

February 7, 1996

Mr. J. W. Hampton
Vice President, Oconee Site
Duke Power Company
P. O. Box 1439
Seneca, SC 29679

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SUBJECT: ISSUANCE OF AMENDMENTS - OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (TAC NOS. M94617, M94618, M94619)

Dear Mr. Hampton:

The Nuclear Regulatory Commission has issued the enclosed Amendment Nos. 214, 214, and 211 to Facility Operating Licenses DPR-38, DPR-47, and DPR-55, respectively, for the Oconee Nuclear Station, Units 1, 2, and 3. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated February 6, 1996.

The amendments revise TS Section 3.16, "Containment Hydrogen Control Systems." The change adds a footnote to TS 3.16.3.b. to allow a one-time outage duration extension in regard to the Containment Hydrogen Control System flow path. This extension is necessary to install and test plant modifications, which will allow the Containment Hydrogen Control System to perform as designed, without the potential for inoperability due to water accumulation in the flow path.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Leonard A. Wiens, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270 and 50-287

- Enclosures: 1. Amendment No. 214 to DPR-38
- 2. Amendment No. 214 to DPR-47
- 3. Amendment No. 211 to DPR-55
- 4. Safety Evaluation

cc w/encl: See next page
DOCUMENT NAME: G:\OCONEE\OCOEMERG.AMD

CR1

OFFICE	PDII-2/MA	PDII-2/PM	SCSB	OGC	PDII-2/D
NAME	L. BERRY	L. WIENS	C. BERLINGER	C. MUMFORD	H. BERKOW
DATE	2/7/96	2/7/96	2/7/96	2/7/96	2/7/96
COPY	YES NO	YES NO	YES NO	YES NO	YES NO

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 7, 1996

Mr. J. W. Hampton
Vice President, Oconee Site
Duke Power Company
P. O. Box 1439
Seneca, SC 29679

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Sincerely,

A handwritten signature in black ink, appearing to read "A. Wiens".

Leonard A. Wiens, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270 and 50-287

Enclosures:

1. Amendment No. 214 to DPR-38
2. Amendment No. 214 to DPR-47
3. Amendment No. 211 to DPR-55
4. Safety Evaluation

cc w/encl: See next page

Mr. J. W. Hampton
Duke Power Company

Oconee Nuclear Station

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 214
License No. DPR-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 1 (the facility) Facility Operating License No. DPR-38 filed by the Duke Power Company (the licensee) dated February 6, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 3.B of Facility Operating License No. DPR-38 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 214, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION.



for

Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: February 7, 1996



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

DOCKET NO. 50-270

OCONEE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 214
License No. DPR-47

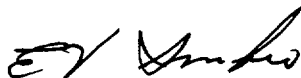
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 2 (the facility) Facility Operating License No. DPR-47 filed by the Duke Power Company (the licensee) dated February 6, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 3.B of Facility Operating License No. DPR-47 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 214, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: February 7, 1996



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

DOCKET NO. 50-287

OCONEE NUCLEAR STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 211
License No. DPR-55

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 3 (the facility) Facility Operating License No. DPR-55 filed by the Duke Power Company (the licensee) dated February 6, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 3.B of Facility Operating License No. DPR-55 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 211, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



for

Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: February 7, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 214

FACILITY OPERATING LICENSE NO. DPR-38

DOCKET NO. 50-269

AND

TO LICENSE AMENDMENT NO. 214

FACILITY OPERATING LICENSE NO. DPR-47

DOCKET NO. 50-270

AND

TO LICENSE AMENDMENT NO. 211

FACILITY OPERATING LICENSE NO. DPR-55

DOCKET NO. 50-287

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove Pages

3.16-1
3.16-2

Insert Pages

3.16-1
3.16-2

3.16.3 Components in the Containment Hydrogen Control Systems' flow path shall be operable on each Oconee unit with the following exceptions.

- a. If the flow path is inoperable it shall be restored to operable status within 7 days.
- b. If an inoperable flow path is not restored to operable status within 7 days, then the affected unit shall be at hot shutdown within the next 12 hours and at cold shutdown within an additional 24 hours. ⁽¹⁾

Bases

The Containment Hydrogen Control Systems are required at approximately 460 hours (19.2 days) following a LOCA to limit hydrogen concentration to 4.1 percent by volume.

The Containment Hydrogen Recombiner System is utilized as the primary method to maintain the post-accident containment atmosphere hydrogen concentration below its lower flammability limit of 4.1 percent by volume. The Containment Hydrogen Recombiner System includes a portable hydrogen recombiner which will be moved to the affected unit following a LOCA, anchored to its foundation, and connected to piping penetrations. Also included is a portable control panel, which will be locally mounted near the recombiner, anchored to its foundation and connected to its motor control center and the recombiner.

The Reactor Building Hydrogen Purge System is composed of a portable purging station and a portion of the Penetration Room Ventilation System. The purge system is operated as necessary (if the Containment Hydrogen Recombiner System is inoperable) to maintain the hydrogen concentration below the control limit.

The Containment Hydrogen Recombiner System is the preferable method of post-accident hydrogen control since it produces no radioactive gaseous release to the atmosphere. Therefore, the Containment Hydrogen Recombiner System will be utilized as the primary method to control the containment hydrogen concentration below 4.1 percent by volume.

Reference

FSAR, Section 15.16

Oconee 1, 2 and 3

3.16-2

Amendment No. 214 (Unit 1)
Amendment No. 214 (Unit 2)
Amendment No. 211 (Unit 3)

- (1) A one-time allowable outage time of fourteen (14) days is granted for installation of a modification to provide drainage for moisture which may accumulate in the Containment Hydrogen Recombiner System piping.

3.16 CONTAINMENT HYDROGEN CONTROL SYSTEMS

Applicability

Applies to the Containment Hydrogen Recombiner System (including portable hydrogen recombiner unit and the Hydrogen Recombiner System flow path) and the Reactor Building Hydrogen Purge System whenever any Oconee unit is above cold shutdown conditions.

Objective

To define the conditions necessary to assure the availability of adequate containment hydrogen control capability.

Specification

3.16.1 The Containment Hydrogen Control Systems shall be operable as follows:

- a. A portable hydrogen recombiner unit shall be operable and available for connection to the affected unit.
- b. If no portable hydrogen recombiner unit is operable and available for connection to the affected unit, a portable hydrogen recombiner unit shall be restored to an operable status within 7 days.
- c. If the conditions in 3.16.1.b can not be satisfied, the Reactor Building Hydrogen Purge System shall be verified operable within the next 48 hours.
- d. With the Reactor Building Hydrogen Purge System operable, restore a hydrogen recombiner unit to operating status within 30 days or submit a report to the NRC within the next 30 days describing the circumstances resulting in inoperable equipment and plans for returning the equipment to service and for any interim surveillance testing of the purge system.
- e. With a portable hydrogen recombiner unit operable, the Reactor Building Hydrogen Purge System is not required to be operable.

3.16.2 If the conditions under Technical Specification 3.16.1.c are not met, the Oconee Units shall be in hot shutdown within the next 12 hours and in cold shutdown in an additional 24 hours.

Oconee 1, 2 and 3

3.16-1

Amendment No. 214 (Unit 1)
Amendment No. 214 (Unit 2)
Amendment No. 211 (Unit 3)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 214 TO FACILITY OPERATING LICENSE DPR-38
AMENDMENT NO. 214 TO FACILITY OPERATING LICENSE DPR-47
AND AMENDMENT NO. 211 TO FACILITY OPERATING LICENSE DPR-55

DUKE POWER COMPANY

OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3

DOCKET NOS. 50-269, 50-270, AND 50-287

1.0 INTRODUCTION

By letter dated February 6, 1996, Duke Power Company (the licensee), submitted a request for changes to the Oconee Nuclear Station, Units 1, 2, and 3, Technical Specifications (TS). The amendments revise TS Section 3.16, "Containment Hydrogen Control Systems." The change adds a footnote to TS 3.16.3.b. to allow a one-time outage duration extension in regard to the Containment Hydrogen Control System flow path. This extension is necessary to install and test plant modifications, which will allow the Containment Hydrogen Control System to perform as designed, without the potential for inoperability due to water accumulation in the flow path.

