In addition, the method in Figure 13 of B&W Report BAW-1511P for predicting the change in weld material USE properties resulting from neutron irradiation damage is more accurate than the method in Regulatory Guide 1.99. Hence, we utilized Regulatory Guide 1.99 methodology for estimating the change in vessel material  $RT_{NDT}$ , and Figure 3 in B&W Report BAW-1511P for estimating the change in reactor vessel material USE. We believe that Figure 3 in B&W Report BAW-1511P is more accurate than Regulatory Guide 1.99 for estimating the change in USE resulting from irradiation damage for Oconee vessel and surveillance weld materials because Figure 3 in B&W Report BAW-1511P was generated from reactor vessel surveillance weld materials similar to the Oconee vessel and surveillance weld materials, and the Regulatory Guide 1.99 curve was generated from reactor vessel weld materials utilized throughout the nuclear industry.

The USE for the Oconee beltline materials must exceed 50 ft-lbs at the 1/4 thickness location in order to meet the safety margins required by Paragraph IV.A.2 of Appendix G, 10 CFR Part 50. Using Figure 3 in B&W Report BAW-1511P, we estimate that the limiting materials in Oconee 1, 2 and 3 reactor vessel beltlines will have USE less than 50 ft-lbs at the 1/4 thickness location when their neutron fluence ( $E > 1\text{MeV}$ ) exceeds  $5 \times 10^{18}\text{n/cm}^2$ ,  $4.8 \times 10^{18}\text{n/cm}^2$  and  $7.5 \times 10^{18}\text{n/cm}^2$ , respectively. Based on the neutron fluence estimated by the licensee for each beltline material and the uncertainty in vessel dosimetry identified by B&W\*, we conclude that the USE energy at the 1/4 thickness location for the Oconee beltline reactor vessel materials will exceed 50 ft-lbs for the period of time that the proposed pressure-temperature curves are applicable.

Using the method for predicting shift in  $RT_{NDT}$  in Regulatory Guide 1.99, Rev. 1, the neutron fluence estimates of the licensee, the unirradiated material properties in B&W Reports BAW-1511P, October 1980, and BAW-10046P, March 1976, and the method of calculating pressure-temperature limits identified in Standard Review Plan Section 5.3.2, the proposed pressure-temperature limit curves for Oconee Units 1, 2 and 3 meet the safety margins of Appendix G, 10 CFR Part 50, and are acceptable for 15 EFPY.

\*C. Whitmarsh, Draft B&W Report to be Published.

Table 1

Comparison of Change in Properties for OC III-B and  
OC II-A Capsule Weld Materials

	Change in RT <sub>NDT</sub> (°F)			Change in USE (Percentage)			*
	Capsule	Reg. 1.99	BAW-1511P	Capsule	Reg. 1.99	BAW-1511P	
WF 209-1B	89	170	N/A	24	34	24.5	
WF-209-1A	104	226	N/A	28	36	27	

\* Estimated per Figure 3, page C-10 of B&W Report BAW-1511P, October 1980.

### Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

### Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of an accident previously evaluated, do not create the possibility of an accident of a type different from any evaluated previously, and do not involve a significant reduction in a margin of safety, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: **MAR 11 1983**

The following NRC personnel have contributed to this Safety Evaluation:  
L. Lois, B. Elliot, E. Conner.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKETS NOS. 50-269, 50-270 AND 50-287DUKE POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY  
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 119, 119 and 116 to Facility Operating Licenses Nos. DPR-38, DPR-47 and DPR-55, respectively, issued to Duke Power Company, which revised the Technical Specifications (TSs) for operation of the Oconee Nuclear Station, Units Nos. 1, 2 and 3, located in Oconee County, South Carolina. The amendments become effective on March 14, 1983.

These amendments revise the TSs concerning the heatup, cooldown and inservice test limitations for the reactor coolant systems of each Oconee unit.

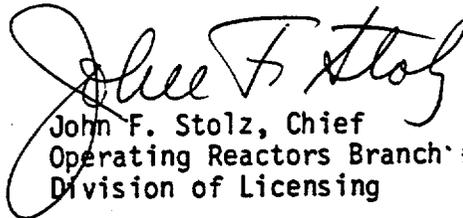
The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated November 12, 1982, as supplemented February 24, 1983, (2) Amendments Nos. 119, 119, and 116 to Licenses Nos. DPR-38, DPR-47 and DPR-55, respectively, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Oconee County Library, 501 West Southbroad Street, Walhalla, South Carolina 29691. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 11th day of March 1983.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing