

August 2, 2001

MEMORANDUM TO: William D. Beckner, Chief  
Technical Specification Branch  
Division of Regulatory Improvement Programs  
Office of Nuclear Reactor Regulation

FROM: Guy S. Vissing, Senior Project Manager, Section 1 **/RA/**  
Project Directorate I  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

SUBJECT: CLOSURE OF FITZPATRICK BEYOND SCOPE ISSUE (BSI), BSI-F2,  
ITS SURVELLANCE REQUIREMENTS (SR) 3.6.1.7, CONSISTING OF  
CHANGING THE SR FOR THE CONTAINMENT VACUUM BREAKERS  
TO BE CONSISTENT WITH THE INSERVICE TESTING PROGRAM,  
(TAC NO. MB2382)

By letter dated May 31, 2001, Entergy Nuclear Operations, the licensee, submitted a change to FitzPatrick SR 3.6.1.7 consisting of changing the SR for the containment vacuum breakers to be consistent with the inservice testing program which changes the testing frequency from 31 days to 92 days. This would be a BSI since it changes the requirements of NUREG-1433, Revision I, "Standard Technical Specifications, General Electric Plants BWR/4" dated April 1995. The attachment provides the necessary safety evaluation to close out the subject BSI.

Docket No. 50-333

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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
IMPROVED TECHNICAL SPECIFICATION CONVERSION LICENSE AMENDMENT  
BEYOND SCOPE ISSUE  
ENTERGY NUCLEAR OPERATIONS  
JAMES A. FITZPATRICK NUCLEAR PLANT  
DOCKET NO. 50-333

1.0 INTRODUCTION AND BACKGROUND

By letter dated May 20, 1999, and subsequent letters the Power Authority of the State of New York (PASNY), the then licensee, requested an amendment to convert the current Technical Specifications (CTS) to the improved Standard Technical Specifications (STS) format in the Improved Technical Specifications (ITS) for the James A. FitzPatrick Nuclear Power Plant (FitzPatrick). On November 21, 2000, ownership of FitzPatrick was transferred to Entergy Nuclear FitzPatrick, LLC, to possess and use and to Entergy Nuclear Operations, Inc, to possess, use and operate FitzPatrick. By letter dated January 26, 2001, Entergy Nuclear Operations requested that the NRC continue to review and act on all requests before the Commission which had been submitted by PASNY before the transfer. Accordingly, the staff continued its review of PASNY's request and response concerning the issue of the subject amendment. By letter dated May 31, 2001, Entergy Nuclear Operations proposed a new change to FitzPatrick SR 3.6.1.7 consisting of changing the SR for the containment vacuum breakers to be consistent with the Inservice Testing (IST) Program which changes the testing frequency from 31 days to 92 days. This proposed change would be a beyond scope issue since it changes the requirements of NUREG-1433, Revision I, "Standard Technical Specifications, General Electric Plants BWR/4" dated April 1995.

2.0 EVALUATION

BSI-F2 SR 3.6.1.7.1, SR 3.6.1.7.2 , B 3.6.1.7 Suppression Chamber-to-Drywell Vacuum Breakers

The proposed change would revise the frequency of performing a functional test of each required vacuum breaker from 31 days to a new schedule in accordance with the IST Program which is 92 days.

The licensee stated that the IST Program lists all valves required to be tested in accordance with American Society of Mechanical Engineers Section XI. In addition, the proposed ITS 5.5.7 requires the IST Program to be established, implemented and maintained. These controls are adequate to ensure the required tests are performed

at the appropriate frequencies and changes to the requirement to the IST Program will be controlled by the provisions of 10 CFR 50.59.

The Frequency of CTS 4.7.A.5.a, which requires exercising each Suppression Chamber-to-Drywell vacuum breaker through an open-close cycle, is proposed to be extended from "monthly" to a frequency that is "In Accordance with the Inservice Testing (IST) Program" in the proposed IST SR 3.6.1.7.2. The licensee stated that at FitzPatrick, the vacuum breakers are not located in the harsh environment of the suppression chamber. The valves are located in the reactor building (secondary containment) where the environment is similar to that which exists for many primary and secondary containment isolation valves which are subjected to tests on a Frequency that is in accordance with the IST Program (92 days). In addition, similar SRs for the Reactor Building-to-Suppression Chamber vacuum breakers, which are of a similar design, have similar design functions, are also located in the reactor building, are performed on a Frequency that is in accordance with IST Program as stated in CTS 4.7.A.4.a (ITS SR 3.6.1.7.2).

The licensee provided in their DOC (Discussion of Change) L1 for the proposed ITS 3.6 a historical review of Suppression Chamber-to-Drywell vacuum breaker surveillance data that has been performed for the past 5 years and the data indicate there were no failures of the vacuum breakers to properly operate through a full open-close cycle operation. Therefore, based on (1) the valve reliability performance and (2) the longer test interval that has been approved for the similar Reactor Building-Suppression Chamber vacuum breakers and other valves located in areas with a similar environment (not a harsh environment), the staff finds the proposed extension of the SR in the proposed ITS SR 3.6.1.7.2 from the current 31 days to a frequency that is "In accordance with Inservice Testing Program" (92 days) Frequency to be acceptable.

Principal Contributor: G. Vissing

Date: August 2, 2001