



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

October 26, 1982

DO NOT REMOVE

Dockets Nos. 50-269, 50-270
and 50-287

*Posted
Amndt. 115
to DPR-47*

Mr. Hal B. Tucker
Vice President - Nuclear
Production Department
Duke Power Company
P. O. Box 33189
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

The Commission has issued the enclosed Amendments Nos. 115, 115, and 112 to Licenses Nos. DPR-38, DPR-47 and DPR-55 for the Oconee Nuclear Station, Units Nos. 1, 2 and 3. These amendments consist of changes to the Station's common Technical Specifications (TSs) in response to your requests dated July 30, 1982 and September 10, 1982.

These amendments revise the TSs to reflect a Duke Power Company reorganization and personnel qualifications in the Administrative Controls Section and also correct the local leak testing requirements for certain containment penetrations.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Philip C. Wagner

Philip C. Wagner, Project Manager
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Amendment No. 115 to DPR-38
2. Amendment No. 115 to DPR-47
3. Amendment No. 112 to DPR-55
4. Safety Evaluation
5. Notice

cc w/enclosures: See next page

Duke Power Company

cc w/enclosure(s):

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 115
License No. DPR-38

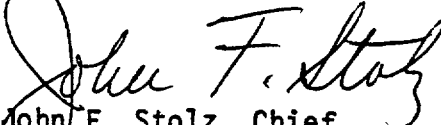
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendments by Duke Power Company (the licensee) dated July 30, 1982 and September 10, 1982, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-38 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 115 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 26, 1982



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-270

OCONEE NUCLEAR STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 115
License No. DPR- 47

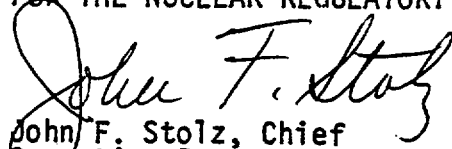
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendments by Duke Power Company (the licensee) dated July 30, 1982 and September 10, 1982, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-47 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 115 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 26, 1982



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-287

OCONEE NUCLEAR STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 112
License No. DPR-55

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendments by Duke Power Company (the licensee) dated July 30, 1982 and September 10, 1982, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense security or to the health and safety of the public; and

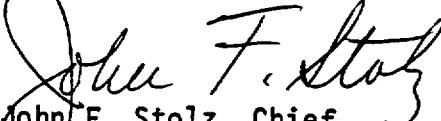
issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-55 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 112 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 26, 1982

ATTACHMENTS TO LICENSE AMENDMENTS

AMENDMENT NO. 115 TO DPR-38

AMENDMENT NO. 115 TO DPR-47

AMENDMENT NO. 112 TO DPR-55

DOCKETS NOS. 50-269, 50-270 AND 50-287

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment numbers and contain vertical lines indicating the area of change.

Remove Pages

4.4-9

4.4-10

4.4-11

6.1-1

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6.1-2

6.1-3

6.1-4

6.1-5

6.1-7

6.1-8

6.2-1

6.3-1

Insert Pages

4.4-9

4.4-10

4.4-11

6.1-1

6.1-1a

6.1-2

6.1-3

6.1-4

6.1-5

6.1-7

6.1-8

6.2-1

6.3-1

TABLE 4
LIST OF PENETRATIONS WITH 10CFR50,
APPENDIX F REQUIREMENTS

PENETRATION NUMBER	SYSTEM	TYPE A SYSTEM CONFI.	LOCAL LEAK TEST	REMARKS
36 37	RB emergency sump recirculation line	Not Vented	None required	Note 5
38	Quench tank cooler inlet line	Note 1	Type C	Note 2, 7d
39	HP Nitrogen supply	Note 1	None required	Note 3 (manual valves)
(Unit 2, 3) Only	CFT Vent line	Note 1	None required	Note 3 (manual valves)
40	RB emergency sump drain line	Note 1	None required	
41	Instrument air supply & ILRT verification line	Note 1	None required	Note 3 (manual valves)
42	SPARE	Not in Use		
43	OTSG A drain line	Note 1	None required	Note 7b
44	Component cooling to control rod drive inlet line	Note 1	Type C	Note 3, 7d
45	ILRT instrument line	Not Vented	Type C	Note 3, 7a
46	Reactor head-wash filtered water inlet	Note 1	Type C	Note 3 (manual valves)

TABLE 1
LIST OF PENETRATIONS WITH 10CFR50,
APPENDIX J TEST REQUIREMENTS

PENETRATION NUMBER	SYSTEM	TYPE A TEST SYSTEM CONDITION	LOCAL LEAK TEST	REMARKS
47 (Unit 1 only)	Demineralized water supply to RC pump seal vents	Note 1	Type C	Note 3, 7d
48	Breathing air inlet	Note 1	None required	Note 3 (manual valves)
49 (Unit 1 only)	LP Nitrogen supply	Note 1	None required	Note 3 (manual valves)
50	OTSG A Emergency FDW line	Not Vented	None required	Note 5
51	ILRT Pressurization line	Note 1	None required	Note 6a, 7a
52	HP Injection to 'B' loop	Not Vented	None required	Note 5
53 (All)	HP Nitrogen supply to 'A' core flood tank	Note 1	None required	Note 3 (manual valves)
(Unit 2, 3)	LP Nitrogen supply	Note 2	None required	Note 3 (manual valves)
54	Component cooling outlet line	Note 1	Type C	Note 3, 7b, 9(8)
55	Demineralized water supply	Note 1	Type C	(Unit 1) Note 3, (manual valves) (Unit 2,3) Note 3, 9 (manual valves)
56	Spent fuel canal fill and drain	Note 1	None required	Note 3 (manual valve)
57 (Unit 1 only)	DHR return line	Not Vented	None required	Note 4

TABLE
LIST OF PENETRATIONS WITH 10CFR50,
APPENDIX J TEST REQUIREMENTS

PENETRATION NUMBER	SYSTEM	TYPE A TEST SYSTEM CONDITION	LOCAL LEAK TEST	REMARKS
58 (All)	OTSG B sample line	Note 1	Type C	Note 7b
(Unit 2, 3)	Pressurizer sample line	Note 1	Type C	Note 2, 7b
59	CF tank sample line	Note 1	None required	Note 2
60	RB sample line (outlet)	Note 1	Type C	Note 2, 7b, 9
61	RB sample line (inlet)	Note 1	Type C	Note 3, 7b, 9
62 (Units 2, 3 only)	DHR return line	Not vented	None required	Note 4
	Personnel hatch	Vented	Type B	Note 6b
	Emergency hatch	Vented	Type B	Note 6b
	Equipment hatch	Vented	Type B	Note 6c
	Electrical penetration	Vented	Type B	Note 6a

6.0 ADMINISTRATIVE CONTROLS

6.1 ORGANIZATION, REVIEW, AND AUDIT

6.1.1 Organization

- 6.1.1.1 The Station Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.
- 6.1.1.2 In all matters pertaining to actual operation and maintenance and to these Technical Specifications, the station Manager shall report to and be directly responsible to the Vice President, Nuclear Production Department, through the General Manager, Nuclear Stations. The organization is shown in Figure 6.1-2.
- 6.1.1.3 The station organization for Operations, Technical Services and Maintenance shall be functionally as shown in Figure 6.1-1. Minimum operating shift requirements are specified in Table 6.1-1.
- 6.1.1.4 Incorporated in the staff of the station shall be personnel meeting the minimum requirements encompassing the training and experience described in Section 4 of ANSI/ANS-3.1-1978, "Selection and Training of Nuclear Power Plant Personnel" except for the Site Health Physicist, the Superintendent of Operations and the Operating Engineer.