2.0 BACKGROUND

Oconee Nuclear Station consists of three 2568 Mwt pressurized water reactors located in Seneca, South Carolina and operated by Duke Power Company (the licensee).

The Containment Hydrogen Recombiner System (CHRS) is the primary method for controlling post-accident hydrogen generation at Oconee Nuclear Station. The Reactor Building Hydrogen Purge System (RBHPS) is intended as a backup to the CHRS. However, the licensee has stated in conversations with the staff that the RBHPS would not be immediately available as a backup. The CHRS consists of a portable hydrogen recombinder, control panel for the recombinder, and a portion of the Penetration Room Ventilation System (PRVS). When needed, the recombinder is moved to the affected unit, anchored to a foundation designated for this purpose, and connected to the PRVS piping which runs to and from containment penetrations.

On February 1, 1996, at 1:30 p.m., Oconee Nuclear Station declared the CHRS and RBHPS inoperable due to the potential condensation of steam from the containment atmosphere following a loss-of-coolant accident (LOCA) in the piping to and from the CHRS or RBHPS. If the affected flow paths are not restored to an operable status within 7 days, Technical Specification 3.16.3.b requires that the affected unit shall be in hot shutdown within the next

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12 hours and at cold shutdown within an additional 24 hours. The condensation phenomenon is common to all three Oconee units. The NRC was notified of this condition via a 10 CFR 50.72 notification at approximately 4:30 p.m. on February 1, 1996.

The flow path from containment to the hydrogen recombiner consists of approximately 250 feet of three inch diameter stainless steel piping. Similarly, the return flow path from the recombiner to containment is also approximately 250 feet in length. The routed piping contains several low points. Following an accident, the gas mixture flowing through this piping will include saturated steam. Recent calculations by the licensee determined that steam condensation will create loop seals in the low points of the piping that will interrupt flow to the hydrogen recombiner. These loop seals may inhibit the ability of the recombiner to reduce the post-accident hydrogen concentration.

The licensee is considering a long-term solution to this problem. Because this long term solution will take some time to implement, the licensee proposed an interim solution which satisfies the Technical Specifications and the intent of the design basis of the system, and which could be implemented under the criteria of 10 CFR 50.59. The licensee has estimated that the interim design will then be in place for approximately 60 days before the long-term design modification is operable. However, the licensee has determined that more time may be required to complete installation and testing of this interim solution than is permitted by the current Technical Specifications allowed outage time for the CHRS. The licensee has therefore requested, by letter dated February 6, 1996, changes to the Oconee Nuclear Station Technical Specifications to permit a one-time extension of the 7-day allowable outage time for the CHRS to 14 days.

Until the completion of ongoing installation and testing of the interim modification, the CHRS will continue to be inoperable. Even though the work is being performed on an expedited basis, the licensee will not be able to complete the modification within the allowed time frame of 7 days.

3.0 EVALUATION

The staff finds the licensee's proposed change to the Technical Specifications to increase the allowed outage time for the CHRS to be acceptable for the following reasons.

First, the probability of a loss-of-coolant accident occurring during the additional time CHRS is inoperable while implementing the interim design is low. Within 14 days (7 days in addition to that permitted by the existing LCO allowed outage time) the licensee intends to have in place an interim design which will maintain the OPERABILITY of the CHRS.

The increase in risk due to an inoperable CHRS and RBHPS for 14 days is not significant. Neither the CHRS nor the RBHPS performs a required core cooling function; therefore, there will be no increase in the core damage frequency. Furthermore, structural integrity of the Oconee containment would be maintained, even with concentrations of hydrogen that would be present should

no hydrogen be removed for 30 days following a LOCA, as discussed in the licensee's February 6, 1996, submittal.

Therefore, the staff finds the licensee's proposed increase in the allowed outage time of the CHRS to a total of 14 days to be acceptable.

4.0 STATEMENT OF EMERGENCY CIRCUMSTANCES

The licensee's application for the TS change has been timely. An engineering review of the Containment Hydrogen Control System (CHCS) identified a condition where condensation may accumulate in the flow path to the hydrogen recombiner, potentially rendering the system inoperable. This potential inoperability was identified on February 1, 1996, and had not been previously known. The NRC was promptly notified and the licensee aggressively pursued the development and implementation of corrective actions to restore the system to operable status. The current allowable outage time for the CHCS is seven (7) days. Although the licensee has expedited the development and installation of a temporary modification to restore the system to operable status, they are unable to complete the installation and testing of the modification within the allowable 7 days.

The NRC staff finds that failure to grant the proposed changes in a timely manner would result in shutdown of Oconee, Units 1, 2, and 3. The staff also finds that the licensee has responded in a timely manner, and has not delayed its application to take advantage of the Emergency License Amendment provisions of the 10 CFR 50.91. Accordingly, the staff concludes that the licensee has satisfied the requirements of 10 CFR 50.91(a)(5), and that a valid emergency exists.

5.0 FINAL NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

The Commission's regulations in 10 CFR 50.92 state that the Commission may make a final determination that a license amendment involves no significant hazards considerations, if operation of the facility, in accordance with the amendment would not (1) involve a significant increase in the probability or consequences of any accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

The amendments have been evaluated against the three standards in 10 CFR 50.92(c). In its analysis of the issue of no significant hazards consideration, as required by 10 CFR 50.91(a), the licensee has provided the following:

- (1) Involve a significant increase in the probability or consequences of an accident previously evaluated:

No. The Containment Hydrogen Control System is not an accident initiator. The Containment Hydrogen Control System functions following a loss of coolant accident (LOCA) to control combustible gas concentration accumulating inside containment.

This one-time, seven day extension of the allowable outage time for the Containment Hydrogen Control System flow path does not increase the probability of a LOCA or any other previously evaluated accident.

The hydrogen generation which occurs following a design basis LOCA is a slow process driven by sump radiolysis and metal corrosion. Calculations have shown that many days are required to reach the regulatory limit of 4 v/o. A hydrogen concentration slightly above 4 v/o is generally accepted as the lower flammability limit. Furthermore, assuming no credit for the CHRS or RBHPS, the concentration thirty days following a design basis LOCA, is approximately 5.5 v/o. Studies of containment structural capacity and the effects of hydrogen combustion have shown that concentrations much higher than 4 v/o are required to threaten the integrity of a large dry containment like the Oconee containment. Concentrations in excess of 12 v/o would be required to present a challenge to the integrity of the Oconee containment. Concentrations of this magnitude are only expected during core damage accidents like those studied in probabilistic risk analyses. Therefore, this proposed change does not increase the consequences of an accident previously evaluated.

- (2) Create the possibility of a new or different kind of accident from any kind of accident previously evaluated:

No. As stated previously, the Containment Hydrogen Control System is not an accident initiator. Rather it provides mitigation functions post-accident.

Increasing the allowable outage extension time does not introduce any new credible failure modes. Therefore, this proposed change does not create the possibility of a new or different kind of accident from any kind of accident previously evaluated.

- (3) Involve a significant reduction in a margin of safety:

No. The margin of safety is not significantly reduced by a one-time fourteen (14) day allowable outage time for the Containment Hydrogen Control System flow path. As stated above, hydrogen generation following a design basis LOCA is a slow process which will not reach concentrations which would be required to threaten containment integrity within thirty days. For large, dry containments, the fact that hydrogen generation following a design basis event is not a significant safety concern has been recognized for several years. In 1993, the NRC hosted a workshop on the elimination of requirements marginal to safety. One of the items discussed was the requirements for hydrogen recombiners. Since it has been recognized that the

requirements concerning the hydrogen recombiner are incidental to safety, this one-time increase in the allowed outage time does not represent a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, finds that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff has determined that the amendment request involves no significant hazards consideration.

6.0 STATE CONSULTATION

Based upon the written notice of the proposed amendments, the South Carolina State official had no comments.

7.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments. The Commission has made a final no significant hazards consideration finding with respect to the amendments.

8.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Lobel

Date: February 7, 1996