



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

March 31, 1981

Dockets Nos. 50-269, ~~50-270~~
and 50-287

Mr. William O. Parker, Jr.
Vice President - Steam Production
Duke Power Company
P. O. Box 33189, 422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Parker:

The Commission has issued the enclosed Amendments Nos. 95, 95, and 92 for Licenses Nos. DPR-38, DPR-47 and DPR-55 for the Oconee Nuclear Station, Units Nos. 1, 2 and 3. These amendments consist of changes to the Station's common Technical Specifications (TSs) and are in response to your exemption request to 10 CFR Part 20 dated May 19, 1980 and letter of December 17, 1980. After discussions with your staff, we agreed that an exemption to 10 CFR Part 20 is not needed and an addition to the Technical Specifications is more appropriate than the requested exemption.

These amendments revise the TSs by providing for the incineration of waste oil contaminated with very low levels of radioactivity under limits more stringent than 10 CFR 20 and represent the Commission's approval of the disposal of this licensed material by incineration in accordance with the provisions of 10 CFR Part 20.305.

Copies of the Safety Evaluation and Environmental Impact Appraisal and the Notice of Issuance/Negative Declaration are also enclosed.

Sincerely,

A handwritten signature in cursive script that reads "John F. Stolz".

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Amendment No. 95 to DPR-38
2. Amendment No. 95 to DPR-47
3. Amendment No. 92 to DPR-55
4. Safety Evaluation and Environmental Impact Appraisal
5. Notice of Issuance/Negative Declaration

cc w/enclosures:
See next page

810409612

Duke Power Company

cc w/enclosure(s):

Mr. William L. Porter
Duke Power Company
P. O. Box 2178
422 South Church Street
Charlotte, North Carolina 28242

Oconee Public Library
201 South Spring Street
Walhalla, South Carolina 29691

Honorable James M. Phinney
County Supervisor of Oconee County
Walhalla, South Carolina 29621

Director, Criteria and Standards
Division
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U. S. Environmental Protection Agency
Washington, D. C. 20460

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Atlanta, Georgia 30308

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DeBevoise & Liberman
1200 17th Street, N.W.
Washington, D. C. 20036

cc w/enclosure(s) & incoming dtd.:
5/19/80 & 12/17/80

Office of Intergovernmental Relations
116 West Jones Street
Raleigh, North Carolina 27603



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT NO.1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 95
License No. DPR-38

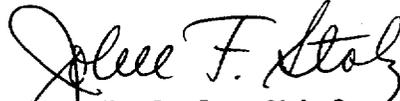
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The filing by Duke Power Company (the licensee) dated May 19, 1980, as supplemented December 17, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the filing, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-38 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 95 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 31, 1981



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-270

OCONEE NUCLEAR STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 95
License No. DPR-47

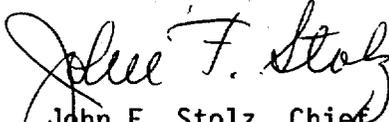
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The filing by Duke Power Company (the licensee) dated May 19, 1980, as supplemented December 17, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the filing, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-47 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 95 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 31, 1981



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-287

OCONEE NUCLEAR STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

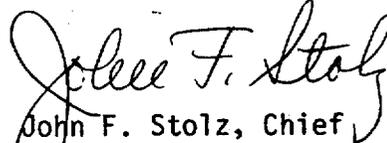
Amendment No. 92
License No. DPR-55

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The filing by Duke Power Company (the licensee) dated May 19, 1980, as supplemented December 17, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the filing, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-55 is hereby amended to read as follows:
 - 3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 92 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, reading "John F. Stolz".

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 31, 1981

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 95 TO DPR-38

AMENDMENT NO. 95 TO DPR-47

AMENDMENT NO. 92 TO DPR-55

DOCKETS NOS. 50-269, 50-270 AND 50-287

Revise Appendix A as follows:

Remove Pages

3.10-3

3.10-4

Insert Pages

3.10-3

3.10-4

- 3.10.8 The reactor building shall not be purged unless the following conditions are met:
- a. Reactor building purge shall be through the high efficiency particulate filters and charcoal filters until the activity concentration is below the occupational limit inside the reactor building, at which time bypass may be initiated.
 - b. If reactor building is purged, the purge shall be through the high efficiency particulate filters whenever irradiated fuel is being handled or any objects are being handled over irradiated fuel in the reactor building.
- 3.10.9 Used oil, contaminated by radioactivity, may be incinerated in the Station auxiliary boiler provided it meets the following limits:
- a. Oil shall not be disposed of by incineration if any 55-gallons contain radioactivity in excess of the quantities given in 10 CFR Part 20, Appendix C;
 - b. The rate of incineration shall be limited such that the concentration in the stack does not exceed 0.5 times the quantity given in 10 CFR Part 20, Appendix B, Table 2, Col. 1;
 - c. Contaminated oil shall be burned no more than six hours in any calendar quarter.
- 3.10.10 In addition to the above continuous sampling and monitoring requirements, gaseous radioactive waste sampling and activity analysis shall be performed in accordance with Table 4.1-3. Records shall be maintained and reports of the sampling and analysis results shall be submitted in accordance with Section 6.6 of these specifications.

Bases

It is expected that the releases of radioactive materials and gaseous wastes will be kept within the design objective levels and will not exceed on an instantaneous basis the dose rate limits specified in 10CFR20.

These levels provide reasonable assurance that the resulting annual exposure from noble gases to the whole body or any organ of an individual will not exceed 10 mRem per year. At the same time, the licensee is permitted the flexibility of operation compatible with considerations of health and safety to assure that the public is provided a dependable source of power under unusual operating conditions which may temporarily result in releases higher than the design objective levels but still within the concentration limits specified in 10CFR20. It is expected that using this operational flexibility under unusual operating conditions, the licensee shall exert every effort to keep levels of radioactive materials and gaseous wastes as low as practicable and that annual releases will not exceed a small fraction of the annual average concentration limits specified in 10CFR20. These efforts shall include consideration of meteorological conditions during releases.

The anticipated annual releases from the three Oconee reactor units have been developed taking into account a combination of system variables including fuel failure, primary system leakage, and the performance of radio-isotope removal mechanisms. The values assumed for these variables include the following:

- a. Reactor coolant fission product concentration corresponding to 0.25 percent fuel cladding defects;
- b. Steam generator primary-to-secondary leakage rate of 20 gpd;
- c. Reactor coolant leakage to the containment building of 120 gpd and 12 containment vents per year;
- d. Primary coolant stripped 12 times per year;
- e. Decay time of the waste gas processing system - 30 days;
- f. Decontamination factor of 1,000 for iodine in the evaporator;
- g. Charcoal filter decontamination factor of 10 for iodine removal in the purge exhaust system.

The application of the above estimates results in the radio-gas discharge rates shown in Table III-13 of the "Final Environmental Statement Related to Operation of Oconee Nuclear Station Units 1, 2, and 3".

The noble gas release rates stated in the objectives are based on a X/Q value from the annual meteorological data. The dispersion factor used, 3.6×10^{-6} sec/m³, is conservative and the release rate is controlled to a small fraction of 10CFR20 requirements at the exclusion area boundary (.02 of 10CFR20 = less than 10 mRem per year).

The I-131 and particulate release rates stated in the objectives limits the concentration at the exclusion area boundary to much less than 1 percent of the MPC listed in 10CFR20. The release rate also controls the expected iodine dose due to the milk pathway (using concentration factor in the milk pathway of 700) at the nearest cow and the nearest dairy (taken as five miles west, $X/Q = 1.22 \times 10^{-7}$ sec/m³) to less than five millirem per year. This meets the intent of proposed Appendix I to 10CFR50. A survey will be conducted once per year to assure that no milk producing cows are within a five mile radius of the plant.

The maximum one-hour release rate limits the dose rate at the exclusion area boundary to less than 2 mRem per hour even during periods of unfavorable meteorology (using conservative meteorological conditions, i.e., two hour X/Q of 1.16×10^{-4} sec/m³ accident meteorology).

The maximum activity in a gaseous waste tank is specified as 17,200 curies/ \bar{E} based on a postulated tank rupture that allows all of the contents to escape to the atmosphere. This specification limits the maximum off-site dose to well below the limits of 10CFR100.

The lowest practicable gaseous release objectives expressed in this specification are based on the guidelines contained in the proposed Appendix I of 10CFR50. Since these guidelines have not yet been adopted, the release objectives of this specification will be reviewed at the time Appendix I becomes a regulation to assure that this specification is based upon the guidelines contained therein.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION AND ENVIRONMENTAL IMPACT APPRAISAL BY THE OFFICE OF NUCLEAR
REACTOR REGULATION

SUPPORTING AMENDMENT NO. 95 TO FACILITY OPERATING LICENSE NO. DPR-38

AMENDMENT NO. 95 TO FACILITY OPERATING LICENSE NO. DPR-47

AMENDMENT NO. 92 TO FACILITY OPERATING LICENSE NO. DPR-55

OCONEE NUCLEAR STATION, UNITS NOS. 1, 2, AND 3

DUKE POWER COMPANY

DOCKETS NOS. 50-269, 50-270 AND 50-287

Introduction

By letters dated May 19, 1980 and December 17, 1980, Duke Power Company (DPC or the licensee) submitted a request to dispose of radioactively contaminated waste oil by incineration in the auxiliary boiler at the Oconee Nuclear Station. Oconee currently has the oil in storage in 55-gallon drums. It is contaminated with isotopes of cesium and cobalt. The sources of this oil are the turbine building sumps and the reactor coolant pump (RCP) motor oil. The RCP motor oil is changed every two years and generates approximately 1,000 gallons of waste oil for each Oconee unit. The licensee wants to dispose of the oil because the stored oil is an onsite radiological and fire hazard and there is a lack of adequate storage space.

The low level waste disposal sites will not accept unsolidified contaminated oil. DPC provided the specific radionuclides present in each drum of oil, and the respective concentrations. The predominant isotopes present are Cs-134, Cs-137, Co-58 and Co-60.

The licensee's radiological assessment addressed airborne release pathways for normal conditions. After evaluating alternatives, the licensee's conclusion was that incineration provided the best means of disposal. The licensee did not provide an assessment of accident conditions where a fire may burn a portion or all of the oil. Also, consequences of liquid spills were not addressed in the request.

We have conducted an independent review and evaluation of the potential radiological hazards associated with incineration of the slightly contaminated oil in the auxiliary boiler.

I. Safety Evaluation

Radioactive Release Considerations

Airborne releases will take place during normal operations of the auxiliary boiler. The pathway for these releases will be the stack. The contaminated oil that is fed to the boiler will be diluted with fuel oil. Table I shows the data supplied by the licensee that was used in performing their radiological assessment. We found these data to be acceptable and they were used to perform our assessment.

Our assessment has shown that if all the radioactivity that is contained in the oldest 24 drums of waste oil, which has the lowest concentration of radioactivity of the stored oil, is released to the atmosphere through burning, the quantity of radioactivity released would be less than 0.003% of the annual Technical Specification (TS) airborne release limits at Ocone. Also, the amount released would be only 0.11% of the total amount of particulate activity actually released during normal operation for 1979. The analysis also showed that the average concentration of radioactivity leaving the stack every second would be 18% of the 10 CFR 20, Appendix B, Table II limits and the highest concentration of radioactivity leaving would be 53% of the limits. We are adding a TS which will limit any future incinerations of waste oil to radioactivity levels equivalent to the oldest 24 drums described in the May 19 letter, and more stringent than 10 CFR 20 limits for gaseous effluent releases.

Dose Control Onsite/Offsite

Potential onsite radiation dose problems are minimized by the small quantities and concentration of activity present. The dose rates at the surface of the drums from the contained byproduct materials will be less than 0.06 mrem/hr. Consequently, the contaminated oil will present no dose problems as long as it is properly contained.

A spill of the contaminated oil could produce both surface and airborne contamination. However, the surface contamination would not exceed 400 dpm per 100 cm², which is 40% of the limit for release for uncontrolled use as found in Regulatory Guide 1.86. The low concentration of radioactivity in conjunction with the normal radiation control practices should preclude any significant doses from surface contamination.

Hypothetically, the offsite maximum permissible concentration (MPC) could be exceeded by accidentally burning the undiluted contaminated oil. With perfect combustion, the offsite MPC of 10 CFR Part 20, Appendix B, Table II, could be exceeded briefly by a factor of five, but in an accidental fire, only a small fraction of the oxygen in the air would react so the activity concentration would not be expected to exceed the offsite MPC value. Exposure time will be limited; exposure to the airborne contamination from the burning of the most contaminated drum of oil will be expected to produce a whole body dose commitment of less than 0.0006 mrem during normal operations. The licensee proposes incineration of the waste oil diluted with the boiler fuel oil.

The potential for offsite radiation exposure develops when the oil is burned in the auxiliary boiler. The oil is to be burned at a rate not exceeding 5.0 gal/min

while the stack flow rate is 100,000 cfm, see Table I. The radioactivity concentration in the stack will not exceed 250 pCi/m³, which is approximately 53% of the offsite MPC. A drum of oil would burn in 11 minutes. Exposure to the undiluted stack exhaust for the entire period would amount to 3.1 MPC-minutes. The average level of activity in the drums is about one third the maximum. Consequently exposure to the undiluted stack exhaust during the burning of the entire oldest 24 drums of oil would constitute an exposure of 25 MPC-minutes; the calculated corresponding whole body dose commitments are:

Adult:	0.01 mrem
Teen:	0.007 mrem
Child:	0.003 mrem
Infant:	0.001 mrem

Our assessment showed that a reasonable criteria for burning would allow the concentration to reach 0.5 times MPC and permit burning for six hours per quarter. By doing this, the exposure to undiluted stack exhaust during the entire quarterly burn period would amount to a maximum of 120 MPC-minutes and the corresponding doses would be seven times the doses given above (0.07 mrem to an adult, etc.). Of course, such direct exposure to stack exhaust is not possible and actual offsite doses would be lower by a factor of ten or more.

We conclude that the incineration of waste oil, subject to the TS restrictions discussed below, in this manner will not result in unacceptable offsite doses, and burning this oil is consistent with the NRC principles of, "as low as reasonably achievable" of 10 CFR 50.36a. The Commission, therefore, grants approval of the disposal of slightly contaminated oil by incineration in accordance with the provisions of 10 CFR 20.305, subject to the TS restrictions discussed below.

Technical Specification Change

The proposed TS provides for conservative rates of radiation release by:

1. limiting radioactivity in any 55-gallon drum to 10 CFR 20 - Appendix C levels,
2. limiting rate of incineration such that stack concentration of radioactivity is one-half of 10 CFR 20 - Appendix B, and
3. limiting period of burning to six hours in any calendar quarter.

We conclude that this TS limits the effects of incineration of slightly contaminated oil to acceptable levels.

Conclusion on Safety

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

II. ENVIRONMENTAL IMPACT APPRAISAL

Radiological Impact

As stated above in the Safety Evaluation, the Duke May 19, 1980 request, with the limits established in the TSS, will provide reasonable assurance that releases of radioactive material will be sufficiently low during normal operations to meet 10 CFR 20, Appendix B limits.

Radiation Dose Impact

The total radioactivity contained in the oldest 24 55-gallon drums of slightly contaminated oil is 31.85 μCi , see Table I. The highest radioactivity in any single drum is 4.12 μCi . Occupational exposure in handling the drums during the incineration process has been assessed in the above Safety Evaluation along with an assessment of doses. The impact is expected to be less than 1% of station exposure limits. The TS change provided by these amendments will limit the activity in future drums of contaminated oil, prior to incineration, to levels equivalent to the oldest 24 drums.

As stated in the above Safety Evaluation, the highest offsite whole body dose commitment from burning all 24 barrels of this slightly contaminated oil would be 0.01 mrem for an adult. Since our calculations of the dose commitment contain the conservative assumptions that, 1) 24 barrels of oil are burned at one time (the proposed TS limits burning to six hours in any calendar quarter), and 2) the stack exhaust would be undiluted at the time of such burning (the proposed TS limits the rate of incineration to 0.5 times 10 CFR Part 20), the calculated value of 0.01 mrem is considered to be quite conservative. In the Final Environmental Statement (FES) related to the operation of the Oconee Nuclear Station, the staff calculated the estimated annual radiation dose to the population, in a two mile radial distance from the facility, to be 0.25 mrem/yr from gaseous effluents released by the three reactor units at the Station, (FES, Table V-4, page 124). Therefore, we find the offsite dose consequences of burning the slightly contaminated oil to be considerably less than the yearly, estimated radiation exposure to the population from routine releases. The numerical guides for meeting the "as low as reasonably achievable" (ALARA) criterion contained in Appendix I to 10 CFR Part 50, specify 15 mrem as the annual objective for radioactive material in particulate form to be released from

nuclear reactors. The annual dose commitment as a result of burning this oil plus the dose contribution from gaseous effluents released by the three reactor units at the station, will therefore, be still well within the guidelines set forth in 10 CFR 50, Appendix I. Since the additional restrictions required by the TS provide an additional degree of assurance that the radiological consequences of burning this slightly contaminated oil will be minimal, we find, in accordance with 10 CFR Part 20.305, that the burning of this slightly contaminated oil is an acceptable means of disposal.

Conclusion and Basis for Negative Declaration

On the basis of the foregoing, we conclude that there would be no significant environmental impact attributable to the proposed burning of such slightly contaminated oil. As a result of this conclusion, the Commission has further concluded that no environmental impact statement for the proposed action need be prepared, and that a negative declaration to this effect is appropriate.

Dated: March 31, 1981

TABLE I

DATA USED FOR RADIOLOGICAL ASSESSMENT

Maximum Amount of Radionuclides Present
In the 24 Drums of Oil

Co-58	1.08 μCi
Co-60	1.39 μCi
Cs-134	6.52 μCi
Cs-137	<u>22.86 μCi</u>
Total	31.85 μCi

Total Oil Feed Rate to Boiler = 18.8 gal/min.

Percentage of Contaminated Oil = 5gpm/18.8 gpm = 27%

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKETS NOS. 50-269, 50-270 AND 50-287DUKE POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES
AND NEGATIVE DECLARATION

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 95, 95 and 92 to Facility Operating Licenses Nos. DPR-38, DPR-47 and DPR-55, respectively, issued to Duke Power Company, which revised the Technical Specifications for operation of the Oconee Nuclear Station, Units Nos. 1, 2 and 3, located in Oconee County, South Carolina. The amendments are effective as of the date of issuance.

These amendments revise the Station's Common Technical Specifications by providing for the incineration of waste oil contaminated with very low levels of radioactivity under limits more stringent than 10 CFR Part 20 and represent approval of disposal by incineration as required by 10 CFR 20.305.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has prepared an environmental impact appraisal for this action and has concluded that an environmental impact statement is not warranted because there will be no significant environmental impact attributable to this action.

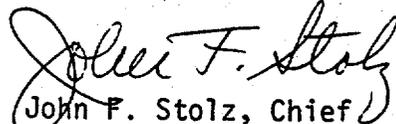
For further details with respect to this action, see (1) the licensee's request dated May 19, 1980, as supplemented December 17, 1980, (2) Amendments

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Nos. 95, 95, and 92 to Licenses Nos. DPR-38, DPR-47 and DPR-55, respectively, and (3) the Commission's related Safety Evaluation and Environmental Impact Appraisal. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Oconee County Library, 201 South Spring Street, Walhalla, South Carolina. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 31st day of March 1981.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing