Mr. W. R. McCollum, Jr. Vice President, Oconee Site Duke Energy Corporation P. O. Box 1439 Seneca, SC 29679

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 RE: TECHNICAL

SPECIFICATION BASES CHANGE

Dear Mr. McCollum:

By letter dated May 19, 1999, you informed the staff of a change to the Oconee Nuclear Station, Units 1, 2, and 3 Improved Technical Specifications (ITS) Bases Section 3.4.13. The purpose of the change is to incorporate a statement to clarify that a loss of reactor coolant through reactor coolant pump seals and system valves to connecting systems that vent to the gas vent header, from which coolant can be returned to the reactor coolant system, is not considered to be "Reactor Coolant System Leakage." The provision was contained in the Oconee Technical Specifications prior to conversion to the ITS.

The purpose of this letter is to distribute the attached revised TS pages to the appropriate TS manual holders.

Sincerely,

Original signed by:

David E. LaBarge, Senior Project Manager, Section 1 Project Directorate II Division of Licensing Project Management Office of Nuclear Reactor Regulation

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Docket Nos. 50-269, 50-270, and 50-287

Enclosure: Revised Bases Page

cc w/encl: See next page

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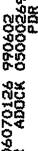
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UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

June 2, 1999

Mr. W. R. McCollum, Jr. Vice President, Oconee Site Duke Energy Corporation P. O. Box 1439 Seneca, SC 29679

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David E. LaBarge, Senior Project Manager, Section 1

Project Directorate II

Division of Licensing Project Management

Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270, and 50-287

Enclosure: Revised Bases Page

Oconee Nuclear Station

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BASES

APPLICABLE SAFETY ANALYSES (continued)

Primary to secondary LEAKAGE is a factor in the dose releases outside containment resulting from a SLB accident. To a lesser extent, other accidents or transients involve secondary steam release to the atmosphere, such as a steam generator tube rupture (SGTR). The leakage contaminates the secondary fluid and can be released to the environment.

The safety analysis assumptions for the SLB accident bounds 300 gallon per day primary to secondary LEAKAGE in one generator as an initial condition. The dose consequences resulting from the SLB accident are within the limits defined in 10 CFR 100.

RCS operational LEAKAGE satisfies Criterion 2 of 10 CFR 50.36 (Ref.3).

| LCO

RCS LEAKAGE includes leakage from connected systems up to and including the second normally closed valve for systems which do not penetrate containment and the outermost isolation valve for systems which penetrate containment. Loss of reactor coolant through reactor coolant pump seals and system valves to connecting systems which vent to the gas vent header and from which coolant can be returned to the RCS shall not be considered as RCS LEAKAGE.

RCS operational LEAKAGE shall be limited to:

a. <u>Pressure Boundary LEAKAGE</u>

No pressure boundary LEAKAGE is allowed, being indicative of material deterioration. LEAKAGE of this type is unacceptable as the leak itself could cause further deterioration, resulting in higher LEAKAGE. Violation of this LCO could result in continued degradation of the RCPB. LEAKAGE past seals, gaskets, and steam generator tubes is not pressure boundary LEAKAGE.

(continued)