Specialty Chemicals
Honeywell
Route 45 North
P.O. Box 430
Metropolis, IL 62960
618 524-2111
618 524-6239 Fax

July 20, 2001

Certified Mail:
2617-1992

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk,
Washington, D.C. 20555

Re: Shipment of CaF2 Pond Materials
License No. SUB-526 / Docket No. 40-03392

Dear Sirs:

The Hastie Trucking & Mining Company, Cave-In-Rock, Illinois would like to purchase the synthetic fluorspar and lime mix from the Environmental Protection Facility (EPF) ponds (surface impoundments). Hastie Mining became aware of this material through another trucking company because Honeywell is closing out 'A' Pond and shipping the material to Texas for disposal. This current project has been approved by the Illinois EPA, letter dated February 23, 2001, and the USNRC, letter dated September 12, 2000. Hastie Mining could use this material as one of the ingredients to be blended with natural fluorspar and to manufacture a fluorspar briquette for the steel industry. The Honeywell pond material blended at a ratio of 50% with natural fluorspar enhances the formation of the briquettes. This pond material has value and will reduce the use of raw materials to make the fluorspar briquettes. Honeywell understands at this time it may be too late to obtain all of the necessary corporate and regulatory approvals to transfer the material from 'A' pond to Hastie Mining; but there are four other ponds that will require closure within the near future.

The synthetic fluorspar and lime mix from the EPF ponds does contain uranium that is greater than 212 pCi/gm (313 PPM). The average uranium concentration of the four remaining ponds is 381 pCi/gm (562 PPM). The blending of natural fluorspar with pond material will increase the fluorspar concentration from approximately 70% CaF_2 to 90% + before it is processed into briquettes. Blending of this material will take place in two stages. End dump trailers will be loaded with natural fluorspar to a weight of 12 tons at Hastie Mining. These trailers will then be sent to Honeywell's Metropolis Works and topped off with an additional 12 tons of pond material and shipped back to Hastie Mining for off-loading at the pug mill for the final blending prior to manufacturing briquettes.

Using this two-stage procedure for blending will provide the following benefits:

1. It allows Honeywell to ship this material in end dump trailers without using trailer liners. Hastie Mining does not want the added expense, labor, and cost for disposal

NMSSOIPublic

of liners. By pre-loading each dump trailer with natural fluorspar, it will allow the pond material to off-load without using trailer liners. The pond material contains approximately 25% water and has a consistency of "tooth paste" and would not dump without using a liner.

2. The dose calculation for the backhoe operator at the 'A' Pond closure demonstrated compliance for the EPA 25 mrem/year dose limit. The total effective dose equivalent would not exceed 6 mrem/year. This was demonstrated to the NRC for closure of 'A' Pond in July 2001. The remaining four ponds would have a similar dose calculated over a four or five-year period for closure.

3. The Honeywell pond material should not create any dusting due to the high moisture content. This is beneficial for removing the material from the ponds as well as unloading the end dump trailers. It will also eliminate any dusting at the pug mill to complete a homogenous product.

4. The first stage of blending the pond material with natural fluorspar in the trailer would reduce the average uranium concentration to 191 pCi/gm (281 PPM). This average uranium concentration in the pond material and natural fluorspar in the dump trailers leaving the Honeywell site would be below the current NRC license condition number 13 of 212 pCi/gm (313 PPM) for synthetic calcium fluoride. Honeywell will perform employee dose calculations at Hastie Mining briquette facility within the first year of operation to demonstrate regulatory compliance.

Summary:

The Hastie Trucking & Mining Company is one of the last active fluorspar mines in the United States. They supply 800 to 1000 tons weekly of fluorspar to various industries at the present time. Most of their fluorspar comes from purchasing government stockpiles and selling it with fluorspar mined at Cave-In-Rock. This government source of natural fluorspar is coming to an end. The current business practice allows them to be more competitive with the imported fluorspar from China and Mexico, which has glutted the market with lower price product. They have already blended the synthetic calcium fluoride with natural fluorspar for a flux material at the steel mills. The manufacture of briquettes using the Honeywell materials will produce a higher quality product when blended with the natural fluorspar. The new briquette facility will enable Hastie to provide a quality product that will not breakdown using the materials from our calcium fluoride ponds.

An additional safety factor is the close proximity of the Hastie Mining facility. This site is located within 60 miles of Honeywell and will reduce the possibility of a transportation incident for hauling materials. Local trucking companies that are already familiar with these products will be used for transportation. It also keeps a non-hazardous material out of the hazardous waste burial site; the low levels of uranium should not prohibit the finished product from being used as fluxing agent in the steel industry.

The U.S. Nuclear Regulatory Commission found no significant impact on using synthetic CaF_2 in briquette plants when it is blended with natural fluorspar. This information was

published in the Federal Register, January 22, 1992. The CaF_2 Pond material contains lime, which does reduce the concentration of the synthetic CaF2; but when mixed with lime it does produce a higher quality product when blended with natural fluorspar. The uranium concentration will be maintained below the 212 pCi/gm in the finished briquette as outlined in the Federal Register, dated January 22, 1992 for synthetic calcium fluoride blended with natural fluoride.

We have made a review of the NRC regulation and cannot find any regulation that would prohibit Honeywell from performing the above actions to use this blended pond material for the steel industry. At this time it is our intention to perform the necessary functions as addressed in this letter to transport pond materials to Hastie Mining.

If you have additional comments or questions, please call Hugh Roberts at 618-524-6349 or Marshall Shepherd at 618-524-6238.

Sincerely,

J. William Lessig
Plant Manager

JWL/sm

Attachment: Federal Register – January 22, 1992

cc: M. L. Shepherd
H. C. Roberts

U.S. Nuclear Regulatory Commission
Attention: Chief Operations Branch
Div. Of Fuel Cycle Safety & Safeguards, NMSS
Washington, DC 20555

U.S. NRC, Region III
Attention: Fuel Cycle Branch Chief
801 Warrenville Road
Lisle, IL 60532-4351

U.S. NRC
Attention: Project Manager
Fuel Cycle Licensing Branch
Division of Fuel Cycle Safety & Safeguards, NMSS
Office of Nuclear Material Safety & Safeguards
Washington, DC 20555-001

Hastie Trucking & Mining Company
R.R. #1, Box 55
Cave-In-Rock, IL 62919

Signed at Washington, DC this January 10, 1992.

Marvin M. Fooks,

Director, Office of Trade Adjustment Assistance.

[FR Doc. 92-1494 Filed 1-21-92; 8:45 am]

BILLING CODE 4510-30-M

TA-W-25,933, Dekalb Energy Co., Denver, CO; TA-W-25,933A Operations at Various Other Locations in Colorado; TA-W-25,933B Operations at all Locations in Texas; Amended Certification Regarding Eligibility to Apply for Worker Adjustment Assistance

In accordance with section 223 of the Trade Act of 1974 (19 U.S.C. 2273) the Department of Labor issued a Certification of Eligibility to Apply for Worker Adjustment Assistance on August 14, 1991 applicable to all workers of Dekalb Energy Company in Denver, Colorado and in various other locations in the state of Colorado.

The Department is amending the certification to include all locations in the State of Texas.

New information received by the Department shows that the Dekalb workers in Texas are under the control of the Denver, Colorado headquarters and that worker separations occurred at the Dekalb Energy Company in Texas because of reduced exploration and drilling activity.

The amended notice applicable to TA-W-25,933 is hereby issued as follows:

"All workers of Dekalb Energy Company, Denver, Colorado (TA-W-25,933) and in various other locations throughout the State of Colorado (TA-W-25,933A) and at all locations in Texas (TA-W-25,933B) who became totally or partially separated from employment on or after January 1, 1991 are eligible to apply for adjustment assistance."

Signed at Washington, DC this January 10, 1992.

Marvin M. Fooks,

Director, Office of Trade Adjustment Assistance.

[FR Doc. 92-1493 Filed 1-21-92; 8:45 am]

BILLING CODE 4510-30-M

[TA-W-26,516]

H.H. Cutler Co., Grand Rapids, MI; Termination of Investigation

Pursuant to section 221 of the Trade Act of 1974, an investigation was initiated on November 4, 1991 in response to a worker petition which was filed on behalf of workers at H.H. Cutler Company, Grand Rapids, Michigan.

The petitioner has requested that the petition be withdrawn. Consequently, further investigation in this case would serve no purpose; and the investigation has been terminated.

Signed at Washington, DC this 13th day of January 1992.

Marvin M. Fooks,

Director, Office of Trade Adjustment Assistance.

[FR Doc. 92-1495 Filed 1-21-92; 8:45 am]

BILLING CODE 4510-30-M

NATIONAL SCIENCE FOUNDATION

Federal Network Council Advisory Committee; Notice of Amendment

A Federal Advisory Networking Council Advisory Committee (FNCAC) meeting is being held on January 29, 1992, 9 a.m. to 4 p.m.

The location of the FNCAC meeting has been changed from the National Science Foundation to the Ramada Inn, 1430 Rhode Island Avenue, NW., Washington, DC 20005 (in the Cambridge Room, 2nd floor).

For additional information, contact Lynn Behnke, Executive Assistant, Federal Networking Council, 4001 N. Fairfax Drive, suite 200, Arlington, VA 22203-1614. Telephone: (703) 522-6410.

The notice for this meeting originally appeared in the January 9, 1992 issue of the Federal Register, Vol. 57, No. 6, p. 935.

January 15, 1992.

M. Rebecca Winkler,

Committee Management Officer.

[FR Doc. 92-1464 Filed 1-21-92; 8:45 am]

BILLING CODE 7555-01-M

NUCLEAR REGULATORY COMMISSION

[Docket No. 40-3392]

Finding of no significant impact and notice of opportunity for a hearing amendment of source materials; license no. SUB-526; Allied-Signal, Inc.; Metropolis, Illinois

The U.S. Nuclear Regulatory Commission (the Commission) is considering the amendment of Source Materials License No. SUB-526 for the Allied-Signal, Inc., facility located in Metropolis, Illinois, to authorize the release of calcium fluoride for use in the steel-making industry.

Summary of the Environmental Assessment

Identification of the Proposed Action: The proposed action is to ship the synthetic CaF_2 to briquette plants where it will be blended with natural CaF_2 (fluorspar) to form briquettes used as a fluxing agent in the steel-making industry. The synthetic CaF_2 is a byproduct of Allied's uranium hexafluoride (UF_6) conversion operation and contains trace amounts of natural uranium, thorium-230, radium-226, and arsenic.

The Need For The Proposed Action: Currently, Allied is authorized to transport synthetic CaF_2 to an Allied hydrofluoric acid (HF) production plant where it is blended with natural CaF_2 for routine HF production. Allied produces more CaF_2 than can be used in HF production, and the proposed action would authorize the recycle of the excess synthetic CaF_2 instead of Allied having to dispose of it. Also, the use of the synthetic CaF_2 will decrease the amount of natural CaF_2, a non-renewable natural resource, currently being used in the production of steel.

Environmental Impacts of the Proposed Action: The Allied UF_6 conversion process produces appropriately 4,000 to 6,000 tons of CaF_2 (dry weight basis) annually. When the CaF_2 is produced, it has a water content of approximately 20 percent.

For the most recent three-year period (1988-1990), the natural uranium content of the CaF_2 has averaged 131 pCi/gm (dry weight basis). This corresponds to a uranium content of 105 pCi/gm for CaF_2 with a 20 percent moisture content.

All CaF_2 produced at Allied is sampled and analyzed for uranium content. Currently at Allied, if the uranium concentration of the CaF_2 is less than 338.5 pCi/gm or 500 parts per million, the CaF_2 is added to the warehouse inventory. If the concentration is greater than 338.5 pCi/gm, then the CaF_2 is either disposed of as low-level radioactive waste or it is blended into the warehouse inventory, as long as doing so will not cause the uranium content of the inventory to exceed 338.5 pCi/gm.

A recent comparison by Allied of synthetic CaF_2 and natural fluorspar (CaF_2) follows:

Element or isotope	Synthetic CaF_2	Natural CaF_2
Natural Uranium	105 pCi/gm	18 pCi/gm.
Radium-226	0.26 pCi/gm	1.4 pCi/gm.
Thorium-230	1.9 pCi/gm	2.1 pCi/gm.
Arsenic	483 parts per million.	292 parts per million.

The license concluded that the only impurity that may be of public or worker

impact is the natural uranium. The levels of radium-226 and thorium-230 detected in the synthetic CaF_2 are comparable to levels found in the natural CaF_2. The airborne levels of arsenic detected in Allied's synthetic CaF_2 warehouse are less than 1 percent of the Occupational Safety and Health Administration (OSHA) permissible exposure limit of 0.010 mg/m³.

The level of natural uranium is approximately six times greater in the synthetic CaF_2 than in the natural CaF_2. However, the information supplied by the licensee shows that the synthetic CaF_2's natural uranium concentration is comparable to levels of natural uranium found in materials to which the general population is routinely exposed, such as Florida phosphate rocks (120 pCi/gm), Tennessee bituminous shale (50–80 pCi/gm), and cattle feed supplements (up to 122 pCi/gm).

A radiation dose assessment has been performed by Allied to determine the critical group and exposed general population doses which might result from the recycle of the synthetic CaF_2. The following table summarizes the maximum dose expected for individuals in the critical group and for the exposed general population. Allied used the following assumptions to complete the dose assessment:

1. Natural uranium is the only radioactive material present in the synthetic CaF_2. The concentration of the natural uranium is 105 pCi/gm. The solubility fraction of the natural uranium is 6.5 percent Class D and 93.5 percent Class W, as determined from lung fluid solubility testing. The particle size is one micron.

2. The finished briquettes are composed of 25 percent synthetic CaF_2.

3. The concentration of total dust in the briquetting work is 15 mg/m³ of which 25 percent is synthetic CaF_2. The briquetting plant worker wears a one-half face respirator as required by OSHA.

4. Based on the briquettes containing 25 percent synthetic CaF_2 and a distance of 1 meter, the external exposure rate to the briquette plant operator is 0.055 μR/hour.

Critical group	No. of persons	Total [1] mrem/year
Truck driver—Allied to loading dock.	2	5.2E-2
Clamshell operator at Metropolis dock.	1	1.3E-1
Clamshell operator at Ohio dock.	1	6.6E-2
Clamshell operator at Indiana dock.	1	6.6E-2
Truck driver—Ohio dock to briquette plant.	2	2.2E-1
Truck driver—Indiana dock to briquette plant.	2	1.9E-1
Briquette plant operator	1	7.6E-1
Truck driver—Briquette plant to steel mill.	2	8.3E-2
Total Critical Group Collective Dose.		1.6

[1] Total dose=Deep Dose Equivalent plus the Committed Effective Dose Equivalent times the number of persons exposed.

Exposed general population	No. of persons	Total mrem/year
CaF_2 truck route—Metropolis.	70	8.69E-5
CaF_2 truck route—Ohio	700	4.51E-4
CaF_2 truck route—Indiana	140	9.02E-5
Briquette truck routes—Ohio and Indiana.	420	5.15E-4
Total General Population Collective Dose.		1.14E-3

The dose assessment also evaluated two credible accidents: The overturning of a dump truck load of synthetic CaF_2 enroute to the briquette plant could produce a total effective dose commitment of 0.0004 mrem to an emergency response worker; and the overturning of a dump truck of briquettes enroute to a steel mill could produce a total effective dose commitment of 0.00001 mrem to an emergency response worker.

While the staff agrees with Allied's conclusion that the briquette plant operator is the maximally exposed individual, an independent dose assessment was performed. Most of Allied's assumptions were used in this assessment, however, no credit was given for the respiratory protection worn by the briquette plant operator, and the Th-230 and Ra-226 concentrations were included.

Based on the staff's independent assessment, the briquette plant operator will receive a committed effective dose equivalent of 12.35 mrem per year and a deep-dose equivalent of 0.11 mrem per year. The total effective dose equivalent is 12.46 mrem per year. However, if the concentration of natural uranium in the synthetic CaF_2 is 338.5 pCi/gram, then the briquette plant operator's total effective does equivalent will be 40.17 mrem per year, which exceeds the 25 mrem per year limit found in 40 CFR 190. Therefore, to ensure the dose received shall be within all federal limits, the staff recommends that the concentration of uranium in the synthetic CaF_2 released to each briquette manufacturer not exceed the average of 212 pCi/gram for any consecutive 12-month period.

In addition, the staff calculated the dose to the briquette plant operator if only the natural CaF_2 was used to manufacture the briquettes. Many of the same assumptions Allied used to calculate the dose from synthetic CaF_2 were used except that the natural uranium, Ra-226, and Th-230 concentrations in natural CaF_2, listed above, were used; no protection factor was allowed, and all CaF_2 used in the briquette was natural CaF_2. From this dose assessment, it was determined that the briquette plant operator would receive a committed effective dose equivalent of 45.48 mrem per year. The committed dose effective equivalent that the briquette plant operator would receive using 25 percent synthetic CaF_2 and 25 percent natural CaF_2 would drop to 35.20 mrem per year, of which 12.46 mrem would be from the synthetic CaF_2 and 22.79 mrem would be from the natural CaF_2. Therefore, by manufacturing the briquettes with 25 percent synthetic CaF_2 and 25 percent natural CaF_2, the dose to the briquette operator would be lower than if only natural CaF_2 was used. Based on lung solubility tests performed by the licensee, the uranium in the synthetic CaF_2 is more soluble in the body than the uranium in the natural CaF_2. Therefore, the biological clearance rate of the uranium in the synthetic CaF_2 is more rapid, thereby resulting in a lower committed effective dose equivalent.

After the briquettes are charged into the slag on the top of the steel melt, the amount of uranium contained in the briquettes is not transferred to the steel but remains in the slag.[1, 2] Members of the general public working with the finished steel products will receive no radiation exposure as a result of the synthetic CaF_2 being used in the briquettes.

The licensee calculated that the briquettes will comprise not more than 5 percent of the total slag weight. The final uranium content in the slag will be 1.6 pCi/gram. Based on this uranium content, Allied estimated that the dose to a member of the public from any foreseeable use of the steel mill slag will not exceed an effective dose equivalent of 0.02 mrem/year.

Over 90 percent of the slag generated by the steel mills is stored at the mills in U.S. Environmental Protection Agency (EPA)-approved storage areas with both dusting and leaching conditions

[1] Mautz E. W., et.al., "Uranium Decontamination of Common Metals by Smelting—A Review," Document No. NLCO–1113, February 1975.

monitored. Occasionally, steel mill slag is used as a minor constituent in cement manufacturing or as road fill for highway construction where drainage, subsoil, and paving circumstances permit. The level of uranium contained in the slag is so small that any environmental effects from storage, road fill, or cement manufacturing would be insignificant.

Conclusion: The staff's dose assessment performed for the proposed action demonstrates that the doses received by members of the critical group and the exposed general population was well below the dose limits of 100 mrem/year and 25 mrem/year, as specified in 10 CFR part 20 (56 FR 23360–474) and 40 CFR part 190, respectively. To ensure the dose limits are not exceeded, the staff recommends that the uranium concentration of the synthetic CaF2 shall not exceed 212 pCi/gram averaged over any consecutive 12-month period. The environmental impact from using the synthetic CaF_2 is insignificant. The uranium content in the slag will be less than the 10 pCi/gram limit for unrestricted release of natural uranium set in Option 1 of the staff's Branch Technical Position, "Disposal and On-site Storage of Thorium or Uranium Wastes from Past Operations" (46 FR 52061–63). With Allied limiting the concentration of natural uranium in the synthetic CaF_2 to be sold to the steel-making industry to less than 338.5 pCi/gram, then the limit set in 10 CFR 40.13(a) for unimportant quantities of source material will also not be exceeded. Therefore, the staff concludes that there will be no significant environmental impacts associated with the proposed action.

Alternatives to the Proposed Action: The alternative to the proposed action would be the denial of the proposed action. By denying approval of the amendment, Allied would not be able to sell the synthetic CaF_2 to the steel-making industry. While this would eliminate any possible negative impact to human health and safety due to the trace amounts of natural uranium in the CaF_2, there would be an increased burden placed on the environment because the synthetic CaF_2 would have to be disposed of, probably in a landfill. Furthermore, the amount of natural CaF_2 currently being extracted from natural sources would not be reduced.

Agencies and Persons Consulted: Staff utilized the amendment application dated July 1, 1991, and supplementary information dated October 28, 1991.

Finding of No Significant Impact: The Commission has prepared an Environmental Assessment related to the amendment of Source Materials License No. SUB–526. On the basis of this assessment, the Commission has concluded that environmental impacts that would be created by the proposed licensing action would not be significant and do not warrant the preparation of an Environmental Impact Statement. Accordingly, it has been determined that a Finding of No Significant Impact is appropriate.

The Environmental Assessment and the above documents related to this proposed action are available for public inspection and copying at the Commission's Public Document Room at the Gelman Building, 2120 L Street NW., Washington, DC.

Opportunity for a Hearing

Any person whose interest may be affected by the issuance of this amendment may file a request for a hearing. Any request for hearing must be filed with the Office of the Secretary, U.S. Nuclear Regulatory Commission, Washington, DC 20555, within 30 days of the publication of this notice in the Federal Register be served on the NRC staff (Executive Director for Operations, One White Flint North, 11555 Rockville Pike, Rockville, MD 20852); on the licensee (Allied-Signal, Inc., P.O. Box 430, Metropolis, Illinois 62960); and must comply with the requirements for requesting a hearing set forth in the Commission's regulation, 10 CFR part 2, subpart L, "Informal Hearing Procedures for Adjudications in Materials Licensing Proceedings."

These requirements, which the requestor must describe in detail, are:

1. The interest of the requestor in the proceeding;

2. How that interest may be affected by the results of the proceeding, including the reasons why the requestor should be permitted a hearing;

3. The requestor's areas of concern about the licensing activity that is the subject matter of the proceeding; and

4. The circumstances establishing that the request for hearing is timely, that is, filed within 30 days of the date of this notice.

In addressing how the requestor's interest may be affected by the proceeding, the request should describe the nature of the requestor's right under the Atomic Energy Act of 1954, as amended, to be made a party to the proceeding; the nature and extent of the requestor's property, financial, or other (i.e., health, safety) interest in the proceeding; and the possible effect of any order that may be entered in the proceeding upon the requestor's interest.

Dated at Rockville, Maryland, this 14th day of January, 1992.

For the Nuclear Regulatory Commission.

John W.N. Hickey,

Chief, Fuel Cycle Safety Branch, Division of Industrial and Medical Nuclear Safety, NMSS.

[FR Doc. 92–1503 Filed 1–21–92; 8:45 am]

BILLING CODE 7590–01–M

[Docket No. 50–348]

Southern Nuclear Operating Co., Inc.; Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF–2 issued to Southern Nuclear Operating Company, Inc. (the licensee), for operation of the Joseph M. Farley Nuclear Plant (Farley), Unit 1, located in Houston County, Alabama. The amendment request was submitted by Alabama Power Company, however, subsequent to the submittal, Amendment No. 90 to Facility Operating License No. NPF–2 authorized Southern Nuclear Operating Company, Inc., to become the licensed operator. The change to Southern Nuclear Operating Company, Inc., as the operator of Farley was implemented on December 23, 1991.

The proposed amendment would reduce the steam generator primary-to-secondary leakage limit for Farley, Unit 1. The current technical specification allows one gallon per minute (1440 gallons/day) total primary-to-secondary leakage through all steam generators and 500 gallons per day through any one steam generator. This amendment request proposes to reduce the leakage limit to 420 gallons per day total primary-to-secondary leakage through all steam generators and 140 gallons per day through any one steam generator.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from