The Site Health Physicist shall have a bachelor's degree in a science or engineering subject or the equivalent in experience, including some formal training in radiation protection, and shall have at least five years of professional experience in applied radiation protection of which three years shall be in applied radiation protection work in one of Duke Power Company's nuclear stations.

A qualified individual who does not meet the above requirements, but who has demonstrated the required radiation protection management capabilities and professional experience in applied radiation protection work at one of Duke Power Company's multi-unit nuclear stations, may be appointed to the position of Site Health Physicist by the station Manager, based on the recommendations of the System Health Physicist and as approved by the General Manager, Nuclear Stations.

The Superintendent of Operations shall have a minimum of eight years of responsible nuclear or fossil station experience, of which a minimum of three years shall be nuclear station experience. A maximum of two years of the remaining five years of experience may be fulfilled by academic training, or related technical training, on a one-for-one time basis. The Superintendent of Operations shall hold or have held a Senior Reactor Operator license.

The Operating Engineer shall have a minimum of eight years of responsible nuclear or fossil station experience, of which a minimum of three years shall be nuclear station experience. A maximum of two

years of the remaining five years of experience may be fulfilled by academic training, or related technical training on a one-for-one time basis. The Operating Engineer shall hold a Senior Reactor Operator license.

- 6.1.1.5 Retraining and replacement of station personnel shall be in accordance with Section 5.5 of the ANSI/ANS-3.1-1978, "Selection and Training of Nuclear Power Plant Personnel."
- 6.1.1.6 A training program for the fire brigade shall meet or exceed the requirements of Section 27 of the NFPA Code-1975, except that training sessions may be held quarterly.
- 6.1.1.7 The two functions of the Shift Technical Advisor, namely accident assessment and operating experience assessment, are fulfilled in the following manner:

- a. An experienced SRO, who has been instructed in additional academic subjects, will be assigned on-shift to provide the accident assessment capability.
- b. The operating experience assessment function will be provided by the Station Safety Review Group.

6.1.2 Technical Review and Control

6.1.2.1 Activities

- a. Procedures required by Technical Specification 6.4 and other procedures which affect station nuclear safety, and changes (other than editorial or typographical changes) thereto, shall be prepared by a qualified individual/organization. Each such procedure, or procedure change, shall be reviewed by an individual/group other than the individual/group which prepared the procedure, or procedure change, but who may be from the same organization as the individual/group which prepared the procedure, or procedure change. Such procedures and procedure changes may be approved for temporary use by two members of the station staff, at least one of whom holds a Senior Reactor Operator's License on the unit(s) affected. Procedures and procedure changes shall be approved prior to use or within seven days of receiving temporary approval for use by the station Manager; or by the Operating Superintendent, the Technical Services Superintendent or the Maintenance Superintendent, as previously designated by the station Manager.
- b. Proposed changes to the Technical Specifications shall be prepared by a qualified individual/organization. The preparation of each proposed Technical Specifications change shall be reviewed by an individual/group other than the individual/group which prepared the proposed change, but who may be from the same organization as the individual/group which prepared the proposed change. Proposed changes to the Technical Specifications shall be approved by the station Manager.
- c. Proposed modifications to station nuclear safety-related structures, systems and components shall be designated by a qualified individual/organization. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modification. Proposed modifications to station nuclear safety-related structures, systems and components shall be approved prior to implementation by the station Manager; or by the Operating Superintendent, the Technical Services Superintendent, or the Maintenance Superintendent, as previously designated by the station Manager.
- d. Individuals responsible for reviews performed in accordance with 6.1.2.1.a, 6.1.2.1.b, and 6.1.2.1.c shall be members of the station supervisory staff, previously designated by the station Manager to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated station review personnel.
- e. Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be reviewed by the station Manager; or by the Operating Superintendent, the Technical Services Superintendent or the Maintenance Superintendent, as previously designated by the station Manager.

- f. Incidents reported pursuant to Technical Specification 6.6.2.1 and violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. Such reports shall be approved by the station Manager and transmitted to the Vice President, Nuclear Production Department, or his designee, and the Director of the Nuclear Safety Review Board.
- g. The station Manager shall assure the performance of special reviews and investigations, and the preparation and submittal of reports thereon, as requested by the Vice President, Nuclear Production Department.
- h. The station security program, and implementing procedures, shall be reviewed at least annually. Changes determined to be necessary as a result of such review shall be approved by the station Manager and transmitted to the Vice President, Nuclear Production Department, or his designee, and the Director of the Nuclear Safety Review Board.
- i. The station emergency plan, and implementing procedures, shall be reviewed at least annually. Changes determined to be necessary as a result of such review shall be approved by the station Manager and transmitted to the Vice President, Nuclear Production Department, or his designee, and the Director of the Nuclear Safety Review Board.
- j. The station Manager shall assure that an independent fire protection and loss prevention inspection and audit shall be performed annually utilizing qualified off-site personnel and that an inspection and audit by a qualified fire consultant shall be performed at intervals no greater than three years.

6.1.2.2 Records

Records of the above activities shall be maintained.

6.1.3 Nuclear Safety Review Board

6.1.3.1 Function

The NSRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations
- b. Nuclear Engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Instrumentation and control
- f. Radiological safety
- g. Mechanical and electrical engineering
- h. Administrative control and quality assurance practices

6.1.3.2 Organization

- a. The Director, members, and alternate members of the NSRB shall be formally appointed by the Vice President, Nuclear Production Department, and shall have an academic degree in an engineering or physical science field; and in

addition, shall have a minimum of five years technical experience, of which a minimum of three years shall be in one or more areas given in 6.1.3.1.

- b. The NSRB shall be composed of at least five members, including the Director. Members of the NSRB may be from the Nuclear Production Department, from other departments within the Company or from external to the Company. A maximum of one member of the NSRB may be from the Oconee Nuclear Station staff.
- c. Consultants may be utilized by the NSRB to provide expert advice to the NSRB, as determined necessary by the Director of the NSRB.
- d. Staff assistance may be provided to the NSRB in order to promote the proper, timely and expeditious performance of its functions.
- e. The NSRB shall meet at least once per six months. The period between such meeting shall not exceed eight months.
- f. A quorum of the NSRB shall consist of the Director, or his designated alternate, and at least two other NSRB members or alternate members. No more than a minority of the quorum shall have line responsibility for operation of Oconee Nuclear Station.

6.1.3.3 Subjects Requiring Review

The following subjects shall be reported to and reviewed by the NSRB:

- a. The safety evaluations for (1) changes to procedures, equipment or systems, and (2) tests or experiments completed under the provisions of 10 CFR 50.59(a)(1) to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in 10 CFR 50.59.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59.
- d. Proposed changes in Technical Specifications or the Facility Operating Licenses.
- e. Violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of station equipment that affect nuclear safety.
- g. Incidents that are the subject of non-routine reports submitted to the Commission.
- h. Quality Assurance Department audits relating to station operations and actions taken in response to these audits.

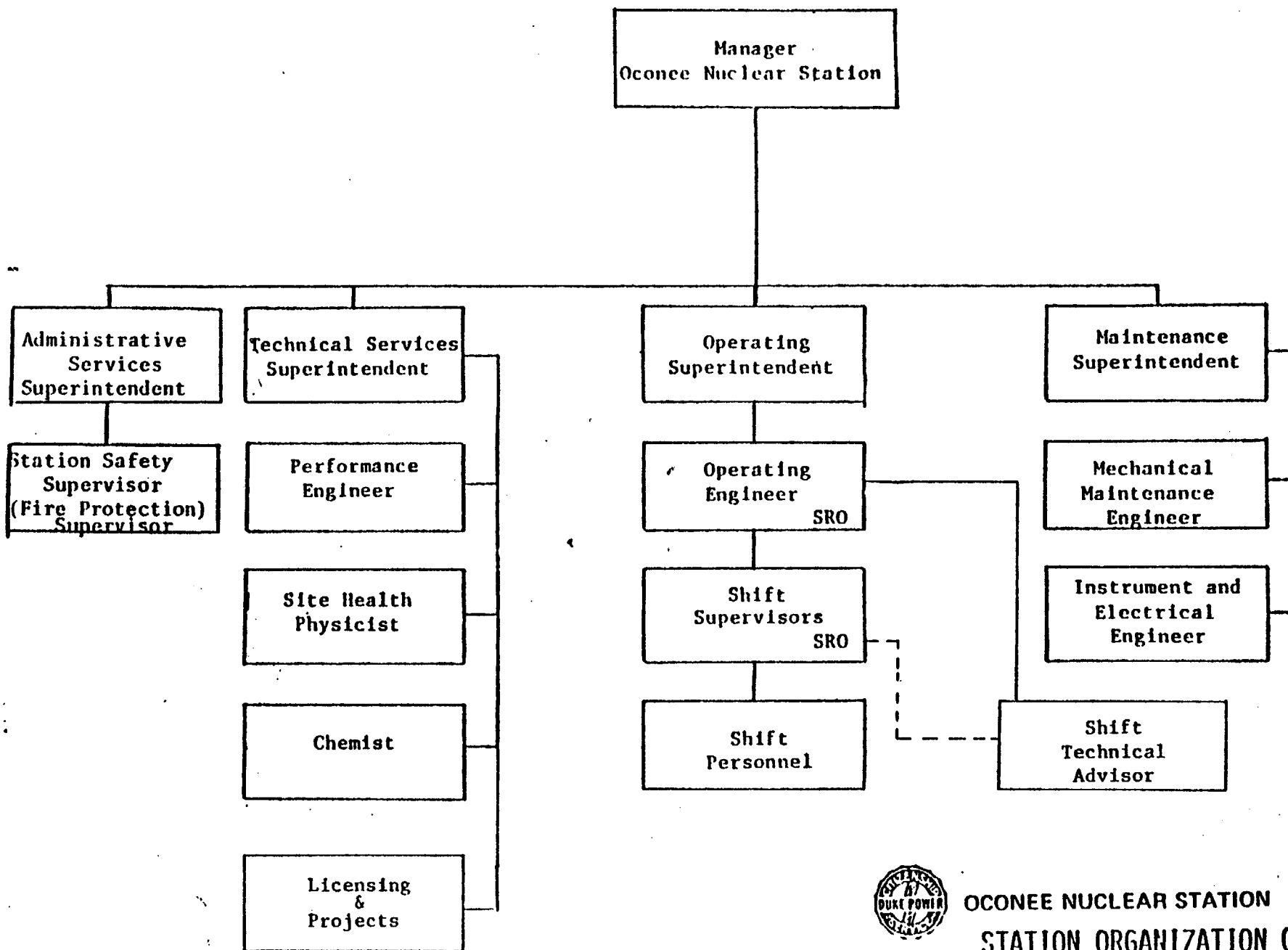
6.1.3.4 Audits

Audits of station activities shall be performed under the cognizance of the NSRB. These audits shall encompass:

