

January 4, 1995

DISTRIBUTION

Mr. J. W. Hampton
Vice President, Oconee Site
Duke Power Company
P. O. Box 1439
Seneca, SC 29679

~~Docket #~~
PUBLIC
PDII-3 RF
S.Varga
PA 0-17 F2
OC/LFDCB T-9 E10
B.Boger (A), RII
OGC 0-15 B18
D.Hag T-4 A43
G.Hill(4) T-5 C3
C.Grimes 0-11 F23
ACRS(4) T-2 E26
M.Sinkule, RII

SUBJECT: ISSUANCE OF AMENDMENTS - OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (TAC NOS.)

Dear Mr. Hampton:

The Nuclear Regulatory Commission has issued the enclosed Amendment Nos. 208, 208, and 205 to Facility Operating Licenses DPR-38, DPR-47, and DPR-55, respectively, for the Oconee Nuclear Station, Units 1, 2, and 3. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated

The amendments reflect the consolidation of the Quality Verification Department with the Nuclear Generation Department that realigned the Nuclear Safety Review Board to report to the Senior Nuclear Officer, change an organizational unit term from "group" to "division," modify titles of positions designated to approve modifications, clarify the responsibilities of the Safety Assurance Manager, and delete the requirement to perform an annual independent fire protection audit.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,
Original signed by:
Leonard A. Wiens, Senior Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270 and 50-287

Enclosures:

- 1. Amendment No. 208 to DPR-38
- 2. Amendment No. 208 to DPR-47
- 3. Amendment No. 205 to DPR-55
- 4. Safety Evaluation

cc w/encl: See next page

DOCUMENT NAME: G:\OCOONEE\OC088340.AMD

To receive a copy of this document, indicate in the box: "C" = Copy without attachment/enclosure "E" = Copy with attachment/enclosure "N" = No copy

OFFICE	LA:PD23:DRPE	E	PM:PD23:DRPE	E	OGC <i>AWI</i>	E	D:PD23:DRPE
NAME	CNorsworthy <i>cn</i>		LWiens:ddt <i>lan</i>		AWI <i>SCM</i>	FALLenspach <i>fra</i>	HBerko <i>hb</i>
DATE	11/30/94		12/1/94		12/1/94 <i>12/21/94</i>	12/5/94	12/1/94

OFFICIAL RECORD COPY

9501100252 950104
PDR ADOCK 05000269
P PDR

NRC FILE CENTER COPY

CP-1

DFOI



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

January 4, 1995

Mr. J. W. Hampton
Vice President, Oconee Site
Duke Power Company
P. O. Box 1439
Seneca, SC 29679

SUBJECT: ISSUANCE OF AMENDMENTS - OCONEE NUCLEAR STATION, UNITS 1, 2,
AND 3 (TAC NOS. M88340, M88341, AND M88342)

Dear Mr. Hampton:

The Nuclear Regulatory Commission has issued the enclosed Amendment Nos. 208, 208, and 205 to Facility Operating Licenses DPR-38, DPR-47, and DPR-55, respectively, for the Oconee Nuclear Station, Units 1, 2, and 3. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated November 11, 1993, as supplemented February 23, April 12, and July 29, 1994.

The amendments reflect the consolidation of the Quality Verification Department with the Nuclear Generation Department that realigned the Nuclear Safety Review Board to report to the Senior Nuclear Officer, change an organizational unit term from "group" to "division," modify titles of positions designated to approve modifications, clarify the responsibilities of the Safety Assurance Manager, and delete the requirement to perform an annual independent fire protection audit.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Leonard A. Wiens, Senior Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270 and 50-287

Enclosures:

1. Amendment No. 208 to DPR-38
2. Amendment No. 208 to DPR-47
3. Amendment No. 205 to DPR-55
4. Safety Evaluation

cc w/encl: See next page

Mr. J. W. Hampton
Duke Power Company

Oconee Nuclear Station

cc:

A. V. Carr, Esquire
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242-0001

Mr. Ed Burchfield
Compliance
Duke Power Company
Oconee Nuclear Site
P. O. Box 1439
Seneca, South Carolina 29679

J. Michael McGarry, III, Esquire
Winston and Strawn
1400 L Street, NW.
Washington, DC 20005

Ms. Karen E. Long
Assistant Attorney General
North Carolina Department of
Justice
P. O. Box 629
Raleigh, North Carolina 27602

Mr. Robert B. Borsum
Babcock & Wilcox
Nuclear Power Division
Suite 525
1700 Rockville Pike
Rockville, Maryland 20852

Mr. G. A. Copp
Licensing - EC050
Duke Power Company
526 South Church Street
Charlotte, North Carolina 28242-0001

Manager, LIS
NUS Corporation
2650 McCormick Drive, 3rd Floor
Clearwater, Florida 34619-1035

Dayne H. Brown, Director
Division of Radiation Protection
North Carolina Department of
Environment, Health and
Natural Resources
P. O. Box 27687
Raleigh, North Carolina 27611-7687

Senior Resident Inspector
U. S. Nuclear Regulatory Commission
Route 2, Box 610
Seneca, South Carolina 29678

Regional Administrator, Region II
U. S. Nuclear Regulatory Commission
101 Marietta Street, NW. Suite 2900
Atlanta, Georgia 30323

Max Batavia, Chief
Bureau of Radiological Health
South Carolina Department of Health
and Environmental Control
2600 Bull Street
Columbia, South Carolina 29201

County Supervisor of Oconee County
Walhalla, South Carolina 29621



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 208
License No. DPR-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 1 (the facility) Facility Operating License No. DPR-38 filed by the Duke Power Company (the licensee) dated November 11, 1993, as supplemented by letters dated February 23, April 12, and July 29, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 3.B of Facility Operating License No. DPR-38 is hereby amended to read as follows:

9501100262 950104
PDR ADOCK 05000269
P PDR

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 208, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: January 4, 1995



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

DOCKET NO. 50-270

OCONEE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 208
License No. DPR-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 2 (the facility) Facility Operating License No. DPR-47 filed by the Duke Power Company (the licensee) dated November 11, 1993, as supplemented by letters dated February 23, April 12, and July 29, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 3.B of Facility Operating License No. DPR-47 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 208, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: January 4, 1995



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

DOCKET NO. 50-287

OCONEE NUCLEAR STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 205
License No. DPR-55

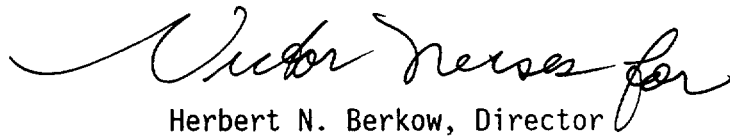
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 3 (the facility) Facility Operating License No. DPR-55 filed by the Duke Power Company (the licensee) dated November 11, 1993, as supplemented by letters dated February 23, April 12, and July 29, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 3.B of Facility Operating License No. DPR-55 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 205, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: January 4, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 208

FACILITY OPERATING LICENSE NO. DPR-38

DOCKET NO. 50-269

AND

TO LICENSE AMENDMENT NO. 208

FACILITY OPERATING LICENSE NO. DPR-47

DOCKET NO. 50-270

AND

TO LICENSE AMENDMENT NO. 205

FACILITY OPERATING LICENSE NO. DPR-55

DOCKET NO. 50-287

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove Pages

6.1-2
6.1-3
6.1-3a(1)
6.1-4
6.1-5
6.1-5a

Insert Pages

6.1-2
6.1-3
6.1-3a(1)
6.1-4
6.1-5
6.1-5a

6.1.2 Technical Review 1 Control

6.1.2.1 Activities

- a. Procedures required by Technical Specification 6.4 and other procedures which affect station nuclear safety, and changes (other than editorial or typographical changes) thereto, shall be prepared by a qualified individual/organization. Each such procedure, or procedure change, shall be reviewed by an individual/group other than the individual/group which prepared the procedure, or procedure change, but who may be from the same organization as the individual/group which prepared the procedure, or procedure change. Such procedures and procedure changes may be approved for temporary use by two members of the station staff, at least one of whom holds a Senior Reactor Operator's License on the unit(s) affected. Procedures and procedure changes shall be approved prior to use or within seven days of receiving temporary approval by an appropriate division manager, superintendent/manager, or one of their designated direct reports.
- b. Proposed changes to the Technical Specifications shall be prepared by a qualified individual/organization. The preparation of each proposed Technical Specifications change shall be reviewed by an individual/group other than the individual/group which prepared the proposed change, but who may be from the same organization as the individual/group which prepared the proposed change. Proposed changes to the Technical Specifications shall be approved by the Station Manager.
- c. Proposed modifications to station nuclear safety-related structures, systems and components shall be designed by a qualified individual/ organization. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modification. Proposed modifications to nuclear safety-related structures, systems, and components shall be approved prior to implementation by the Station Manager or the Manager of Engineering; or for the Station Manager by the Mechanical Maintenance Superintendent, the Operations Superintendent, the I and E/Maintenance Support Superintendent, or the Work Control Superintendent, as previously designated by the Station Manager.
- d. Individuals responsible for reviews performed in accordance with 6.1.2.1.a, 6.1.2.1.b, and 6.1.2.1.c shall be members of the Site supervisory staff, previously designated by the Site Vice President to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated Site review personnel.
- e. Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be reviewed by the Station Manager; or for the Station Manager by the Mechanical Maintenance Superintendent, the Operations Superintendent, the I and E/Maintenance Support Superintendent, or the Work Control Superintendent, as previously designated by the Station Manager.
- f. Incidents reportable pursuant to Technical Specification 6.6.2.1 and violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. The Manager, Safety Assurance shall assure such reports are developed and transmitted to the Site Vice President, or designee, who approves such reports, and to the Director of the Nuclear Safety Review Board.
- g. The Manager, Safety Assurance shall assure the performance of special reviews and investigations, and the preparation and submittal of reports thereon, as requested by the Site Vice President.

- h. Deleted
- i. Deleted
- j. The Manager, Safety Assurance shall assure that an independent fire protection and loss prevention inspection and audit shall be performed utilizing qualified off-site personnel and that an inspection and audit by a qualified fire consultant shall be performed at intervals no greater than three years.
- k. Unplanned onsite releases of radioactive material to the environs shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. The Manager, Safety Assurance shall assure such reports are developed and transmitted to the Site Vice President, or designee, and to the Director of the Nuclear Safety Review Board.
- l. Licensee-initiated changes to the Offsite Dose Calculation Manual (ODCM) shall be documented and records of reviews performed shall be retained for the duration of the unit operating license. This documentation shall contain:
 - 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s); and,
 - 2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations; and,

- 3) Shall become effective upon review and acceptance by the Station Manager and a qualified individual/organization; and,
 - 4) Shall be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Annual Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.
- m. Licensee-initiated changes to the Process Control Program shall be documented and records of reviews performed shall be retained for the duration of the unit license. This documentation shall contain:
- 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s);
 - 2) A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations; and,
 - 3) Changes shall become effective upon review and acceptance by the Station Manager and a qualified individual/organization.

6.1.2.2 Records

Records of the above activities shall be maintained.

6.1.3 Nuclear Safety Review Board

6.1.3.1 Function

The NSRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations
- b. Nuclear Engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Instrumentation and control
- f. Radiological safety
- g. Mechanical and electrical engineering
- h. Administrative control and quality assurance practices

6.1.3.2 Organization

- a. The Director, members and alternate members of the NSRB shall be formally appointed by the Senior Vice President, Nuclear Generation, and shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of five years technical experience, of which a minimum of three years shall be in one or more areas given in 6.1.3.1.

In special cases, candidates for appointment without an academic degree in engineering or a physical science may be qualified with a minimum of ten years experience in one of the areas specified in Specification 6.1.3.1.

6.1.3.3 Subjects Requiring Review

The following subjects shall be reported to and reviewed by the NSRB:

- a. The safety evaluations for (1) changes to procedures, equipment or systems, and (2) tests or experiments completed under the provisions of 10 CFR 50.59(a)(1) to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in 10 CFR 50.59.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59.
- d. Proposed changes in Technical Specifications or the Facility Operating Licenses.
- e. Violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of station equipment that affect nuclear safety.
- g. Incidents that are the subject of non-routine reports submitted to the Commission.
- h. Quality Assurance Program audits relating to station operations and actions taken in response to these audits.
- i. Proposed changes to Fire Protection Program

6.1.3.4 Audits

Audits of Site activities shall be performed under the cognizance of the NSRB. These audits shall encompass:

- a. The conformance of station operation to provisions contained within the Technical Specifications and applicable facility operating license conditions.
- b. The performance, training and qualifications of the station staff.
- c. The results of actions taken to correct deficiencies occurring in equipment, structures, systems or methods of operation that affect nuclear safety.
- d. The performance of activities required by the quality assurance program to meet the criteria of Appendix B to 10 CFR 50.
- e. The station emergency plan and implementing procedures.
- f. The station security plan and implementing procedures.
- g. Any other area of station operation considered appropriate by the NSRB or the Senior Vice President, Nuclear Generation.
- h. The station fire protection program and implementing procedures.
- i. The Offsite Dose Calculation Manual and implementing procedures.
- j. The Radiological Environmental Monitoring Program and the results thereof .
- k. The Process Control Program and implementing procedures for solidification of radioactive wastes.
- l. The performance of activities required by the Quality Assurance Program to meet the criteria of Regulatory Guide 1.21 Revision 1, June 1974 and Regulatory Guide 4.1 Revision 1, April 1975.

6.1.3.5 Responsibilities and Authorities

- a. The NSRB shall report to and advise the Senior Vice President, Nuclear Generation on those areas of responsibility specified in Specifications 6.1.3.3 and 6.1.3.4.
- b. Minutes shall be prepared and forwarded to the Senior Vice President, Nuclear Generation, and to the Site Vice President within 14 days following each formal meeting of the NSRB.
- c. Records of activities performed in accordance with Specifications 6.1.3.3 and 6.1.3.4 shall be maintained.
- d. Audit reports encompassed by Section 6.1.3.4 shall be forwarded to the Senior Vice President, Nuclear Generation and to the Site Vice President and to the management position responsible for the areas audited within 30 days of completion of each audit.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 208 TO FACILITY OPERATING LICENSE NPF-38

AMENDMENT NO. 208 TO FACILITY OPERATING LICENSE 47
AND AMENDMENT NO. 205 TO FACILITY OPERATING LICENSE NPF-55

DUKE POWER COMPANY

OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3

DOCKET NOS. 50-269, 50-270, AND 50-287

1.0 INTRODUCTION

By letter dated November 11, 1993, as supplemented February 23, April 12 and July 29, 1994, Duke Power Company (the licensee) submitted a request for changes to the Oconee Nuclear Station, Units 1, 2, and 3, Technical Specifications (TS). The amendments reflect the consolidation of the Quality Verification Department with the Nuclear Generation Department that realigned the Nuclear Safety Review Board (NSRB) to report to the Senior Nuclear Officer, change an organizational unit term from "group" to "division," modify titles of positions designated to approve modifications, clarify the responsibilities of the Safety Assurance Manager, and delete the requirement to perform an annual independent Fire Protection Audit.

The February 23, April 12 and July 29, 1994, letters provided clarifying information that did not change the scope of the November 11, 1993, application or the initial proposed no significant hazards consideration determination.

2.0 EVALUATION

CONSOLIDATION OF DEPARTMENTS

The consolidation of the Quality Verification Department with the Nuclear Generation Department is reflected on the subject TS pages in TS 6.1.3.3 wherein Quality Verification Department audits are renamed Quality Assurance Program audits. The audit group is maintained as before with the exception that it is now moved into the Safety Assurance Group. The specification of the list of audits in TS 6.1.3.1 and the requirement that the NSRB review these audits are not changed by this amendment. Since the envelope of the audits to be performed and the NSRB review requirement are unchanged, this change is acceptable.

REALIGNMENT OF NSRB REPORTING RELATIONSHIP - TS 6.1.3.2, 6.1.3.4, 6.1.3.5

The licensee requested changes in an earlier application and the staff approved TS changes in amendments numbered 193, 193, and 190 for Units 1, 2 and 3, respectively, on May 1, 1992, that elevated certain TS-listed reporting relationships to a licensee management level that is generally one level higher than the level addressed by NRC staff guidance, including the Standard Technical Specifications (STS). Included in these changes was the elevation of the NSRB reporting level to the licensee's Executive Vice President. The licensee requests, in the current amendment application, to return this NSRB reporting relationship and associated responsibilities to the Senior Vice President, Nuclear Generation. This change is reflected in TS 6.1.3.2.a, 6.1.3.4.g, and 6.1.3.5. The Senior Vice President, Nuclear Generation, is the licensee's Senior Nuclear Officer and the staff concludes that this proposal is consistent with general staff guidance in the Standard Review Plan and the STS and is acceptable. For purpose of consistency, the title Senior Vice President Nuclear Generation Department was changed to Senior Vice President, Nuclear Generation in TS 6.1.3.4.g. The licensee agreed with this change.

RECORDS OF NSRB ACTIVITIES - TS 6.1.3.5

Consistent with the change in the NSRB reporting relationship discussed above, the Executive Vice President is removed from the list for receipt of NSRB meeting minutes, reports and audit reports, and the Site Vice President is added to this list. This change provides comparable management levels as contained in the STS and is acceptable.

TECHNICAL REVIEW AND CONTROL - TS 6.1.2.1.c, 6.1.2.1.e

The licensee initially proposed to modify the authority for approval of modifications by deleting specifically named management positions and substituting a generalized delegation of authority by the Station Manager. This was subsequently revised in the submittal of July 29, 1994, to a more restrictive proposal to augment the Station Manager's authority for approving modifications by adding the Manager of Engineering and by revising the position titles in these TS to reflect the titles of these positions in the current licensee organization. The title of Mechanical Superintendent is revised to the Mechanical Maintenance Superintendent and the I and E Superintendent is revised to the I and E/Maintenance Support Superintendent. The staff finds these changes to be acceptable as they provide for approval by the appropriate division manager or superintendent level.

SAFETY ASSURANCE MANAGER RESPONSIBILITIES - TS 6.1.2.1.f, 6.1.2.1.k

The Safety Assurance Manager's responsibilities are clarified to those of ensuring the development of TS 6.6.2.1 reports and to provide for a designee who may approve these reports from the prior requirement that the Safety Assurance Manager approve all such reports. The staff finds this change acceptable, as the reports are approved and sent to appropriate management levels.

PROCEDURE APPROVAL AND CHANGE AUTHORITY - TS 6.1.2.1.a

The licensee proposed to change the approval authority from "group" manager to "appropriate division" manager. This change reflects the retitling of comparable management positions due to the reorganization. Either the former or the current title is consistent with the STS guidance for the appropriate level of management responsibility for this function and is acceptable.

ANNUAL INDEPENDENT FIRE PROTECTION AUDIT - TS 6.1.2.1.j

The requirement to perform an "annual" independent fire protection audit, formerly stated in TS 6.1.3.4, was deleted in Amendments 193, 193 and 190 for Units 1, 2 and 3, respectively, issued on May 1, 1992. That amendment retained the requirement to perform such audits, but relocated the audit frequency requirements to the Duke Quality Assurance Topical Report, a document controlled in accordance with 10 CFR 50.54(a). The licensee now seeks to correct an administrative error whereby the word "annual" inadvertently was not deleted from TS 6.1.2.1.j at that time. This constitutes an administrative change, to make the requirements of this TS consistent with the previous TS amendments. The NRC staff finds this change is administrative in nature and is acceptable.

3.0 OVERALL SUMMARY

The staff finds the licensee's proposed changes acceptable, as they meet the appropriate acceptance criteria of Section 13.4 of NUREG-0800, the Standard Review Plan, and are consistent with the Standard Technical Specifications.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the South Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The amendments also relate to changes in recordkeeping, reporting, or administrative procedures or requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: F. Allenspach
L. Wiens

Date: January 4, 1995