August 2, 2001

Mr. Daniel B. Kelley, Chairman B&W Owners Group Core Performance Committee FirstEnergy Nuclear Operating Company Davis-Besse Nuclear Power Station 5501 North State Rt. 2 Oak Harbor, OH 43449

SUBJECT: SAFETY EVALUATION FOR TOPICAL REPORT BAW-10179P, REVISION 4,

"SAFETY CRITERIA AND METHODOLOGY FOR ACCEPTABLE CYCLE

RELOAD ANALYSES," MARCH 2001 (TAC MB1692)

Dear Mr. Kelley:

By letter dated March 5, 2001, the Babcock and Wilcox Owners Group (B&WOG) submitted Topical Report BAW-10179P, Revision 4, "Safety Criteria and Methodology for Acceptable Cycle Reload Analyses," for NRC review and approval. Once approved, this revision can be referenced as the current approved version in plants' technical specifications. Revision 4 to BAW-10179P was submitted as non-proprietary. The staff has completed its review and concludes that Revision 4 to BAW-10179P is acceptable. The staff's safety evaluation is enclosed.

We do not intend to repeat our review of the matters described in the report, and found acceptable, when the report appears as a reference in license applications, except to ensure that the material presented is applicable to the specific plant involved. Our acceptance applies only to the matters described in the report.

In accordance with procedures established in NUREG-0390, we request that you publish an accepted version of this report within 3 months of receipt of this letter. The accepted versions shall incorporate this letter and the enclosed evaluation between the title page and the abstract. The accepted versions shall include an "-A" (designating accepted) following the report identification symbol.

Should our criteria or regulations change so that our conclusions about acceptability of the report are invalidated, the B&WOG and the licensees referencing the topical report will be expected to revise and resubmit their respective documentation, or to submit justification for the continued effective applicability of the topical reports without revision of their respective documentation.

If you have any questions about the staff's review, please contact Stewart Bailey at (301) 415-1321, or by email at snb@nrc.gov.

Sincerely, /RA by Stephen Dembek for/

Stuart A. Richards, Director Project Directorate IV Division of Licensing Project Management Office of Nuclear Reactor Regulation

Project No. 693

Enclosure: Safety Evaluation

cc w/encl: See next page

If you have any questions about the staff's review, please contact Stewart Bailey at (301) 415-1321, or by email at snb@nrc.gov.

Sincerely,

/RA by Stephen Dembek for/

Stuart A. Richards, Director Project Directorate IV Division of Licensing Project Management Office of Nuclear Reactor Regulation

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Enclosure: Safety Evaluation

cc w/encl: See next page

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CC:

Mr. David. J. Firth B&W Owners Group Services Framatome ANP 3315 Old Forest Road Lynchburg, VA 24506-0935

Mr. F. McPhatter, Manager Framatome ANP 3315 Old Forest Road Lynchburg, VA 24506-0935

Mr. James F. Mallay Director, Regulatory Affairs Framatome ANP 2101 Horn Rapids Road Richmond, WA 99352

Mr. J. Holm Framatome ANP 2101 Horn Rapids Road Richland, WA 99352

Mr. M. Schoppman Framatome ANP 1911 N. Ft Myer Drive Rosslyn, VA 22209

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

TOPICAL REPORT BAW-10179P, REVISION 4

"SAFETY CRITERIA AND METHODOLOGY FOR ACCEPTABLE

CYCLE RELOAD ANALYSES"

PROJECT NO. 693

1.0 <u>INTRODUCTION</u>

By letter dated March 5, 2001, the Babcock and Wilcox Owners Group (B&WOG) submitted Revision 4 to Topical Report BAW-10179P, "Safety Criteria and Methodology for Acceptable Cycle Reload Analyses." BAW-10179P provides the criteria and methodology for determining cycle specific limits and setpoints that are included in the Core Operating Limits Reports (COLRs) for the Babcock and Wilcox (B&W) 177-fuel assembly class of nuclear power plants. The report serves as the approved methodology that is referenced in the COLR portion of the Administrative Controls Section of the plant technical specifications.

2.0 **EVALUATION**

In this revision to BAW-10179P, the B&WOG added five topical reports that have been approved by the Nuclear Regulatory Commission since Revision 3 of BAW-10179P was approved. Each topical report is briefly described in an appendix, and the description includes the conditions and limitations on the applicability of the topical reports.

The following have been incorporated into BAW-10179P, Revision 4:

Appendix P: "Addendum 1 and Addendum 2 to Mark-C Fuel Assembly LOCA-Seismic

Analyses - BAW-10133P-A, Rev. 1, Addendum 1 and Addendum 2."

Appendix Q: "The BWU-Z Critical Heat Flux Correlations - BAW-10199P-A,

Addendum 1."

Appendix R: "Evaluation of Advanced Cladding and Structural Material (M5™) in PWR

Reactor Fuel - BAW-10227P-A."

Appendix S: "SCIENCE - BAW-10228P-A."

Appendix T: "Mark-B11 Fuel Assembly Design Report - BAW-10229P-A."

The staff reviewed the safety evaluations that were issued for the above topical reports, the B&WOG's descriptions of the topical reports, and the limitations, if any, on applicability to B&W 177-fuel assembly plants. The staff finds that the summaries for the topical reports, including limitations on applicability to the B&W 177-fuel assembly plants, are adequate.

3.0 CONCLUSION

The staff concludes that BAW-10179P, Revision 4, only adds topical reports that are approved for B&W 177-fuel assembly plants, and that the summaries of the topical reports are adequate. Therefore, Revision 4 may be referenced in plant technical specifications as the current approved version of BAW-10179P.

Principal Contributor: Stewart N. Bailey

Date: August 2, 2001