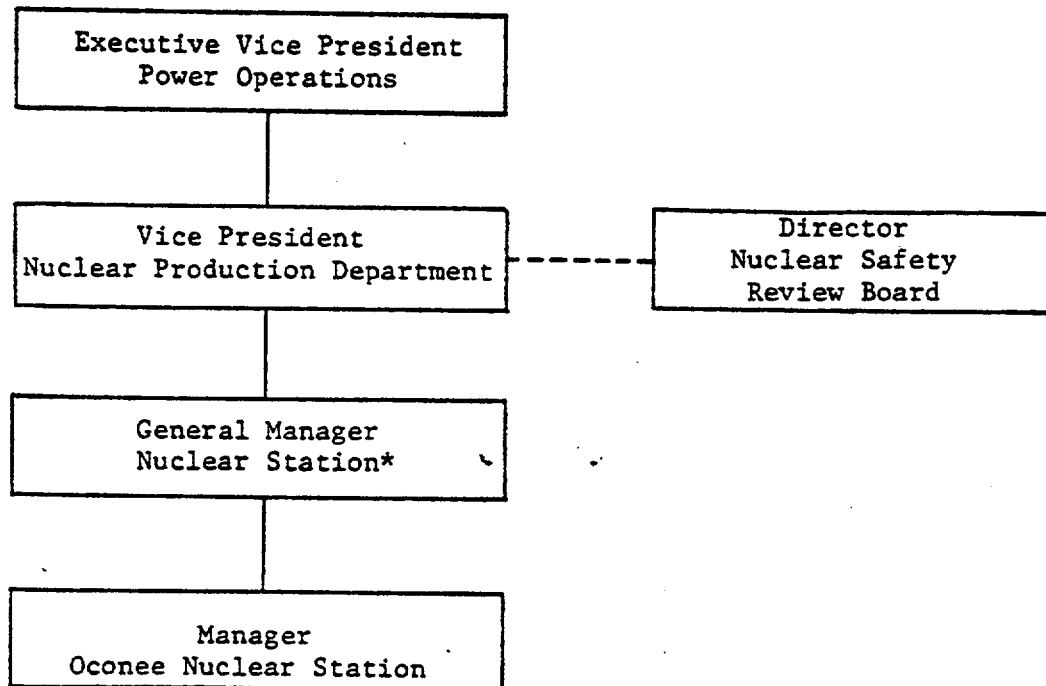
- a. The conformance of station operation to provisions contained within the Technical Specifications and applicable facility operating license conditions at least once per year.
- b. The performance, training and qualifications of the station staff at least once per year.
- c. The results of actions taken to correct deficiencies occurring in equipment, structures, systems or methods of operation that affect nuclear safety at least once per six months.
- d. The performance of activities required by the quality assurance program to meet the criteria of Appendix B to 10 CFR 50 at least once per two years.
- e. The station emergency plan and implementing procedures at least once per two years.
- f. The station security plan and implementing procedures at least once per two years.
- g. Any other area of station operation considered appropriate by the NSRB or the Vice President, Nuclear Production Department.
- h. The station fire protection program and implementing procedures at least once per 24 months.

6.1.3.5 Responsibilities and Authorities

- a. The NSRB shall report to and advise the Vice President, Nuclear Production Department on those areas of responsibility specified in Specifications 6.1.3.3 and 6.1.3.4.
- b. Minutes shall be prepared and forwarded to the Vice President, Nuclear Production Department, and to the Executive Vice President, Power Operations, within 14 days following each formal meeting of the NSRB.
- c. Records of activities performed in accordance with Specifications 6.1.3.3 and 6.1.3.4 shall be maintained.
- d. Audit reports encompassed by Section 6.1.3.4 shall be forwarded to the Vice President, Nuclear Production Department and to the Executive Vice President, Power Operations, and to the management position responsible for the areas audited within 30 days of completion of each audit.



OCONEE NUCLEAR STATION
STATION ORGANIZATION CHART
FIGURE 6.1-1



*Responsible for Fire Protection Program



OCONEE NUCLEAR STATION

MANAGEMENT ORGANIZATION CHART

Figure 6.1-2

6.2 ACTION TO BE TAKEN IN THE EVENT OF A REPORTABLE OCCURRENCE

6.2.1 Any reportable occurrence shall be investigated promptly by the station Manager.

6.2.2 The Vice President, Nuclear Production Department shall be notified of any reportable occurrence. A written report shall be prepared which describes the circumstances leading up to and resulting from the incident and shall recommend appropriate action to prevent or minimize the probability of a recurrence. The report shall be submitted to the Vice President, Nuclear Production Department, and the Director of the Nuclear Safety Review Board.

6.2.3 The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.6.2.

6.3 ACTION TO BE TAKEN IN THE EVENT A SAFETY LIMIT IS EXCEEDED

- 6.3.1 If a safety limit is exceeded, the reactor shall be shut down immediately and maintained in a safe shutdown condition until the Commission authorizes resumption of operation.
- 6.3.2 The violation of a safety limit shall be reported to the Commission, the Vice President, Nuclear Production Department, and the Director of the Nuclear Safety Review Board.
- 6.3.3 - A report shall be prepared which describes (1) applicable circumstances preceding the violation, (2) effects of the violation upon structures, systems or components, and (3) corrective action taken to prevent recurrence. The report shall be reviewed by the Operating Superintendent and the station Manager. The report shall be submitted to the Vice President, Nuclear Production Department, and the Director of the Nuclear Safety Review Board.
- 6.3.4 A report of the violation, with appropriate analyses and corrective action to prevent recurrence shall be submitted to the Commission within 10 days of the violation.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 115 TO FACILITY OPERATING LICENSE NO. DPR-38

AMENDMENT NO. 115 TO FACILITY OPERATING LICENSE NO. DPR-47

AMENDMENT NO. 112 TO FACILITY OPERATING LICENSE NO. DPR-55

DUKE POWER COMPANY

OCONEE NUCLEAR STATION, UNITS NOS. 1, 2 AND 3

DOCKETS NOS. 50-269, 50-270 AND 50-287

1.0 Introduction

By letter dated July 30, 1982, Duke Power Company (Duke or the licensee) requested changes to the common Oconee Nuclear Station (ONS) Technical Specifications (TSs) to reflect a Duke reorganization and revised qualification requirements for the Superintendent of Operations and the Operating Engineer.

An additional application was submitted by letter dated September 10, 1982, which requested changes to the local leak testing requirements for four containment penetrations.

2.0 Evaluation

A. Administrative Controls Changes

• Establishment of the Nuclear Production Department

The current Oconee TSs indicate that the Manager, Oconee Nuclear Station, reports to the Manager, Nuclear Production, who in turn reports to the Vice President, Steam Production. The proposed reorganization divides the functions of the Vice President, Steam Production, into three new positions: Vice President, Nuclear Production; Vice President, Fossil Production; and Vice President, Production Support. Reporting to the Vice President, Nuclear Production, will be a General Manager, Nuclear Stations, to whom the Nuclear Station Managers will report. Also reporting to the Vice President, Nuclear Production, will be a Manager, Nuclear Reliability Assurance; Manager, Nuclear Safety Assurance; Manager, Nuclear Operation; Manager, Nuclear Maintenance; Manager, Nuclear Technical Services; Manager, Nuclear Engineering Services; Manager, Nuclear Administrative Services; and Manager, Nuclear General Services.

We consider that these proposed changes provide for better integration of those functions needed to support the operation of the nuclear station under a single high level corporate official and are acceptable. This is reflected in revised TS Figure 6.1-2 and numerous text changes.

- Establishment of an In-Line Operating Engineer

The current TSs show the Shift Supervisors reporting directly to the Operating Superintendent, who meets the qualification requirements for Operations Manager of Section 4.2.2 of ANSI/ANS 3.1-1978, "American National Standards for Selection and Training of Nuclear Power Plant Personnel", with an Operating Engineer as a staff assistant to the Operating Superintendent. The licensee plans to upgrade the qualifications of the Shift Operating Engineer to that of the Operations Manager of Section 4.2.2 of ANSI/ANS 3.1-1978 and place the Shift Operating Engineer in a line position between the Shift Supervisors and the Operating Superintendent. This organization is shown in revised TS Figure 6.1-1. The licensee proposes to alter the qualification requirements for the Operating Superintendent such that he must hold or have held a senior license on the unit.

