

Docket Nos. 50-269
50-270
and 50-287

JANUARY 30 1978

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Duke Power Company
ATTN: Mr. William O. Parker, Jr.
Vice President - Steam Production
Post Office Box 2178
422 South Church Street
Charlotte, North Carolina 28242

Gentlemen:

The Commission has issued the enclosed Amendments Nos. 55, 55, and 52 for License Nos. DPR-38, DPR-47 and DPR-55 for the Oconee Nuclear Station, Unit Nos. 1, 2 and 3. These amendments consist of changes to the Station's common Technical Specifications and are in response to your requests dated October 31, 1977 and December 2, 1977.

These amendments revise the Technical Specifications to: (1) delete the requirements for Annual Reports, (2) require written confirmation for prompt reportable occurrences, and (3) delete the Technical Specification section on Respiratory Protection Program.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original Signed By

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:

1. Amendment No. 55 to License DPR-38
2. Amendment No. 55 to License DPR-47
3. Amendment No. 52 to License DPR-55
4. Safety Evaluation
5. Notice of Issuance

cc w/enclosure:
see next page

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SURNAME >	DNeighbors:at		ASchwencer				
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W

cc: Mr. William L. Porter
Duke Power Company
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422 Church Street
Ch , North Carolina 28242

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DeBevoise & Liberman
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Oconee Public Library
201 South Spring Street
Walhalla, South Carolina 29691

Honorable James M. Phinney
County Supervisor of Oconee County
Walhalla, South Carolina 29621

Chief, Energy Systems
Analyses Branch (AW-459)
Office of Radiation Programs
U. S. Environmental Protection Agency
Room 645, East Tower
401 M Street, S. W.
Washington, D. C. 20460

U. S. Environmental Protection Agency
Region IV Office
ATTN: EIS COORDINATOR
345 Coutland Street, N. E.
Atlanta, Georgia 30308

Chrys Baggett
State Clearinghouse
Division of Policy Development
116 West Jones Street
Raleigh, N.C. 27603



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 55
License No. DPR-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Duke Power Company (the licensee) dated October 31, 1977 and December 2, 1977, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility License No. DPR-38 is hereby amended to read as follows:

"3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 55, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications."

3. This license amendment is effective April 1, 1978.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: **JANUARY 30 1978**



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-247

OCONEE NUCLEAR STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 55
License No. DPR-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Duke Power Company (the licensee) dated October 31, 1977 and December 2, 1977, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility License No. DPR-47 is hereby amended to read as follows:

"3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 55, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications."

3. This license amendment is effective April 1, 1978.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: **JANUARY 30 1978**



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-287

OCONEE NUCLEAR STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 52
License No. DPR-55

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Duke Power Company (the licensee) dated October 31, 1977 and December 2, 1977, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility License No. DPR-55 is hereby amended to read as follows:

"3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 52, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications."

3. This license amendment is effective April 1, 1978.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: **JANUARY 30 1978**

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 55 TO DPR-38

AMENDMENT NO. 55 TO DPR-47

AMENDMENT NO. 52 TO DPR-55

DOCKET NOS. 50-269, 50-270 AND 50-287

Remove the following pages and insert revised identically numbered pages:

6.6-1
6.6-2
6.6-3
6.6-4
6.6-5

Delete the following pages:

6.7-1
6.7-2
6.7-3
6.7-4
6.7-5
6.7-6

<u>Section</u>		<u>Page</u>
6.3	ACTION TO BE TAKEN IN THE EVENT A SAFETY LIMIT IS EXCEEDED	6.3-1
6.4	STATION OPERATING PROCEDURES	6.4-1
6.5	STATION OPERATING RECORDS	6.5-1
6.6	STATION REPORTING REQUIREMENTS	6.6-1
6.6.1	<u>Routine Reports</u>	6.6-1
6.6.2	<u>Non-Routine Reports</u>	6.6-6
6.6.3	<u>Special Reports</u>	6.6-9

6.6 STATION REPORTING REQUIREMENTS

6.6.1 Routine Reports

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Director of the Office of Inspection and Enforcement, Region II unless otherwise noted.

6.6.1.1 Startup Report

A summary report of unit startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the facility license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal or hydraulic performance of the unit. Startup reports shall be submitted (1) within 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) nine months following initial criticality, whichever occurs first. If a startup report does not cover all three events, i.e., initial criticality, completion of the startup test program and resumption or commencement of commercial power operation, supplementary reports shall be submitted at least every three months until all three events are completed.

6.6.1.2 Monthly Operating Report

Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Director, Office of Management Information and Program Control, U.S. Nuclear Regulatory Commission, Washington, D.C., 20555, with a copy to the appropriate Regional Office, to be submitted by the fifteenth of each month following the calendar month covered by the report.

6.6.1.3 Personnel Exposure and Monitoring Report

A tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions, ^{1/}e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

6.6.1.4 Reporting of Radioactive Effluent Releases

Data shall be reported to the Commission semi-annually in a form similar to that shown in Table 6.6-1 and shall include the following:

- (1) Gaseous Releases
 - (a) Total radioactivity (in curies) releases of noble and activation gases.
 - (b) Maximum noble gas release rate during any one-hour period.
 - (c) Total radioactivity (in curies) released, by nuclide, based on representative isotopic analyses performed.

^{1/}This tabulation supplements the requirements of §20.407 of 10 CFR Part 20

(d) Percentage applicable limits released.

(2) Iodine Releases

(a) Total I-131, I-133, I-135 radioactivity (in curies) released.

(b) Total radioactivity (in curies) released, by nuclide, based on representative isotopic analyses performed.

(c) Percentage of limit.

(3) Particulate Releases

(a) Gross radioactivity (β - γ) released (in curies) excluding background radioactivity.

(b) Gross alpha radioactivity released (in curies) excluding background radioactivity.

(c) Total radioactivity released (in curies) of nuclides with half-lives greater than eight days.

(d) Percentage of limit.

(4) Liquid Releases

(a) Gross radioactivity (β - γ) released (in curies) excluding tritium and average concentration released to the unrestricted area at the Keowee Hydro unit.

(b) The maximum concentration of gross radioactivity (β - γ) released to the unrestricted area (averaged over the period of release).

(c) Total tritium and alpha radioactivity (in curies) released and average concentration released to the unrestricted area at the Keowee Hydro unit.

(d) Total dissolved gas radioactivity (in curies) and average concentration released to the unrestricted area at the Keowee Hydro unit.

(e) Total volume (in liters) of Keowee Hydro liquid waste released.

(f) Total volume (in liters) of dilution water used prior to release from the restricted area.

(g) Total radioactivity (in curies) released, by nuclide, based on representative isotopic analyses performed.

(h) Percentage of limit for total activity released.

(5) Solid Waste

- (a) The total amount of solid waste packaged (in cubic feet).
- (b) Estimated total radioactivity (in curies):
- (c) Disposition including date and destination if shipped off site.

6.6.1.5. Environmental Monitoring

An Environmental Monitoring Report containing the following information shall be submitted prior to March 1 of each year.

- (a) For each medium sampled during the reporting period, the following information shall be provided.
 - 1. Number of sampling locations.
 - 2. Total number of samples.
 - 3. Number of locations at which levels are found to be significantly greater than local backgrounds.
 - 4. Highest, lowest, and the average concentrations or levels of radiation for the sampling point with the highest average and description of the location of that point with respect to the site.
- (b) If levels of station-contributed radioactive materials in environmental media indicate the likelihood of public intakes in excess of 3 percent of those that could result from continuous exposure to the concentration values listed in Appendix B, Table II, Part 20, estimates the likely resultant exposure to individuals and to population groups, and assumptions upon which estimates are based shall be provided. (These values are comparable to the top of Range I, as defined in FRC Report No. 2.)
- (c) If statistically significant variations in off-site environmental concentrations with time are observed and are attributed to station releases, correlation of these results with effluent releases shall be provided.

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6.6.2 Non-Routine Reports

6.6.2.1 Reportable Occurrences

a. Prompt Notification with Written Followup

The types of events listed below shall be reported within 24 hours of discovery by telephone, and confirmed by telegraph, mailgram or facsimile no later than the first normal working day following the event to the Director, Office of Inspection and Enforcement, Region II, or his designate with a written followup report within two weeks to the Director, Office of Inspection and Enforcement, Region II (copy to the Director, Office of Management Information and Program Control).

- (1) Failure of the Reactor Protective System to trip, as required, when a monitored parameter reaches the setpoint specified as the limiting safety system setting in the Technical Specifications.
- (2) Operation of the unit or affected systems when any parameter or operation subject to a limiting condition for operation is less conservative than the least conservative aspect of the limiting condition for operation established in the Technical Specifications.
- (3) Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary or primary containment.
- (4) Reactivity anomalies involving disagreement with predicted value of reactivity balance under steady-state conditions greater than or equal to 1% $\Delta k/k$; a calculated reactivity balance indicating shutdown margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds, or if subcritical, an unplanned reactivity insertion of more than 0.5% $\Delta k/k$; or any unplanned criticality.
- (5) Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the Safety Analysis Report.
- (6) Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the Safety Analysis Report.
- (7) Conditions arising from natural or man-made events that, as a direct result of the event, require unit shutdown, operation of safety systems, or other protective measures required by Technical Specifications.
- (8) Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the Safety Analysis Report or in the bases for the Technical Specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 55 TO FACILITY LICENSE NO. DPR-38

AMENDMENT NO. 55 TO FACILITY LICENSE NO. DPR-47

AMENDMENT NO. 52 TO FACILITY LICENSE NO. DPR-55

DUKE POWER COMPANY

OCONEE NUCLEAR STATION, UNIT NOS. 1, 2 AND 3

DOCKET NOS. 50-269, 50-270 AND 50-287

Introduction

By letter dated October 31, 1977, Duke Power Company (the licensee) requested changes to the Technical Specifications incorporated in the Oconee Nuclear Station operating licenses which would delete the requirement for Annual Reports.

