

JAFNPP

3.5 (cont'd)

4.5 (cont'd)

|    | <u>Item</u>  | <u>Frequency</u>   |
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|    | e. a verification that each valve (manual, power operated, or automatic) in the flowpath that is not locked, sealed or otherwise secured in position, is in the correct position.  | Once per 31 Days   |
|    | f. an air test shall be performed on the containment spray headers and nozzles.  | Once per 5 Years   |
| 2. | Should one RHRSW pump of the components required in 3.5.B.1 above be made or found inoperable, continued reactor operation is permissible only during the succeeding 30 days provided that during such 30 days all remaining components of the containment cooling mode subsystems are operable. | 2. When it is determined that one RHRSW pump of the components required in 3.5.B.1 above is inoperable, the remaining components of the containment cooling mode subsystems shall be verified to be operable immediately and daily thereafter.   |
| 3. | Should one of the containment cooling subsystems become inoperable or should one RHRSW pump in each subsystem become inoperable, continued reactor operation is permissible for a period not to exceed 7 days.*  | 3. When one containment cooling subsystem becomes inoperable, the redundant containment cooling subsystem shall be verified to be operable immediately and daily thereafter. When one RHRSW pump in each subsystem becomes inoperable, the remaining components of the containment cooling subsystems shall be verified to be operable immediately and daily thereafter. |
| 4. | If the requirements of 3.5.B.2 or 3.5.B.3 cannot be met, the reactor shall be placed in a cold condition within 24 hr.   | <i>*During the installation of modification 00-125 to the "B" RHRSW strainer, continued reactor operation is permissible for a period not to exceed 11 days.</i>   |
| 5. | Low power physics testing and reactor operator training shall be permitted with reactor coolant temperature < 212°F with an inoperable component(s) as specified in 3.5.B above.   |  |

Amendment No. ~~3, 95, 148, 151, 153, 171, 203, 241, 259, 271~~

## 3.5 BASES (cont'd)

B. Containment Cooling Mode (of the RHR System)

The containment heat removal portion of the LPCI/containment spray mode is provided to remove heat energy from the containment in the event of a loss-of-coolant accident. For the flow specified, the containment long-term pressure is limited to less than 8 psig and, therefore, is more than ample to provide the required heat removal capability.

Each subsystem of the containment cooling mode (of the RHR System) consists of two RHR Pumps, two RHR service water pumps, one heat exchanger and a flowpath capable of recirculating water from the suppression pool through the heat exchanger and back to primary containment. Either subsystem is capable of performing the containment cooling function. Loss of one RHR service water pump does not seriously jeopardize the containment cooling capability as any two of the remaining three pumps can satisfy the cooling requirements. Since there is some redundancy left, a thirty-day repair period is adequate. Loss of one subsystem of the containment cooling mode leaves one remaining system to perform the containment cooling function. The operable system is verified to be operable each day when the above condition occurs. Based on the fact that when one containment cooling subsystem becomes inoperable only

one system remains, a seven day repair period was specified.\*

Low power physics testing and reactor operator training with inoperable components will be conducted only when the containment cooling mode of RHR is not required for the safety of the plant.

Calculations have been made to determine the effects of the design basis LOCA while conducting low power physics testing or operator training at or below 212°F. The results of these conservative calculations show that the suppression pool water temperature will not exceed 170°F. Therefore LPCI and Core Spray Systems will not be adversely affected by the postulated LOCA.

*\*During the installation of modification 00-125 to the "B" RHRSW strainer, the seven day repair period may be extended to eleven days. The Conditional Core Damage Probability with the plant in this configuration for eleven days has been determined to be below the threshold probability of 1 E-6 for risk significance of temporary changes to the plant configuration in the EPRI PSA Applications Guide.*