We find that the proposed organizational change is acceptable as the Shift Operating Engineer will meet the qualification requirements for the individual performing the function of Operations Manager of Section 4.2.2 of ANSI/ANS 3.1-1978 and meets the staff position of Revision 1 to Regulatory Guide 1.8.

- Operating Experience Assessment

The current TSs state that "several engineers, familiar with plant operations and representing diverse technical backgrounds will be assigned to provide the operating experience assessment." The licensee proposes to change this requirement to state that "the operating experience assessment function will be provided by the Station Safety Review Group."

We find this proposed change acceptable as it does not dilute the operating experience assessment function.

- B. Local Leak Test Requirements

License Amendments Nos. 104, 104 and 101 were issued to ONS Units 1, 2 and 3, respectively, on November 6, 1981. These amendments revised the containment leak rate testing requirements in accordance with Appendix J to 10 CFR 50. Subsequently, it was determined that the local leak test requirements for four penetrations (Numbers 46, 55, 60 and 61) are more restrictive than required by Appendix J. Therefore, by letter dated September 10, 1982, Duke requested the local leak test requirements be corrected from Type B to Type C for these four penetrations.

Appendix J to 10 CFR 50 requires: Type B tests to detect local leaks and to measure leakage across each pressure-containing or leakage limiting boundary for penetrations incorporating designs such as resilient seals, bellows and flexible metal seals, air lock doors and doors with resilient seals; and Type C tests to measure containment isolation valve leakage rates. The four penetrations have diaphragm-type, containment isolation valves on both the inside and outside of the reactor building. A misinterpretation of the Appendix J designation of resilient seals to include diaphragm-type valves

ed to the proposal indicating that Type B tests would be performed on these penetrations. Since the results of either Type B or Type C leak testing will provide acceptable indications of containment integrity, we approved the proposal as requested in the above mentioned license amendments.

Since the change from a Type B to Type C test for Penetrations 46, 55, 60 and 61 will continue to provide acceptable indications of containment integrity and since these containment isolation valves should appropriately (in accordance with Appendix J) be Type C leak tested, we find this change to be acceptable.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of an accident previously evaluated, do not create the possibility of an accident of a type different from any evaluated previously, and do not involve a significant reduction in a margin of safety, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 26, 1982

The following NRC personnel have contributed to this Safety Evaluation:
Philip C. Wagner, Fred Allenspach.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKETS NOS. 50-269, 50-270 AND 50-287DUKE POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 115, 115 and 112 to Facility Operating Licenses Nos. DPR-38, DPR-47 and DPR-55, respectively, issued to Duke Power Company, which revised the Technical Specifications (TSs) for operation of the Oconee Nuclear Station, Units Nos. 1, 2 and 3, located in Oconee County, South Carolina. The amendments are effective as of the date of issuance.

These amendments revise the TSs to reflect a Duke Power Company reorganization and personnel qualifications in the Administrative Controls Section and also correct the local leak testing requirements for certain containment penetrations.

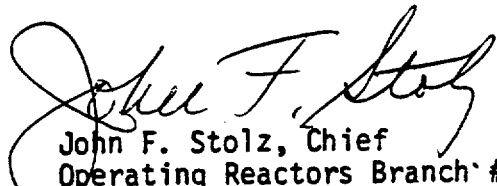
The applications for the amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

For further details with respect to this action, see (1) the applications for amendments dated July 30, 1982, and September 10, 1982, (2) Amendments Nos. 115, 115, and 112 to Licenses Nos. DPR-38, DPR-47 and DPR-55, respectively, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Oconee County Library, 501 West Southbroad Street, Walhalla, South Carolina. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 26th day of October 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing