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FEBRUARY 16 1978

Docket Nos. 50-269
50-270
and 50-287

Duke Power Company
ATTN: Mr. William O. Parker, Jr.
Vice President - Steam Production
422 South Church Street
P. O. Box 2178
Charlotte, North Carolina 28242

Gentlemen:

The Commission has issued the enclosed Amendments Nos. 58, 58, and 55 for Licenses Nos. DPR-38, DPR-47 and DPR-55 for the Oconee Nuclear Station, Units 1, 2 and 3. These amendments consist of changes to the Station's common Technical Specifications and are in response to your request dated July 18, 1977.

These amendments revise the Technical Specifications Table 4.18-1 by adding shock suppressors and changing the classification of the location of certain shock absorbers.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original Signed By

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:

1. Amendment No. 58 to DPR-38
2. Amendment No. 58 to DPR-47
3. Amendment No. 55 to DPR-55
4. Safety Evaluation
5. Notice of Issuance

cc w/enclosures:
see next page

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February 16, 1978

cc: Mr. William L. Porter
Duke Power Company
P. O. Box 2178
422 South Church Street
Charlotte, North Carolina 28242

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Oconee Public Library
201 South Spring Street
Walhalla, South Carolina 29691

Honorable James M. Phinney
County Supervisor of Oconee County
Walhalla, South Carolina 29621

Chief, Energy Systems
Analyses Branch (AW-459)
Office of Radiation Programs
U. S. Environmental Protection Agency
Room 645, East Tower
401 M Street, S. W.
Washington, D. C. 20460

U. S. Environmental Protection Agency
Region IV Office
ATTN: EIS COORDINATOR
345 Coutland Street, N. E.
Atlanta, Georgia 30308

Chrys Baggett
State Clearinghouse
Division of Policy Development
116 West Jones Street
Raleigh, N.C. 27603



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 58
License No. DPR-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duke Power Company (the licensee) dated July 18, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

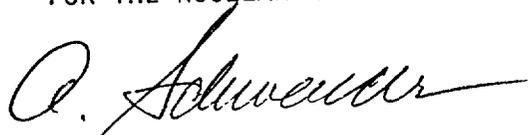
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility License No. DPR-38 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 58, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 16, 1978



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-270

OCONEE NUCLEAR STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 58
License No. DPR-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duke Power Company (the licensee) dated July 18, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility License No. DPR-47 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 58, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 16, 1978



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-287

OCONEE NUCLEAR STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 55
License No. DPR-55

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duke Power Company (the licensee) dated July 18, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility License No. DPR-55 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 55, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 16, 1978

ATTACHMENT TO LICENSE AMENDMENT NOS. 58, 58 AND 55
FACILITY OPERATING LICENSE NOS. DPR-38, DPR-47 AND DPR-55
DOCKET NOS. 50-269, 50-270 AND 50-287

Revise Appendix A as follows:

1. Remove the following pages and replace with identically numbered revised pages.

4.18-3
4.18-4
4.18-5
4.18-6
4.18-7
4.18-8
4.18-9
4.18-10
4.18-11
4.18-12
4.18-13

2. Add page 4.18-11a

TABLE 4.18-1
Unit 1 Safety Related Shock Suppressors (Snubbers)

<u>Sketch/Hanger No.</u>	<u>System</u>	<u>Suppressor Especially Difficult To Remove</u>	<u>Suppressor Inaccessible During Normal Operation</u>	<u>Suppressor in High Radiation Area During Shutdown*</u>	
1-124	Main Steam Line (01A)		X		
1-125(A&B)			X		
1-127					
1-128					
1-129					
1-130			X		
1-132(A,B,C,D)				X	
1-134					
1-135					
1-147					
1-149(A&B)				X	
1-151					
1-152					
H 11A				X	
H 12A				X	
H 10B			X		
H 11B			X		
1-941	Main Steam Bypass to Condenser (01A-1)				
1-944					
1-945					
1-3135	Main Steam Supply to Auxiliary Equipment (01A-3)				
1-1305	Main Steam Supply to Emergency Feedwater Pump Turbine (01A-4)				
1-1310					
1-1315					
H 7B	Main Feedwater Line (03)		X		
H 10A			X		

4.18-3

Amendments - 58, 58-A, 55

TABLE 4.18-1
Unit 1 Safety Related Shock Suppressors (Snubbers)

<u>Sketch/Hanger No.</u>	<u>System</u>	<u>Suppressor Especially Difficult To Remove</u>	<u>Suppressor Inaccessible During Normal Operation</u>	<u>Suppressor in High Radiation Area During Shutdown*</u>	
1-1289	Emergency Feedwater Line (03A)				
1-1292					
1-1293					
1-1294					
1-1295					
1-1296					
1-1297					
1-1298					
1-1299					
1-5600					
1-5601					
1-5602			X		X
1-5603			X		X
1-5604					
1-5605					
1-5606					
H 7B			X		
1-4100	Reactor Coolant System (50)		X		
1-4102			X		
1-4104			X		
1-4105			X		
1-4107			X		
1-4109			X		
1-4111			X		
1-4112			X		
1-4113			X		
1-4115			X		
1-4116			X		
1-4117			X		
H 1				X	
H 3			X		
H 4			X		
H 5			X		

4.18-4

Amendments 58, 58 & 55

TABLE 4.18-1
Unit 1 Safety Related Shock Suppressors (Snubbers)

Sketch/Hanger No.	System	Suppressor Especially Difficult To Remove	Suppressor Inaccessible During Normal Operation	Suppressor in High Radiation Area During Shutdown*
H 7	Reactor Coolant System (50) (Continued)		X	
H 8			X	
H 9			X	
H 10			X	
H 11			X	
H 12			X	
H 1A			X	
H 2A			X	
H 3A			X	
H 17A	High Pressure Injection System (51)		X	
H 1E			X	
H 5 (A&B)	Low Pressure Injection System (53)		X	
H 40C			X	
H 41C			X	
1-2139	Reactor Building Spray System (54)			
1-2149				
H 9A			X	
H 9B			X	
H 5	Pressurizer Relief Valve Discharge (57)		X	
H 6			X	
H 9			X	
H 10			X	
H 11			X	
H 14			X	
H 15			X	
H 17			X	
H 18			X	
H 22			X	
H 26			X	
H 27		X		

4.18-5

Amendments 58, 58 & 55

TABLE 4.18-1
Unit 2 Safety Related Shock Suppressors (Snubbers)

Sketch/Hanger No.	System	Suppressor Especially Difficult To Remove	Suppressor Inaccessible During Normal Operation	Suppressor in High Radiation Area During Shutdown*	
2-124	Main Steam Line (01A)		X		
2-125 (A&B)			X		
2-127					
2-128					
2-129					
2-130					
2-132 (A,B,C,D)				X	
2-134					
2-135					
2-147					
2-149 (A&B)			X		
2-151					
2-152					
H 2A			X		
H 8A			X		
H 2B			X		
H 8B			X		
2-941	Main Steam Bypass to Condenser (01A-1)				
2-944					
2-945					
2-3135	Main Steam Supply to Auxiliary Equipment (01A-3)				
2-1309	Main Steam Supply to Emergency Feedwater Pump Turbine (01A-4)				
2-1322					
2-1323					
2-1324					
2-1326					
2-1327					
2-1329					
2-1333					

4.18-6

Amendments 58, 58 & 55

TABLE 4.18-1
Unit 2 Safety Related Shock Suppressors (Snubbers)

Sketch/Hanger No.	System	Suppressor Especially Difficult To Remove	Suppressor Inaccessible During Normal Operation	Suppressor in High Radiation Area During Shutdown*	
H 7A	Main Feedwater Line (03)		X		
H 6B			X		
2-1289	Emergency Feedwater Line (03A)				
2-5656					
2-5663					
2-5685					
2-5691					
H 1A				X	
H 3A			X		
H 1B	Reactor Coolant System (50)		X		
2-4100			X		
2-4105			X		
2-4107			X		
2-4109			X		
2-4111			X		
2-4112			X		
2-4113			X		
2-4114			X		
2-4115			X		
2-4117			X		
2-4119			X		
2-4120			X		
H 1				X	
H 3				X	
H 4			X		
H 5			X		
H 7			X		
H 8			X		
H 9			X		

4.18-7

Amendments 58, 58 & 55

TABLE 4.13-1
 Unit 2 Safety Related Shock Suppressors (Smubbers)

Sketch/Hanger No.	System	Suppressor Especially Difficult To Remove	Suppressor Inaccessible During Normal Operation	Suppressor in High Radiation Area During Shutdown*
H 10	Reactor Coolant System (50) (Continued)		X	
H 11			X	
H 12			X	
H 1A			X	
H 2A			X	
H 3A			X	
2-4482	High Pressure Injection System (51)			
H 2A			X	
H 1E			X	
2-2086	Low Pressure Injection (53)			
2-2089				
2-4206				
H 3			X	
H 1 C			X	
2-2139	Reactor Building Spray System (54)			
2-2149				
2-2172				
2-2174				
H 9A			X	
H 9B			X	
H 9	Spent Fuel Cooling (56)		X	
H 10			X	

4.13-8

Amendments 58, 58 & 55

TABLE 4.18-1

Unit 2 Safety Related Shock Suppressors (Snubbers)

<u>Sketch/Tagger No.</u>	<u>System</u>	<u>Suppressor Especially Difficult To Remove</u>	<u>Suppressor Inaccessible During Normal Operation</u>	<u>Suppressor in High Radiation Area During Shutdown*</u>
H 7	Pressurizer Relief Valve Discharge (57)		X	
H 9			X	
H 12			X	
H 13			X	
H 15			X	
H 16			X	
H 17			X	
H 20			X	
H 21			X	
H 23			X	
H 25			X	
H 26		X		

4.18-9

Amendments 58, 58 & 55

TABLE 4.18-1
Unit 3 Safety Related Shock Suppressors (Snubbers)

<u>Sketch/Hanger No.</u>	<u>System</u>	<u>Suppressor Especially Difficult To Remove</u>	<u>Suppressor Inaccessible During Normal Operation</u>	<u>Suppressor in High Radiation Area During Shutdown*</u>	
3-124	Main Steam Line (01A)		X		
3-125 (A&B)			X		
3-126					
3-128					
3-129					
3-130			X		
3-131				X	
3-132 (A, B, C, D)				X	
3-133					
3-135					
3-147 (A&B)					
3-149				X	
H 2A				X	
H 8A				X	
H 2B			X		
H 8B			X		
3-956	Main Steam Bypass to Condenser (01A-1)				
3-957					
3-959					
3-960					
3-3109	Main Steam Supply to Auxiliary Equipment (01A-3)				
3-1311	Main Steam Supply to Emergency Feedwater Pump Turbine (01A-4)				
3-1312					
3-1314					
3-1316					
3-1317					
3-1318					
3-1319					
3-1320					

4.18-10

Amendments 58, 58 & 55

TABLE 4.18-1
Unit 3 Safety Related Shock Suppressors (Snubbers)

Sketch/Hanger No.	System	Suppressor Especially Difficult To Remove	Suppressor Inaccessible During Normal Operation	Suppressor in High Radiation Area During Shutdown*
H 7A	Main Feedwater Line (03)		X	
H 6B			X	
H 40A			X	
H 4B			X	
3-1274	Emergency Feedwater Line (03A)			
3-1379				
3-1280				
3-5606				
3-5624				
3-5628				
H 1A	OTSG Recirculation System (04)		X	
H 11			X	
H 46			X	
H 50			X	
H 52			X	
3-4100	Reactor Coolant System (50)		X	
3-4105			X	
3-4107			X	
3-4109			X	
3-4111			X	
3-4112			X	
3-4113			X	
3-4114			X	
3-4115			X	
3-4117			X	
3-4119			X	
3-4120			X	
H 1			X	
H 3			X	

(continued)

4.18-11

Amendments 58, 58 & 55

TABLE 4.18-1
 Unit 3 Safety Related Shock Suppressors (Snubbers)

<u>Sketch/Hanger No.</u>	<u>System</u>	<u>Suppressor Especially Difficult To Remove</u>	<u>Suppressor Inaccessible During Normal Operation</u>	<u>Suppressor in High Radiation Area During Shutdown*</u>
Reactor Coolant System (50) (continued)				
H 4			X	
H 5			X	
H 7			X	
H 8			X	
H 9			X	
H 10			X	
H 11			X	
H 12			X	
H 1A			X	
H 2A			X	
H 3A			X	
H 13A			X	

4.18-11a

Amendments 58, 58 & 55

TABLE 4.18-1

Unit 3 Safety Related Shock Suppressors (Snubbers)

Sketch/Number No.	System	Suppressor Especially Difficult To Remove	Suppressor Inaccessible During Normal Operation	Suppressor in High Radiation Area During Shutdown*
3-2214 H 2A H 1E	High Pressure Injection System (51)		X X	
3-4271 3-4273 3-4280 3-4281 3-4282 3-4287 3-4288 H 3 H 1C	Low Pressure Injection System (53)		X X	
3-2140 3-2165 3-2174 H 9A H 9B	Reactor Building Spray System (54)		X X	
3-5700 3-5703 3-5707 3-5709 3-5712 3-5716 3-5718 H 9 H 10	Spent Fuel Cooling System (56)		X X	

4.18-12

Amendments 58, 58 & 55

TABLE 4.13-1

Unit 3 Safety Related Shock Suppressors (Saubbers)

<u>Sketch/Hanger No.</u>	<u>System</u>	<u>Suppressor Especially Difficult To Remove</u>	<u>Suppressor Inaccessible During Normal Operation</u>	<u>Suppressor in High Radiation Area During Shutdown*</u>
H 7	Pressurizer Relief Valve Discharge (57)		X	
H 9			X	
H 12			X	
H 13			X	
H 15			X	
H 16			X	
H 17			X	
H 20			X	
H 21			X	
H 23			X	
H 25			X	
H 26			X	

4.18-13

Amendments 58, 58 & 55.

*Modifications to this Table due to changes in high radiation areas should be submitted to the NRC as part of the next license amendment.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 58 TO FACILITY LICENSE NO. DPR-38

AMENDMENT NO. 58 TO FACILITY LICENSE NO. DPR-47

AMENDMENT NO. 55 TO FACILITY LICENSE NO. DPR-55

DUKE POWER COMPANY

OCONEE NUCLEAR STATION, UNIT NOS. 1, 2 AND 3

DOCKET NOS. 50-269, 50-270 AND 50-287

Introduction

By letter dated July 18, 1977, Duke Power Company (the licensee) requested changes to the Oconee Nuclear Station Units 1, 2 and 3 Technical Specifications which would add shock suppressors to Table 4.18-1 and change the classification of the location for some others.

Discussion/Evaluation

Hydraulic shock suppressors (snubbers) are required to ensure that the structural integrity of safety related systems is maintained during and following a seismic or other event resulting in large system dynamic loads. Table 4.18-1 of the common Oconee Technical Specifications provides a list of safety related snubbers installed at Oconee.

The licensee has proposed to add 22 safety-related snubbers to the Technical Specifications and to revise the status of 33 others. By status is meant classification into one of several categories defined as "accessible", "especially difficult to remove", "inaccessible during normal operation", and "high radiation area during shutdown". We conclude that the addition of snubbers subject to the requirements of the Technical Specifications is acceptable because it will result in greater surveillance of the snubbers at Oconee. Also, we find that the proposed changes in status are acceptable. Of the 33 snubbers, 25 were redesignated as "inaccessible during normal operation", 2 were redesignated "high radiation area during shutdown", 4 were redesignated "especially difficult to remove" and 2 were renumbered. These snubbers were redesignated after a re-examination of the snubbers at Oconee since this Technical Specification was issued October 13, 1976. This re-examination showed that the snubbers should more properly be redesignated as specified herein. These changes are consistent with the intent we had when we issued the original snubber Technical Specifications. The total number of 55 snubbers is for the three Oconee Units. Identical snubbers for the 3 Units were included as part of the total.

The licensee has also deleted no. H6A since it has been determined that this item is actually a pipe hanger and not a snubber.

Based on our review, we conclude that the above changes are acceptable.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: February 16, 1978

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-269, 50-270 AND 50-287DUKE POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 58 , 58 and 55 to Facility Operating License Nos. DPR-38, DPR-47 and DPR-55, respectively, issued to Duke Power Company which revised the Technical Specifications for operation of the Oconee Nuclear Station, Units Nos. 1, 2 and 3, located in Oconee County, South Carolina. The amendments are effective within 30 days after the date of issuance.

These amendments revise the Technical Specifications Table 4.18-1 by adding shock suppressors and changing the classification of the location of certain shock absorbers.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

- 2 -

For further details with respect to this action, see (1) the application for amendments dated July 18, 1977, (2) Amendment Nos. 58, 58 and 55 to License Nos. DPR-38, DPR-47 and DPR-55, respectively, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. and at the Oconee County Library, 201 South Spring Street, Walhalla, South Carolina 29691. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 16th day of February.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors