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Docket Nos. 50-269
50-270
and 50-287

Duke Power Company
ATTN: Mr. William O. Parker, Jr.
Vice President
422 South Church Street
Post Office Box 2178
Charlotte, North Carolina 28201

Gentlemen:

The Commission has issued the enclosed Amendments No. 18, 18, and 15 for Licenses No. DPR-38, DPR-47, and DPR-55, for the Oconee Nuclear Station, Units 1, 2, and 3. These amendments are in response to your request dated February 28, 1975, as supplemented June 19 and December 5, 1975.

These amendments revise portions of the Organization, Review and Audit section of the Technical Specifications and consist of (1) the assignment of required technical review and control functions to qualified individuals or groups rather than to an onsite review committee, (2) the assignment of several review functions to the onsite operating organization rather than to an onsite review committee, and (3) a restructured review and audit organization.

Copies of the related Safety Evaluation and the Federal Register Notice are also enclosed.

Please note that we have discontinued the use of separate identifying numbers for changes to technical specifications. Sequential amendment numbers will continue as in the past.

Sincerely,

Original signed by
R. A. Purple

Robert A. Purple, Chief
Operating Reactors Branch #1

Enclosures and cc: See next page

OFFICE ▶	ORB#1		oeld	ORB#1		
SURNAME ▶	GZech:lb			RAPurple		
DATE ▶	1/08/76	1/ /76	1/ /76	1/ /76		

FEB 1 3 1976

Enclosures:
1. Amendment No. 18 to DPR-38
2. Amendment No. 18 to DPR-47
3. Amendment No. 15 to DPR-55
4. Safety Evaluation
5. Federal Register Notice

cc w/enclosures:
See next page

OFFICE ▶						
SURNAME ▶						
DATE ▶						

cc w/enclosures:
Mr. William L. Porter
Duke Power Company
P. O. Box 2178
422 South Church Street
Charlotte, North Carolina 28242

Mr. Troy B. Conner
Conner & Knotts
1747 Pennsylvania Avenue, NW
Washington, D. C. 20006

Oconee Public Library
201 South Spring Street
Walhalla, South Carolina 29691

Honorable Reese A. Hubbard
County Supervisor of Oconee County
Walhalla, South Carolina 29621

cc w/enclosures & incoming:
Office of Intergovernmental
Relations
116 West Jones Street
Raleigh, North Carolina 27603

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 18
License No. DPR-38

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Duke Power Company (the licensee) dated February 28, 1975, as supplemented June 19, and December 5, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B of Facility License No. DPR-38 is hereby amended to read as follows:

AMERICAN REVOLUTION BICENTENNIAL 1776-1976

"B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective March 15 1976.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by
R. A. Purple

Robert A. Purple, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the
Technical Specifications

Date of Issuance: FEB 1 3 1976

DUKE POWER COMPANY

DOCKET NO. 50-270

OCONEE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 18
License No. DPR-47

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Duke Power Company (the licensee) dated February 28, 1975, as supplemented June 19, and December 5, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B of Facility License No. DPR-47 is hereby amended to read as follows:

OFFICE ▶						
SURNAME ▶						
DATE ▶						

"B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective March 15 1976.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by
R. A. Purple

Robert A. Purple, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the
Technical Specifications

Date of Issuance: FEB 13 1976

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-287

OCONEE NUCLEAR STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 15
License No. DPR-55

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Duke Power Company (the licensee) dated February 28, 1975, as supplemented June 19, and December 5, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B of Facility License No. DPR-55 is hereby amended to read as follows:

AMERICAN REVOLUTION BICENTENNIAL 1776-1976

"B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective March 15 1976.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by
R. A. Purple

Robert A. Purple, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the
Technical Specifications

Date of Issuance: FEB 13 1976

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 18 TO FACILITY LICENSE NO. DPR-38

AMENDMENT NO. 18 TO FACILITY LICENSE NO. DPR-47

AMENDMENT NO. 15 TO FACILITY LICENSE NO. DPR-55

DOCKET NOS. 50-269, 50-270 AND 50-287

Revise Appendix A as follows:

Remove Pages	Insert Pages
6.1-1	6.1-1
6.1-2	6.1-2
6.1-3	6.1-3
6.1-4	6.1-4
6.1-5	6.1-5
6.1-6	6.1-6
6.1-7	6.1-7
6.1-8	6.1-8
6.2-1	6.2-1
6.3-1	6.3-1

6.0 ADMINISTRATIVE CONTOLS

6.1 ORGANIZATION, REVIEW, AND AUDIT

6.1.1 Organization

6.1.1.1 The station Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.1.2 In all matters pertaining to actual operation and maintenance and to these Technical Specifications, the station Manager shall report to and be directly responsible to the Vice President, Steam Production, through the Manager, Nuclear Production. The organization is shown in Figure 6.1-2.

6.1.1.3 The station organization for Operations, Technical Services and Maintenance shall be functionally as shown in Figure 6.1-1. Minimum operating shift requirements are specified in Table 6.1-1.

6.1.1.4 Incorporated in the staff of the station shall be personnel meeting the minimum requirements encompassing the training and experience described in Section 4 of the ANSI N18.1-1971, "Selection and Training of Nuclear Power Plant Personnel."

6.1.1.5 Retraining and replacement of station personnel shall be in accordance with Section 5.5 of the ANSI N18.1-1971, "Selection and Training of Nuclear Power Plant Personnel."

6.1.2 Technical Review and Control

6.1.2.1 Activities

a. Procedures required by Technical Specification 6.4 and other procedures which affect station nuclear safety, and changes (other than editorial or typographical changes) thereto, shall be prepared by a qualified individual/organization. Each such procedure, or procedure change, shall be reviewed by an individual/group other than the individual/group which prepared the procedure, or procedure change, but who may be from the same organization as the individual/group which prepared the procedure, or procedure change. Such procedures and procedure changes may be approved for temporary use by two members of the station staff, at least one of whom holds a Senior Reactor Operator's License on the unit(s) affected. Procedures and procedure changes shall be approved prior to use or within seven days of receiving temporary approval for use by the station Manager; or by the Operating Superintendent, the Technical Services Superintendent or the Maintenance Superintendent, as previously designated by the station Manager.

b. Proposed changes to the Technical Specifications shall be prepared by a qualified individual/organization. The preparation of each proposed Technical Specifications change shall be reviewed by an individual/group other than the individual/group which prepared the proposed change, but who may

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be from the same organization as the individual/group which prepared the proposed change. Proposed changes to the Technical Specifications shall be approved by the station Manager.

c. Proposed modifications to station nuclear safety-related structures, systems and components shall be designed by a qualified individual/organization. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification but who may be from the same organization as the individual/group which designed the modification. Proposed modifications to station nuclear safety-related structures, systems and components shall be approved prior to implementation by the station Manager; or by the Operating Superintendent, the Technical Services Superintendent, or the Maintenance Superintendent, as previously designated by the station Manager.

d. Individuals responsible for reviews performed in accordance with 6.1.2.1.a, 6.1.2.1.b, and 6.1.2.1.c shall be members of the station supervisory staff, previously designated by the station Manager to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated station review personnel.

e. Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be reviewed by the station Manager; or by the Operating Superintendent, the Technical Services Superintendent or the Maintenance Superintendent, as previously designated by the station Manager.

f. Incidents reportable pursuant to Technical Specification 6.6.2.1 and violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. Such reports shall be approved by the station Manager and transmitted to the Vice President, Steam Production, or his designee; and to the Director of the Nuclear Safety Review Board.

g. The station Manager shall assure the performance of special reviews and investigations, and the preparation and submittal of reports thereon, as requested by the Vice President, Steam Production.

h. The station security program, and implementing procedures, shall be reviewed at least annually. Changes determined to be necessary as a result of such review shall be approved by the station Manager and transmitted to the Vice President, Steam Production, or his designee; and to the Director of the Nuclear Safety Review Board.

i. The station emergency plan, and implementing procedures, shall be reviewed at least annually. Changes determined to be necessary as a result of such review shall be approved by the station Manager and transmitted to the Vice President, Steam Production, or his designee; and to the Director of the Nuclear Safety Review Board.

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6.1.2.2 Records

Records of the above activities shall be maintained.

6.1.3 Nuclear Safety Review Board

6.1.3.1 Function

The NSRB shall function to provide independent review and audit of designated activities in the areas of:

a. Nuclear power plant operations
b. Nuclear Engineering
c. Chemistry and radiochemistry
d. Metallurgy
e. Instrumentation and control
f. Radiological safety
g. Mechanical and electrical engineering
h. Administrative control and quality assurance practices

6.1.3.2 Organization

a. The Director, members and alternate members of the NSRB shall be formally appointed by the Vice President, Steam Production; shall have an academic degree in an engineering or physical sceince field; and in addition, shall have a minimum of five years technical experience, of which a minimum of three years shall be in one or more areas given in 6.1.3.1.

b. The NSRB shall be composed of at least five members, including the Director, Members of the NSRB may be from the Steam Production Department, from other departments within the Company or from external to the Company. A maximum of one member of the NSRB may be from the Oconee Nuclear Station staff.

c. Consultants may be utilized by the NSRB to provide expert advice to the NSRB, as determined necessary by the Director of the NSRB.

d. Staff assistance may be provided to the NSRB in order to promote the proper, timely and expeditious performance of its functions.

e. The NSRB shall meet at least once per six months. The period between such meetings shall not exceed eight months.

f. A quorum of the NSRB shall consist of the Director, or his designated alternate, and at least two other NSRB members or alternate members. No more than a minority of the quorum shall have line responsibility for operation of Oconee Nuclear Station.

6.1.3.3 Subjects Requiring Review

The following subjects shall be reported to and reviewed by the NSRB:

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a. The safety evaluations for (1) changes to procedures, equipment or systems, and (2) tests or experiments completed under the provisions of 10 CFR 50.59(a)(1) to verify that such actions did not constitute an unreviewed safety question.

b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in 10 CFR 50.59.

c. Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59.

d. Proposed changes in Technical Specifications or the Facility Operating Licenses.

e. Violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.

f. Significant operating abnormalities or deviations from normal and expected performance of station equipment that affect nuclear safety.

g. Incidents that are the subject of non-routine reports submitted to the Commission.

h. Quality Assurance Department audits relating to station operations and actions taken in response to these audits.

6.1.3.4 Audits

Audits of station activities shall be performed under the cognizance of the NSRB. These audits shall encompass:

a. The conformance of station operation to provisions contained within the Technical Specifications and applicable facility operating license conditions at least once per year.

b. The performance, training and qualifications of the station staff at least once per year.

c. The results of actions taken to correct deficiencies occurring in equipment, structures, systems or methods of operation that affect nuclear safety at least once per six months.

d. The performance of activities required by the quality assurance program to meet the criteria of Appendix B to 10 CFR 50 at least once per two years.

e. The station emergency plan and implementing procedures at least once per two years.

f. The station security plan and implementing procedures at least once per two years.

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g. Any other area of station operation considered appropriate by the NSRB or the Vice President, Steam Production.

6.1.3.5 Responsibilities and Authorities

a. The NSRB shall report to and advise the Vice President, Steam Production, on those areas of responsibility specified in Specifications 6.1.3.3 and 6.1.3.4.

b. Minutes shall be prepared and forwarded to the Vice President, Steam Production, and to the Senior Vice President, Production and Transmission, within 14 days following each formal meeting of the NSRB.

c. Records of activities performed in accordance with Specifications 6.1.3.3 and 6.1.3.4 shall be maintained.

d. Audit reports encompassed by Section 6.1.3.4 shall be forwarded to the Vice President, Steam Production, and to the Senior Vice President, Production and Transmission, and to the management positions responsible for the areas audited within 30 days of completion of each audit.

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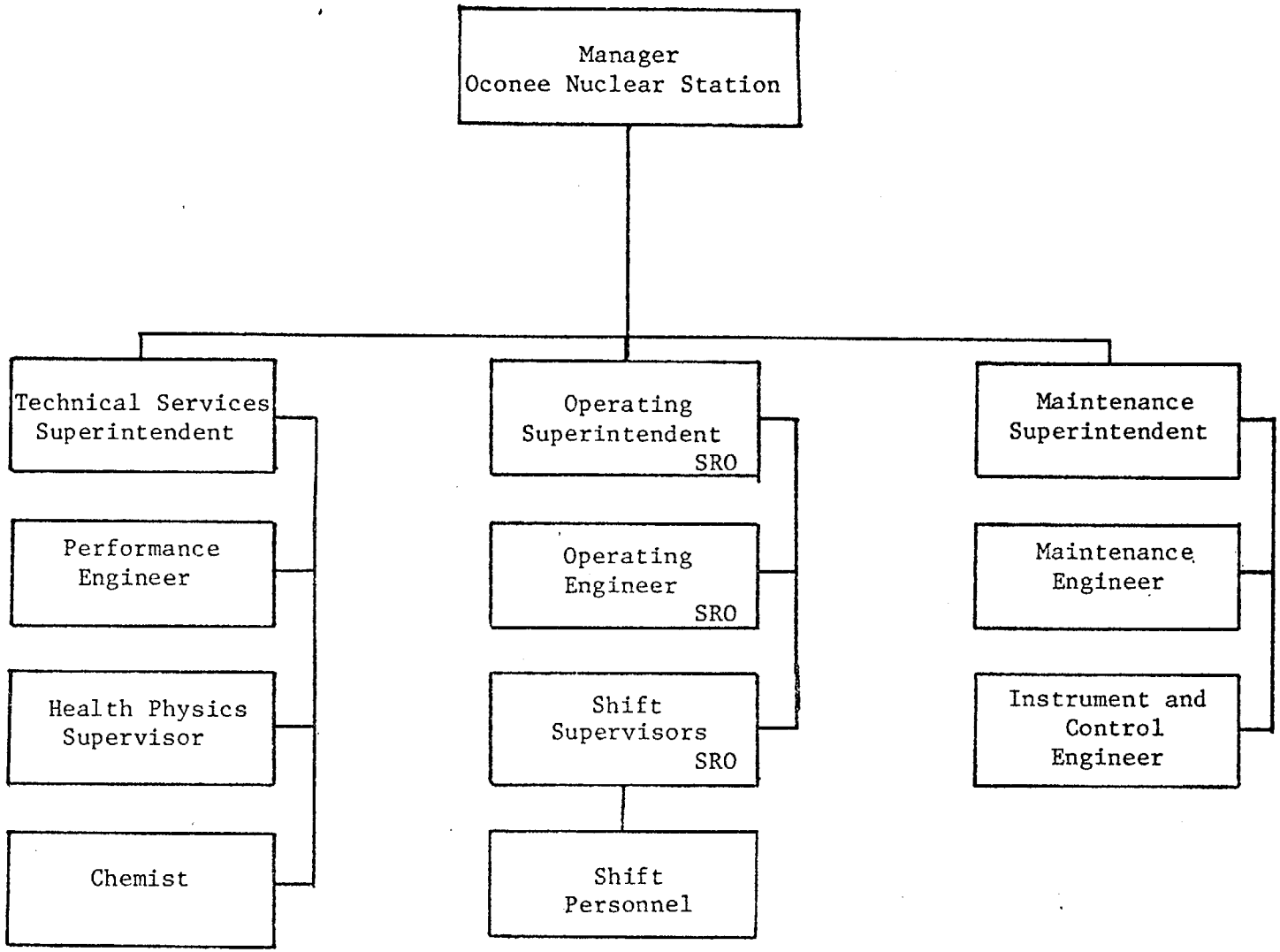
Table 6.1-1
Minimum Operating Shift Requirements
With Fuel in the Three Reactor Vessels

Minimum AEC License Requirement	Unit 1 or 2 Above Cold Shutdown; Unit 3 Cold Shutdown	Units 1 and 2 Above Cold Shutdown; Unit 3 Cold Shutdown	Units 1 or 2 Above Cold Shutdown; Unit 3 Above Cold Shutdown	Units 1 and 2 Cold Shutdown; Unit 3 Above Cold Shutdown	Units 1, 2, and 3 Above Cold Shutdown	Units 1, 2 and 3 Cold Shutdown
Senior Reactor Operator	2	2	2	2	3	2
Reactor Operator	4	4	4	4	4	3
Unlicensed Operator	2	2	2	2	4	

Additional Requirements:

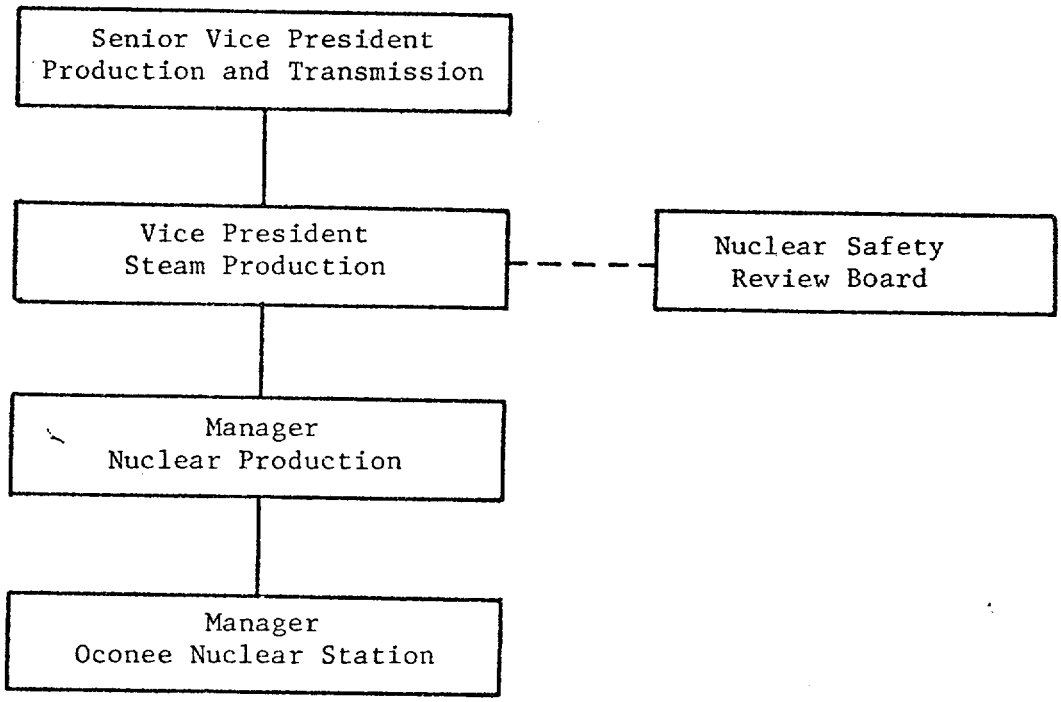
1. One licensed operator per unit shall be in the Control Room at all times when there is fuel in the reactor vessel
2. Two licensed operators shall be in the Control Room during startup and scheduled shutdown of a reactor.
3. At least one licensed operator shall be in the reactor building when fuel handling operations in the reactor building are in progress.
4. An operator holding a Senior Reactor Operator license and assigned no other operational duties shall be in direct charge of refueling operations.
5. At least one person per shift shall have sufficient training to perform routine health physics requirements.
6. If the computer for a reactor is inoperable for more than eight hours, an operator in addition to those required above shall supplement the shift crew.



OCONEE NUCLEAR STATION

STATION ORGANIZATION CHART

FIGURE 6.1-1 | 14/9/1



OCONEE NUCLEAR STATION

MANAGEMENT ORGANIZATION CHART

Figure 6.1-2

Entire Page Revised

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6.2 ACTION TO BE TAKEN IN THE EVENT OF A REPORTABLE OCCURRENCE

6.2.1 Any reportable occurrence shall be investigated promptly by the station Manager.

6.2.2 The Vice President, Steam Production, shall be notified of any reportable occurrence. A written report shall be prepared which describes the circumstances leading up to and resulting from the incident and shall recommend appropriate action to prevent or minimize the probability of a recurrence. The report shall be submitted to the Vice President, Steam Production, and to the Director of the Nuclear Safety Review Board.

6.2.3 The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.6.2.

Entire Page Revised

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6.3 ACTION TO BE TAKEN IN THE EVENT A SAFETY LIMIT IS EXCEEDED

6.3.1 If a safety limit is exceeded, the reactor shall be shut down immediately and maintained in a safe shutdown condition until the Commission authorizes resumption of operation.

6.3.2 The violation of a safety limit shall be reported to the Commission, the Vice President, Steam Production, and to the Director of the Nuclear Safety Review Board immediately.

6.3.3 A report shall be prepared which describes (1) applicable circumstances preceding the violation, (2) effects of the violation upon structures, systems or components, and (3) corrective action taken to prevent recurrence. The report shall be reviewed by the Operating Superintendent and the station Manager. The report shall be submitted to the Vice President, Steam Production, and the Director of the Nuclear Safety Review Board.

6.3.4 A report of the violation, with appropriate analyses and corrective action to prevent recurrence shall be submitted to the Commission within 10 days of the violation.

Entire Page Revised

FEB 1 3 1976

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 18 TO FACILITY LICENSE NO. DPR-38

SUPPORTING AMENDMENT NO. 18 TO FACILITY LICENSE NO. DPR-47

SUPPORTING AMENDMENT NO. 15 TO FACILITY LICENSE NO. DPR-55

DUKE POWER COMPANY

OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3

DOCKET NOS. 50-269, 50-270 AND 50-287

Introduction

By letter dated February 28, 1975, as amended June 19 and December 5, 1975, Duke Power Company (the licensee) requested changes to the Technical Specifications appended to Facility Operating Licenses DPR-38, DPR-47, and DPR-55 for the Oconee Nuclear Station. The proposed changes consist of revised portions of Section 6.1, Organization, Review and Audit, and involve the following:

1. The assignment of required technical review and control functions to qualified individuals or groups rather than to an onsite review committee

2. The assignment of several review functions to the onsite operating organization rather than to an onsite review committee, and

3. A restructured review and audit organization.

Discussion

The Commission requested, by letter dated December 18, 1974, that the licensee review Section 6.1 of its Technical Specifications, Organization, Review and Audit, and submit a proposed change to include the requirements of the guidance provided.

AMERICAN REVOLUTION BICENTENNIAL 1776-1976

By letter dated February 28, 1975, as amended June 19 and December 5, 1975, the licensee submitted a proposed amendment in response to our request and, in addition, requested other changes to the existing technical specifications relating to Section 6.

Specifically, the licensee is requesting the elimination of the Station Review Committee (SRC) as the onsite organization responsible for technical review and control functions. Qualified individuals or groups from the cognizant organization, as selected by the Station Manager, would substitute for the SRC in the review of the following:

1. Procedures and procedure changes which affect station nuclear safety,

2. Proposed changes to the Technical Specifications, and

3. Proposed modifications to nuclear safety related structures, systems and components.

The present policy regarding the above three areas is that an original draft is prepared by qualified individuals or groups from within the responsible organization. A review of the draft is then conducted by the SRC, and recommendations made to the Station Manager regarding the disposition of the orginial proposal. The proposed change would require that a second-party review be conducted by qualified individuals or groups from within the responsible organization, rather than by the SRC, prior to final approval by the Station Manager or his designated alternate.

For other matters normally assigned to the SRC, such as incident investigation, performance of special reviews, review of the station security plan and review of the station emergency plan, the licensee is proposing that the Station Manager assign personnel of the onsite operating organization to perform these functions, rather than having these functions performed by the onsite review committee as presently specified.

The licensee is also proposing the establishment of a Nuclear Safety Review Board (NSRB) to replace the existing Nuclear Safety Review Committee (NSRC). The NSRB would be staffed by five members with the required length of technical experience plus consultants to provide necessary expert advice and additional staff members to assist the NSRB. The NSRB would perform the required independent review and audit functions to verify that the Oconee Nuclear Station is operated and administered in accordance with acceptable and approved standards.

Evaluation

The present procedures utilized by Oconee Nuclear Station for originating procedures, procedure changes and system modifications specify that the cognizant department submit the proposed document to the onsite Station Review Committee (SRC) for review. The SRC then conducts its review and makes its recommendations to the Station Manager who then makes the final approval.

The Station Manager is responsible for appointing members of the SRC Committee, and is required to assign at least 5 members from the station supervisory staff. Representatives are to be from Operations and Technical Services. The present technical specifications refer to ANSI N18.1, Selection and Training of Nuclear Power Plant Personnel, for guidelines for the training and experience requirements of the station supervisory staff.

The proposed amendment would assign qualified individuals or groups from the station supervisory staff to replace the SRC in the review of procedures, procedure changes, technical specification changes and plant modifications involving nuclear safety. These individuals/groups would be previously designated by the Station Manager to perform the reviews. The final approval of the above reviews would be by the Station Manager or, in certain cases, by his designated representative. The individual or group conducting the review would be other than the individual or group who prepared the procedure, procedure change, technical specification change or plant modification, but may be from the same organization as the individual or group which originated the proposal. In addition, for each review conducted, a determination would be made as to whether or not additional, cross-disciplinary review is necessary. If concluded that it is necessary, the additional review would be performed by the appropriate designated station review personnel.

The existing Technical Specifications and the proposed amendment both reference ANSI N18.1 for the minimum training and experience requirements for the station staff and therefore there would be no lessening of the qualifications of the individuals conducting the reviews.

During our review of the proposed changes, we found that certain modifications to the proposal were necessary. These changes were discussed with the licensee and have been incorporated into the proposal.

In view of the above, it is concluded that the review of procedures and procedure changes involving nuclear safety, technical specification changes and plant modifications involving nuclear safety, can be effectively conducted by the licensee, utilizing the methods of the proposed amendment.

The proposed amendment would also reassign other matters presently within the purview of the onsite review committee (SRC), such as incident investigation, performance of special reviews, review of the station security program and review of the station emergency plan, to individuals or ad hoc groups from the operating organization, as assigned by the Station Manager. This would effectively be an equivalent review in that, as presently specified, the members of the onsite review Committee are assigned by the Station Manager from the supervisory personnel of the operating organization. In view of this, and of the preceding discussion, we agree that the Station Review Committee can be eliminated and its reponsibilities assigned to the individuals and groups as discussed herein.

The licensee has also proposed that an off-site Nuclear Safety Review Board (NSRB) be established to replace the present Nuclear Safety Review Committee (NSRC). The NSRB would be staffed by five members, all of whom would have a minimum of nine years technical experience, of which three years would be in one or more of the areas of expertise listed in ANSI N18.7. A maximum of four years of the nine years of experience would be allowed as academic training. These experience requirements represent an upgrading from the existing requirements of the NSRC which consist of an engineering or science degree plus a minimum of three years nuclear experience. In addition, consultants would be utilized to provide expert advice, as determined necessary by the Director of the NSRB. Staff assistance would also be provided to assist the NSRB in the performance of its functions.

The proposed amendment would expand the responsibilities of the NSRB as compared to the present NSRC in that (1) all safety evaluations for actions completed under the provisions of 10 CFR 50.59 would be reviewed, not just those which were considered to involve unreviewed safety questions and (2) all proposed Technical Specification or operating license changes would be reviewed, not just those considered to involve unreviewed safety questions.

We have concluded that the responsibilities of the proposed NSRB are consistent with those presently required of review and audit programs and that the means by which it will carry out these responsibilities are acceptable. In view of this, we agree with the proposed establishment of the NSRB as a replacement for the NSRC.

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental statement, negative declaration, or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: FEB 1 3 1976

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-269, 50-270 AND 50-287

DUKE POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Amendments No. 18, 18 and 15 to Facility Operating Licenses No. DPR-38, DPR-47 and DPR-55, respectively, issued to Duke Power Company which revised Technical Specifications for operation of the Oconee Nuclear Station, Units 1, 2 and 3, located in Oconee County, South Carolina. The amendments are effective March 15 1976.

These amendments revise portions of the Organization, Review and Audit section of the Technical Specifications and consist of (1) changes in the methods by which technical review and control functions are performed, (2) the reassignment of certain specific review requirements to the onsite operating organization and (3) a restructured review and audit organization.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments is not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendments dated February 28, 1975, as supplemented June 19, and December 5, 1975, (2) Amendments No. 18, 18, and 15 to Licenses No. DPR-38, DPR-47, and DPR-55, (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D.C. and at the Oconee County Library, 201 South Spring Street, Walhalla, South Carolina 29691.

A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this FEB 13 1976

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by
R. A. Purple

Robert A. Purple, Chief
Operating Reactors Branch #1
Division of Operating Reactors