By letter dated December 2, 1977, the licensee requested a change to provide written confirmation for prompt reportable occurrences. We are also taking this opportunity to remove the sections of the Technical Specifications pertaining to the Respiratory Protection Program.

Discussion/Evaluation

1. Reporting Requirements

Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", is the basis for reporting requirements found in Technical Specifications today. When these Technical Specifications were issued we requested that licensees use the formats in the guide for the Licensee Event Report (LER) and Monthly Operating Report. In some cases licensees' use of these formats was required by a reference to Regulatory Guide 1.16 in the Technical Specifications. After two years of experience with the reporting requirements identified in this guide we reviewed the scope of information licensees are required to submit in the LER, Annual Operating Report, Monthly Operating Report and Startup Report.

Based on our review of LER's we developed a modified format for the LER to make this document more useful for evaluation purposes. By letters sent in July and August 1977, we informed licensees of the new LER format and requested that they use it. For those licensees who reference Regulatory Guide 1.16 in their Technical Specifications we also requested that they propose a change which would replace this reference with appropriate words from the guide and which would delete mandatory use of the reporting forms contained in the guide.

From our review of all licensee reports we determined that much of the information found in the Annual Operating Report either is addressed in the LER's or Monthly Operating Reports, which are submitted in a more timely manner, or could be included in these reports with only a slight augmentation of the information already supplied. Therefore we concluded that the Annual Operating Report could be deleted as a Technical Specification requirement if certain additional information were provided in the Monthly Operating Reports. As a result we sent letters during September 1977 to licensees informing them that a revised and improved format for Monthly Operating Reports was available and requested that they use it. For those licensees with the Technical Specification reference to Regulatory Guide 1.16 the change deleting this reference, discussed above, would be necessary. In addition, licensees were informed that if they agreed to use the revised format they should submit a change request to delete the requirement for an Annual Operating Report except that occupational exposure data must still be submitted.

The licensee has proposed to delete all but one specified item in the Annual Operating Report. The report which tabulates occupational exposure on an annual basis is needed and therefore, the requirement to submit this information has been retained. The wording has been changed to reflect that used in Standard Technical Specifications. We have determined that the failed fuel examination information does not need to be supplied routinely by licensees because this type of historical data can be obtained in a compiled form from fuel vendors when needed. The information concerning forced reductions in power and outages will be supplied in the revised Monthly Operating Reports and the narrative summary of operating experience will be provided in a monthly basis in the Monthly Operating Report rather than annually. The licensee has committed to use the revised Monthly Operating Report format beginning with its report for January 1978 as requested. We have concluded that all needed information will be provided and deletion of the Annual Operating Report is acceptable.

This change to Section 6.6.2.1a makes the Oconee Technical Specifications consistent with the reporting requirements for other facilities and brings them into conformance with NRC intentions in this regard. This change is administrative in nature.

2. Respiratory Protection Program

The deletion of the current respiratory protection requirements from the Technical Specifications would eliminate conflict between the Technical Specifications and 10 CFR §20.103 as revised November 29, 1976. This agrees with the revocation provision in Section 6.7.1.a.g of the current Technical Specifications which states that Section 6.7 will be superceded upon adoption of the proposed change to 10 CFR §20.103. In the future, as specified in the regulations, allowance may be made for the use of respiratory protective equipment only if its use is as stipulated in Regulatory Guide 8.15, Acceptable Programs for Respiratory Protection. Based on the above, we find this change acceptable.

Environmental Consideration

We have determined that this amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact, and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: JANUARY 30 1976

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-269, 50-270 AND 50-287DUKE POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 55, 55 and 52 to Facility Operating License Nos. DPR-30, DPR-47 and DPR-55, respectively, issued to Duke Power Company for operation of the Oconee Nuclear Station, Unit Nos. 1, 2 and 3, located in Oconee County, South Carolina. The amendments are effective as of the date of issuance.

These amendments revise the Technical Specifications to: (1) delete the requirements for Annual Reports, (2) require written confirmation for prompt reportable occurrences, and (3) delete the Technical Specification section on Respiratory Protection Program.

The applications for the amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §1.5(d)(4) an environmental impact statement or negative

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declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

For further details with respect to this action, see (1) the applications for amendments dated October 31, 1977, and December 2, 1977, (2) Amendment Nos. 55 , 55 and 52 to License Nos. DPR-38, DPR-47 and DPR-55, respectively, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. and at the Oconee County Library, 201 South Spring Street, Wallhalla, South Carolina 29691. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 30th day of January 1978.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors