

Docket

NOV 8 1973

Docket Nos. 50-269
and 50-270

Change No. 6 License No. DPR-38
Change No. 1 License No. DPR-47

Duke Power Company
ATTN: Mr. Austin C. Thies
Senior Vice President,
Production & Transmission
422 South Church Street
P. O. Box 2178
Charlotte, N. C. 28201

Gentlemen:

We are transmitting herewith Technical Specification changes to the operating licenses for Oconee Nuclear Station, Units 1 and 2, which remove the 95% of power operating restriction initially imposed on Unit 1 and extended to Unit 2 pending confirmation of satisfactory performance of Unit 1 at significant power.

Oconee Unit 1 achieved criticality April 19, 1973, and is presently in commercial operation at power levels of up to 95% of full rated power. Based on information which you have made available to the Regulatory staff from the date of criticality to the present, a period of over six months, this power plant has performed as expected in all significant respects during start-up, testing and all modes of power operation, and that the increase to full license power reflected by this change, does not, involve significant hazards consideration.

Based on discussions with you on this date, November 8, 1973, you have committed to fully document this plant performance in an extensive report on or before November 19, 1973.

In view of the above, we have concluded that Oconee Units 1 and 2, can be operated, as requested, at 100% of full rated power (2668 Mwt) with reasonable assurance that the health and safety of the public will not be endangered. Therefore, pursuant to Section 50.50 of 10 CFR Part 50, Technical Specifications, Appendix A to License

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Nos. DPR-38 and DPR-47, have been changed in accordance with the revised Technical Specification 3.11 Maximum Power Restriction enclosed. These changes are identified as Change No. 6 to the Technical Specifications License DPR-38 and Change No. 1 to the Technical Specifications License DPR-47, respectively and are effective immediately.

Sincerely,

Original Signed



R. C. DeYoung, Assistant Director
for Pressurized Water Reactors
Directorate of Licensing

Enclosure:
Technical Specification Changes

cc: William L. Porter
Duke Power Company
P. O. Box 2178
422 South Church St.
Charlotte, N. C. 28261

bcc: J. R. Buchanan, ORNL
T. B. Abernathy, DTIC

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OFFICE	PWR-4	OCB	RO	LEAD/WR	I.C./PWR-4	LEAD/PWRs
	<i>[initials]</i>	<i>[initials]</i>	<i>[initials]</i>	<i>[initials]</i>	<i>[initials]</i>	<i>[initials]</i>
	IAPeltier	[initials]	JKlepper	JHendrie	ASchwencer	RCDeYoung

3.11 MAXIMUM POWER RESTRICTION

Applicability

Applies to the nuclear steam supply system of Units 1 and 2 reactors.

Objective

To maintain core life margin in reserve until the system has performed under operating conditions and design objectives for a significant period of time.

Specification

- 3.11.1 The first reactor core in Unit 1 may not be operated beyond 7500 effective full power hours until supporting analyses and data pertinent to fuel clad collapse under fuel densification conditions have been approved by Directorate of Licensing staff.
- 3.11.2 The first reactor core in Unit 2 may not be operated beyond 11,040 effective full power hours until supporting analysis and data pertinent to fuel clad collapse under fuel densification conditions have been approved by the Directorate of Licensing.

Bases

The licensing staff has reviewed the effects of fuel densification for the first core in Oconee Units 1 and 2 and concluded that clad collapse will not take place within the first fuel cycle (7500 effective full power hours for Unit 1 and 11040 effective full power hours for Unit 2). However, the clad collapse model used is questionable for extrapolation of clad collapse time out beyond the first fuel cycle because of limited experimental verification.