



Entergy Nuclear Northeast
Entergy Nuclear Operations, Inc.
440 Hamilton Avenue
P. O. Box 5029
White Plains, NY 10601-5029
Tel 914 272 3500

July 16, 2001
IPN-01-053

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Stop O-P1-17
Washington, DC 20555-0001

SUBJECT: Indian Point 3 Nuclear Generating Unit No. 3
Docket No. 50-286
Second 10-Year ISI Interval Closeout and Associated Relief Requests

REFERENCE: 1. Safety Evaluation of Second Ten-Year Interval Inservice Inspection Plan and Associated Relief Requests for Indian Point Nuclear Generating Unit No. 3 (TAC NO. 72247), dated November 7, 1991
2. Evaluation of the Second 10-Year Interval Inservice Inspection Plan and Associated Requests for Relief - Indian Point Nuclear Generating Unit No. 3 (TAC NO. M82269), dated December 21, 1994

Dear Sir:

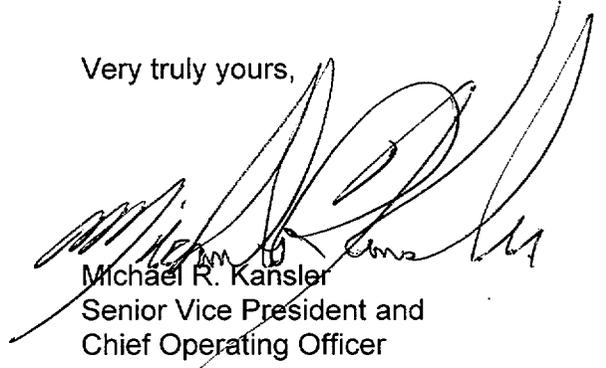
This letter submits the Second 10-Year Inservice Inspection (ISI) Closeout and Associated Relief Requests (Attachment I) for Indian Point Generating Unit No. 3. The Second 10-Year ISI Interval Program was based on Section XI, 1983 Edition with the Summer 1983 Addenda of the ASME Boiler and Pressure Vessel Code, as modified for relief requests approved by the NRC. This Interval ended on July 20, 2000.

Six (6) relief requests (RR 2-Closeout-1 through RR 2-Closeout-6) for the 2nd 10-Year ISI Interval Closeout are included in Attachment I. These relief requests apply to selected welds/components where code-required examination could not be performed, or the code-required volume/area could not be achieved.

A047

There are no new commitments made in this letter. If you have any questions, please contact Ms. Charlene Faison at 914-272-3378.

Very truly yours,

A handwritten signature in black ink, appearing to read "Michael R. Kansler", is written over the typed name and title. The signature is fluid and cursive, with a large, sweeping flourish at the end.

Michael R. Kansler
Senior Vice President and
Chief Operating Officer

cc: Next page.

cc: Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Resident Inspector's Office
Indian Point Unit 3
U.S. Nuclear Regulatory Commission
P.O. Box 337
Buchanan, NY 10511

Mr. Patrick Milano, Project Manager
Project Directorate I
Division of Licensing Project Management
U.S. Nuclear Regulatory Commission
Mail Stop 8E20
Washington, DC 20555

List of Attachments:

- I. Indian Point Nuclear Generating Unit No. 3, Second 10-Year Inservice Inspection Interval Closeout and Associated Relief Requests, Rev. 0

ATTACHMENT I TO IPN-01-053

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

**Second 10-YEAR Inservice Inspection Interval Closeout
and Associated Relief Requests, Rev. 0**

ENTERGY NUCLEAR OPERATIONS, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 3
DOCKET NO. 50-286
DPR-64



Entergy

Nuclear Northeast

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

**Second 10-Year Inservice Inspection Interval Closeout
and Associated Relief Requests**

Rev. 0

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SUMMARY LISTING

Introduction

Class 1 Components

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| 3 | Code Category B-D | Full Penetration Welds of Nozzles in Vessels-Inspection Program B |
| 4 | Code Category B-F | Pressure Retaining Dissimilar Metal Welds |
| 5 | Code Category B-J | Pressure Retaining Welds in Piping |
| 6 | Code Category B-K-1 | Integral Attachments for Piping, Pumps, and Valves |

Class 2 Components

- | | | |
|---|-------------------|--|
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ENCLOSURE SUMMARY

ENCLOSURE 1	B-A, B1.12	6 Pages
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INTRODUCTION

The 2nd Ten Year Inservice Inspection Interval, August 30, 1986 through July 20, 2000, was completed and fulfilled the requirements of ASME Boiler and Pressure Vessel Code, 1983 Edition including the 1983 Summer Addenda, Code of Federal Regulations 10CFR50.55a, United States Nuclear Regulatory Commission Guidelines, Indian Point No. 3 FSAR, Indian Point Unit No.3 Technical Specifications and United States Nuclear Regulatory Commission SER dated December 21, 1994.

Indian Point Unit 3 was designed prior to the implementation of ASME Boiler and Pressure Codes resulting in unforeseen difficulties during the 2nd Interval that determined conformance with certain code requirements to be impractical for this facility. Throughout the 2nd Interval, changes in technology were employed to adhere to the necessities of the code. Examinations were conducted to inspect as much of the specified volume and or surface as reasonably practical. In some cases, examinations were limited by geometric, metallurgical, or design access restrictions and may have been impeded by scanning limitations.

Relief Requests are being submitted pursuant to 10 CFR 50.55a (a)(3)(i), 10 CFR 50.55a (a)(3)(ii), 10 CFR 50.55a (g)(5)(iii) and/or in accordance with an approved Relief Request as stipulated in a Safety-Evaluation (SE) by the Office of Nuclear Reactor Regulation. The requests, along with their specific regulatory basis, are described in detail along with supporting technical discussion, documentation and referenced SE. Included in the Tables and Enclosures are welds examined during the 2nd Ten-Year Interval (8/86 -7/00) whose inspections have **not** met the code required coverage as defined in Code Case N-460, Alternative Examination Coverage for Class 1 and Class 2 Welds, along with sketches and/or drawings that depict the limiting condition.

A number of code items with limitations had been granted relief during the initial 2nd Interval Program Plan submittal and subsequent SER phase. These items are identified in the enclosures with a specific reference to the document granting the relief.

CLASS 1 COMPONENTS

Relief Request RR 2-Closeout-1, Rev. 0

Code Category B-A: **Pressure Retaining Welds in Reactor Vessel**

Item No. B1.10 **Shell Welds**
Item No. B1.12 **Longitudinal**

Relief is requested in accordance with the provisions of 10 CFR 50.55a (g)(5)(iii) from the full Code-required extent of volumetric examination for the following longitudinal weld in the Reactor Vessel Shell Weld listed in Table 1.

Examination Requirements

ASME Section XI, 1983 Edition through Summer 1983 Addenda, requires 100% volumetric examination of one longitudinal shell weld in the beltline region at a design structural discontinuity, if any.

Basis for Relief

Inspections of the Code-required accessible length of one weld was conducted on the RPV Shell Longitudinal Weld #5 where a design structural discontinuity is present with limitations which restricted access. Access to the area is limited because of the cutout for the outlet nozzle centered at 22 degrees. The effective length of the long seam weld is 70 inches. Scanning of the portions of the long seam weld #5 near the nozzle is limited due to interference from the nozzle boss. Coverage is estimated at 76%. (Ref. Technique drawings sheets 1, 14, & 15 attached).

Proposed Alternative Examinations

1. No additional volumetric examinations will be performed. The RPV Shell Longitudinal Weld #5 has been examined to the maximum extent practical from the inside surface.
2. A visual examination (VT-2) has been performed in conjunction with the pressure testing conducted on these components every refuel outage (with no evidence of leakage detected) in accordance with IWA-5000 and IWB-5000, which provides reasonable assurance of component integrity.

RPV Pressure Retaining Weld addressed by this Relief Request is listed in Table 1. Drawings highlighting component design, configuration of interferences, and limitations associated with the examinations are included in Enclosure 1.

Table 1

Code Category B-A / Item No. B1.12					
INT No.	Weld ID	System / Component	Extent Examined	Limitation	Remarks
1-1100	Weld #5	RPV Shell Weld – Longitudinal	76%	Restricted Access due to interference from the nozzle boss.	10” Accumulator Discharge Line 351

ENCLOSURE 1

**B-A, B1.12
(6 pages)**

INDIAN POINT UNIT 3

R. V. COVERAGE ESTIMATE BREAKDOWNS

DIRECTION / ORIENTATION

PARALLEL SCANS

UP / DN

PERP. SCANS

CW / CCW

ITEM / AREA Upper Shell Course Long Seam at 7°

WELD NO. 5

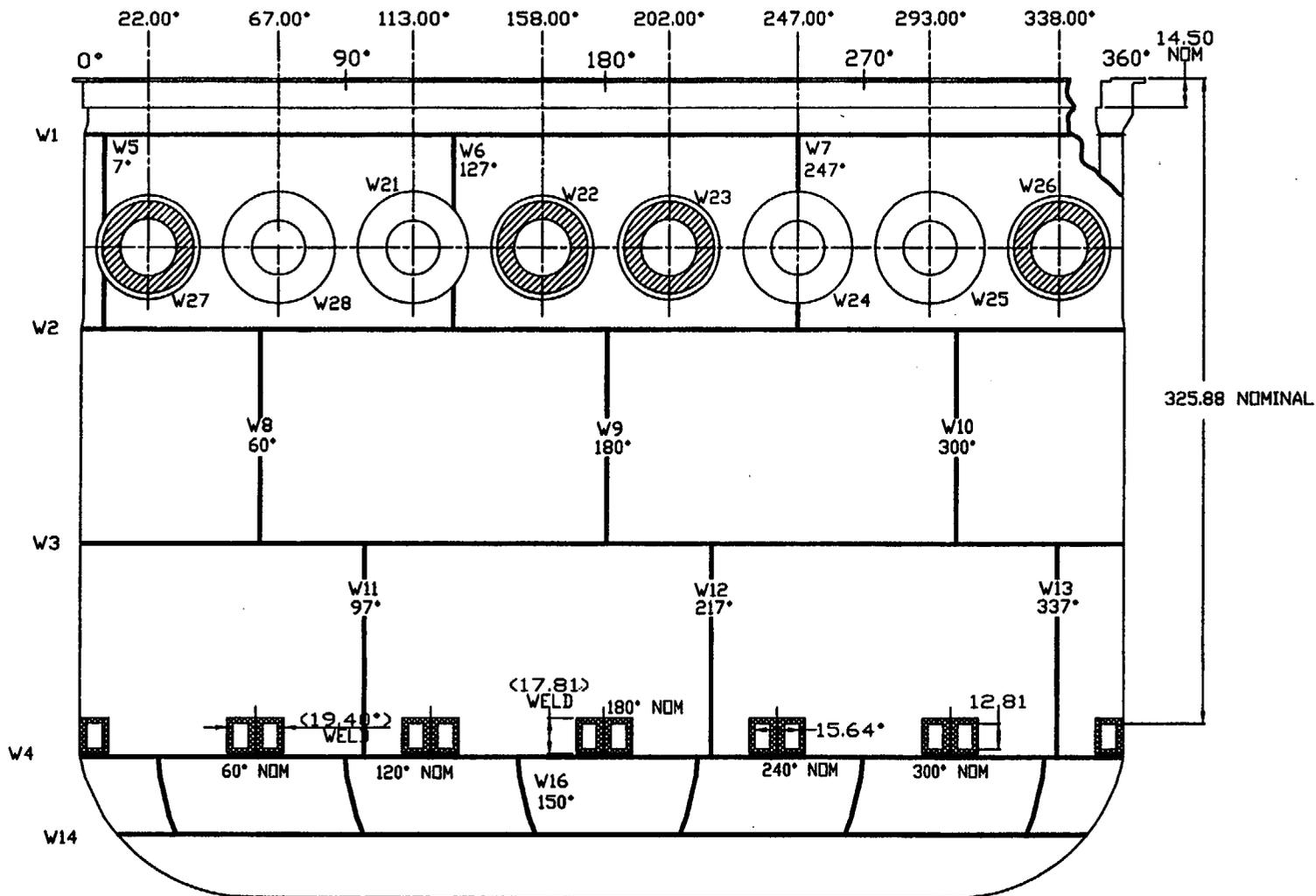
BEAM ANGLES

BEAM DIRECTION	45°		60°		70°		0°			
	WELD	VOLUME								
CCW	78	78	78	78	76	76	75	75		
CW	78	78	78	78	76	76				
UP (IN)	75	75	75	75	75	75				
DOWN (OUT)	75	75	75	75	75	75				
BORE AXIAL										

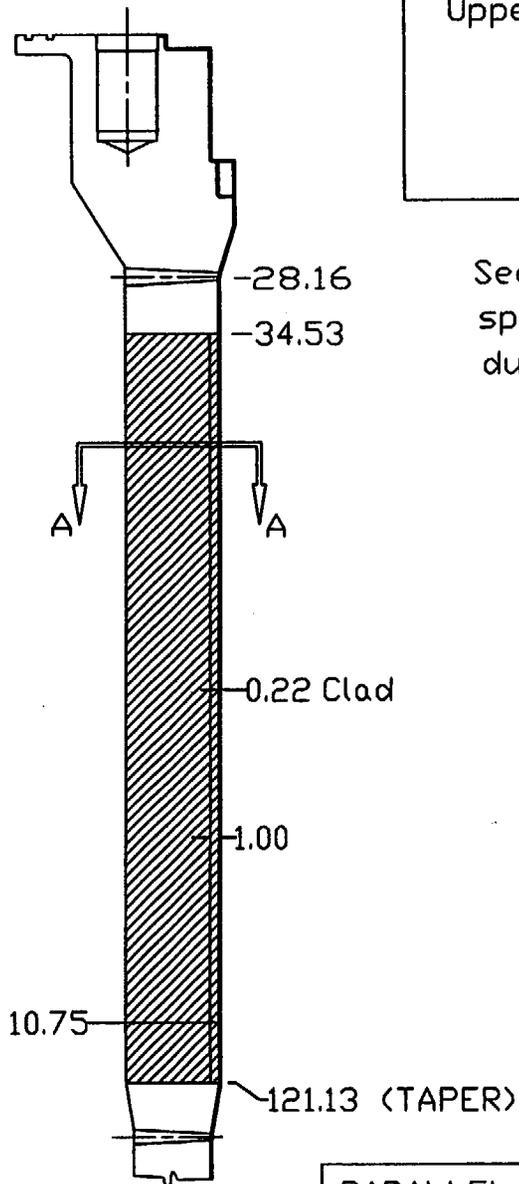
ANALYST



Westinghouse Proprietary Class 2C



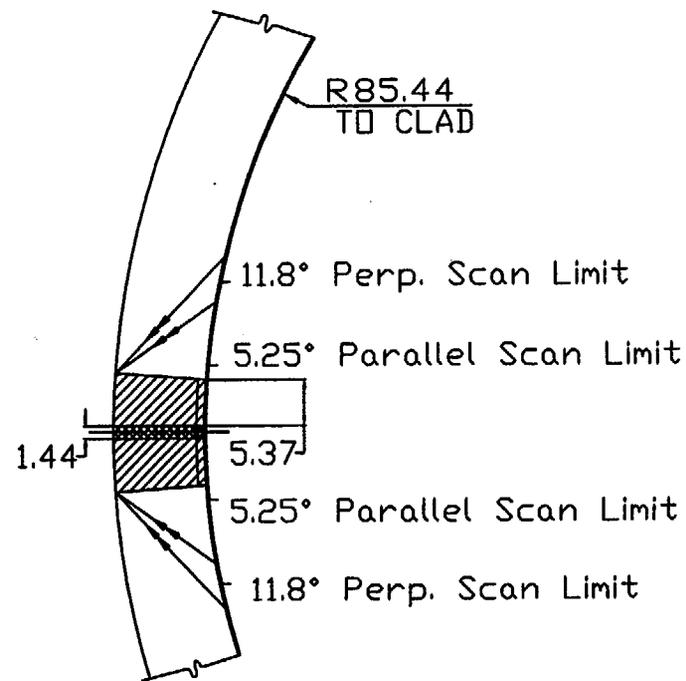
INDIAN POINT 3	
WesDyne International	
SHEET TITLE	Vessel Rollout
WesDyne Drawing # INT-99	
ALL DIMENSIONS IN INCHES UNLESS OTHERWISE NOTED	SHEET 1 OF 23



Upper Long. Welds
 W5 @ 7°
 W6 @ 127°
 W7 @ 247°

See next sheet for
 specific scan areas
 due to limitations

PARALLEL INDEX = 0.33°
 PERP INDEX = 0.5°



SECTION A-A

INDIAN POINT 3	
WesDyne International	
SHEET TITLE	UPPER LONG SEAMS
NUMBER	WesDyne Drawing # INT-99
ALL DIMENSIONS IN INCHES UNLESS OTHERWISE NOTED	SHEET 14 OF 23

PARALLEL SCANS

Weld No.	Dtop	Dbottom	Dstart	Dstop
W5	49.5	59.25	1.75°	12.25°
W5	110.0	115.0	1.75°	12.25°
W6	49.5	67.9	121.75°	132.25°
W6	94.25	115.0	121.75°	132.25°
W6	89.0	94.5	126.75°	138.8°
W7	49.5	61.75	276°	288°
W7	109.0	115.0	241.75°	252.25°

PERPENDICULAR SCANS

Weld No.	Dtop	Dbottom	Dstart	Dstop
W5	35.25	62.25	355.2° (-4.8°)	18.8°
W5	112.0	119.0	355.2° (-4.8°)	18.8°
W6	35.25	65.25	115.2°	138.8°
W6	106.0	119.0	115.2°	138.8°
W7*	35.25	52.0	235.2°	258.8°

* Note: Coverage of the lower portion of W7 accomplished with inlet nozzle Tan scan and scans of W2.

UPPER LONG WELDS

W5 @ 7°
W6 @ 127°
W7 @ 247°

INDIAN POINT 3

WesDyne International

SHEET TITLE UPPER LONG SEAMS

WesDyne Drawing # INT-99

ALL DIMENSIONS IN INCHES
UNLESS OTHERWISE NOTED SHEET 15 OF 23

NEW YORK POWER AUTHORITY
INDIAN POINT UNIT 3
REACTOR VESSEL EXAMINATION
SITE EXIT SUMMARY

09-27-99

Introduction

The Indian Point Unit 3 reactor vessel was examined by WesDyne International in conjunction with the Westinghouse Electric Company, Nuclear Services Division from Sept. 22 through Sept. 27, 1999. Examinations were governed by the Indian Point Unit 3 Plant Technical Specifications, The ASME Boiler and Pressure Vessel Code, Section XI and Section V, 1983 Ed. including summer 1983 Addenda, USNRC Regulatory Guide 1.150 Revision 1 and operating procedure INT-ISI-254, Revision 0. Manual examinations of the flange to shell weld were conducted per procedure INT-ISI_54 Rev. 0. Manual examinations of the threads in flange were conducted per procedure INT-ISI-55 Rev. 0.

Examinations and related activities including calibrations were witnessed by the Indian Point Unit 3 technical staff, the Authorized Nuclear Inservice Inspector and representatives from the USNRC.

Remote Examinations

Instructions for calibrations, examinations and assessment of recorded data is contained in procedure INT-ISI-254 Revision 0. Considered an integral part of the procedure is the Examination Program Plan, Rev. 0. The program plan is a plant specific document detailing the calibration blocks, beam angles, arrangement of transducers on the various end-effectors and twenty-three drawings depicting the reactor vessel section profiles and scanning paths. The scanning paths depicted in the technique drawings served as the input data for the models generated in the Supreem robot motion control software.

Examinations were conducted completely in the contact technique using two WesDyne Paragon multi-channel data acquisition systems, one interfaced to each Supreem scanning platform. The exam plan specified the use of 45, and 60 degree shear wave single element, 0 degree L wave single element and 70 degree L-wave triple element transducers for all vessel shell weld exams. All shell welds were and specified adjacent examination volumes were examined in both parallel and perpendicular scanning directions.

Limitations

Certain areas of a Westinghouse design 4-loop reactor vessel have obstructions which do not allow for transducer movement thereby limiting the coverage of the volumes. Following is a summary of areas where coverage cannot be calculated by any method as being at least 90%.

Weld 4, lower head to lower shell weld.

A total of 6 core support lugs occupying a space of about 20 degrees each are positioned in elevation about 4 inches above the lower shell to lower head circumferential weld. Scans were conducted between core lugs in the perpendicular and parallel directions and the scanning boundaries were maximized by visually assisted positioning of the exam head at the starting and stopping points. Examinations were also conducted beneath the core support lugs to the extent practical in the parallel and perpendicular scanning direction. Complete coverage is not practical even with additional scans and re-positioning of transducers. Coverage is estimated at 66%. (Ref. Technique drawings sheets 1,20 and 21)

Weld 5, upper shell course long seam at 7 degrees vessel axis.

Upper shell long weld No. 5 is located in the vessel upper shell course at an angular location where it intersects the cutout for the outlet nozzle centered at 22 degrees. The effective length of the long seam is 70 inches. Scanning of the portions of the long seam near the nozzle is limited due to interference from the nozzle boss. Coverage is estimated at 76%. (Ref. technique drawings, sheets 1, 14 and 15)

Outlet nozzle to shell weld, tangential scans. Welds 22,23,26 and 27

Nozzle to shell welds are examined from the bore in axial scans and from the vessel inside diameter surface to examine for transverse defects. For the outlet nozzles, the nozzle boss or protrusion completely obstructs scanning of the portion of the examination volume in the nozzle barrel. Coverage is estimated at 100% from the bore, 43% from the vessel ID. (Ref. technique drawings, sheet 12)

D. Kurek
WesDyne International
9-27-99

Code Category B-A: Pressure Retaining Welds in Reactor Vessel

Item No. B1.20 RPV Bottom Head Welds
Item No. B1.21 Circumferential

- Relief Request RR 2-2, rev. 0 for Item B1.21 previously granted via SER (TAC No. 72247, dated November 7, 1991)

Code Category B-A: **Pressure Retaining Welds in Reactor Vessel**

Item No. B1.30 **Shell to Flange Weld**

- Relief Request RR 2-4, rev. 0 for Item B1.30 previously granted via SER (TAC No. 72247, dated November 7, 1991)

Code Category B-A: **Pressure Retaining Welds in Reactor Vessel**

Item No. B1.40 **Head to Flange Weld**

- Relief Request RR 2-5, rev. 1 for item B1.40 previously approved via SER (TAC No. M82269, dated December 21, 1994)

Code Category B-B: Pressure Retaining Welds in Vessels other than Reactor Vessel

Item No. B2.11 Pressurizer Shell-to-Head – Circumferential

Item No. B2.12 Pressurizer Shell-to-Head – Longitudinal

- Relief Request RR 2-8, rev. 0 for item B2.11 & 2.12 previously granted (Ref. TAC No. 72247, dated November 7, 1991).

Relief Request RR 2-Closeout-2, Rev. 0

Code Category B-D: **Full Penetration Welds of Nozzles in Vessels**

Item No. B3.90 **Reactor Vessel Nozzle -to-Vessel Welds**

Relief is requested in accordance with the provisions of 10 CFR 50.55a (g)(5)(iii) from the full Code-required extent of volumetric examination for the following Nozzle-to-Vessel welds listed in Table 2.

Examination Requirements

ASME Section XI, 1983 Edition through Summer 1983 Addenda, requires 100% volumetric examination of all nozzle-to-vessel welds.

Basis for Relief

Inspections of the Code-required accessible length of all nozzle-to-vessel welds were conducted from the bore in axial scans and from the vessel inside diameter surface to examine for transverse defects in accordance with RG 1.150, rev. 1 and the Westinghouse Position Paper (Reference SER, TAC No. 72247, Page 8, dated November 7, 1991, Relief Request RR 2-1, rev. 0 for which the NRC staff had previously determined no relief is required).

For the outlet nozzles, the nozzle boss or protrusion completely obstructs scanning of the portion of the examination volume in the nozzle barrel. Coverage is estimated at 100% from the bore, 43% from the vessel ID (Ref. technique drawings, sheets 1 and 12, and data sheets for Welds #22, 23, 26, 27 attached).

Proposed Alternative Examinations

1. No additional volumetric examinations will be performed. The RPV Nozzle-to-Vessel Welds #22, #23, #26, and #27 have been examined to the maximum extent practical from the inside surface.
2. A visual examination (VT-2) has been performed in conjunction with the pressure testing conducted on these components every refuel outage (with no evidence of leakage detected) in accordance with IWA-5000 and IWB-5000, which provides reasonable assurance of component integrity.

RPV Pressure Retaining Welds addressed by this Relief Request are listed in Table 2. Drawings and data sheets highlighting component design, configuration of interference, and limitations associated with the examinations are included in Enclosure 2.

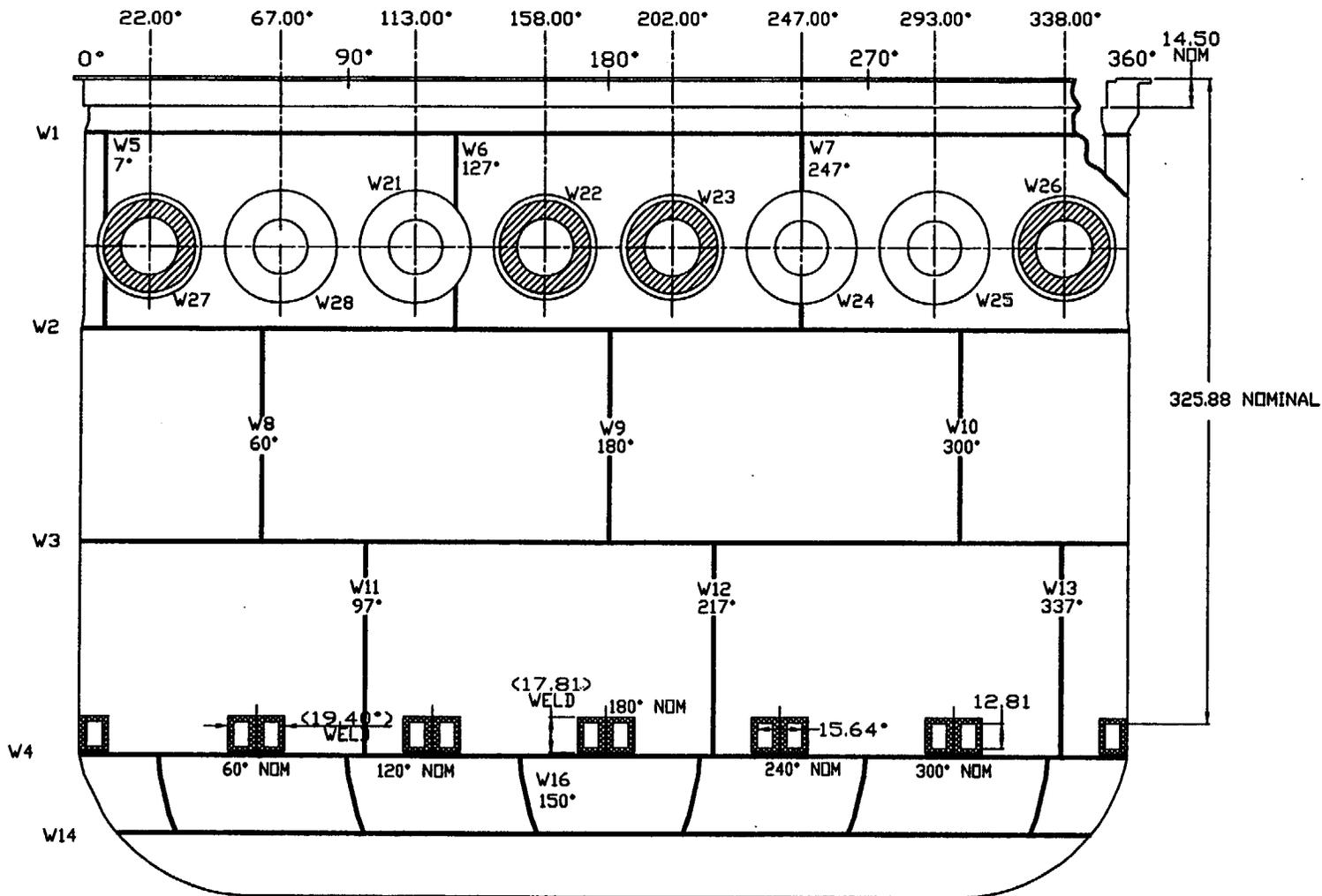
Table 2

Code Category B-D / Item No. B3.90					
INT No.	Weld ID	System / Component	Extent Examined	Limitation	Remarks
1-1100	Weld #22	RPV Nozzle-to-Vessel Weld	100 % from the bore, 43% from the vessel ID	Restricted Access due to interference from the nozzle boss.	
1-1100	Weld #23	RPV Nozzle-to-Vessel Weld	100 % from the bore, 43% from the vessel ID	Restricted Access due to interference from the nozzle boss.	
1-1100	Weld #26	RPV Nozzle-to-Vessel Weld	100 % from the bore, 43% from the vessel ID	Restricted Access due to interference from the nozzle boss.	
1-1100	Weld #27	RPV Nozzle-to-Vessel Weld	100 % from the bore, 43% from the vessel ID	Restricted Access due to interference from the nozzle boss.	

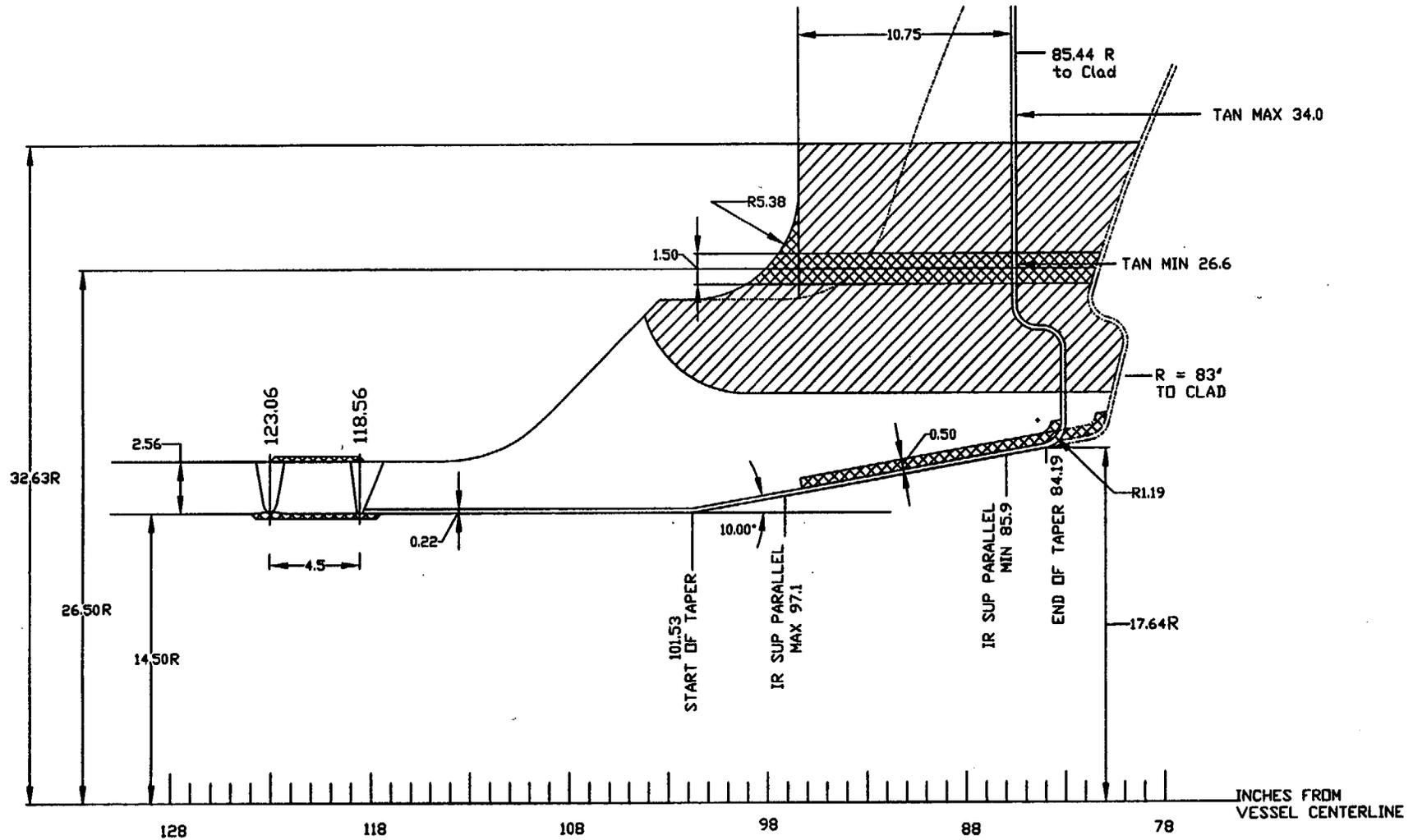
ENCLOSURE 2

**B-D, B3.90
(8 pages)**

Westinghouse Proprietary Class 2C



INDIAN POINT 3	
WesDyne International	
SHEET	Vessel Rollout
TITLE	
WesDyne Drawing # INT-99	
ALL DIMENSIONS IN INCHES UNLESS OTHERWISE NOTED	SHEET 1 OF 23



IR SUP INDEX INCREMENT = 0.3°
 TAN SCAN INDEX INCREMENT = 0.5°

INDIAN POINT 3	
WesDyne International	
SHEET TITLE	Outlet IR - TAN
WesDyne Drawing # INT-99	
ALL DIMENSIONS IN INCHES UNLESS OTHERWISE NOTED	SHEET 12 OF 23

INDIAN POINT UNIT 3

R. V. COVERAGE ESTIMATE BREAKDOWNS

DIRECTION / ORIENTATION

PARALLEL SCANS

Tangential

PERP. SCANS

Bore IN/OUT

ITEM / AREA

Outlet Nozzle to Shell at 158°

WELD NO. 22

BEAM ANGLES

BEAM DIRECTION	45°S ⁽¹⁾		60°S ⁽¹⁾		70°L ⁽¹⁾		10°L ⁽²⁾		50°L ⁽²⁾	
	WELD	VOLUME								
CCW		43		43		43				
CW		43		43		43				
UP (IN)										
DOWN (OUT)										
BORE AXIAL							89.3	89.3	89.4	89.4

- (1) Nozzle Boss Limitation, Tangential Scan
- (2) 10°L and 50°L Designed to work together
Combined Coverage – 98.5%

ANALYST



INDIAN POINT UNIT 3

R. V. COVERAGE ESTIMATE BREAKDOWNS

DIRECTION / ORIENTATION

PARALLEL SCANS
PERP. SCANS

Tangential
Bore IN/OUT

ITEM / AREA

Outlet Nozzle to Shell at 202°

WELD NO. 23

BEAM ANGLES

BEAM DIRECTION	45°S (1)		60°S (1)		70°L (1)		10°L (2)		50°L (2)	
	WELD	VOLUME								
CCW		43		43		43				
CW		43		43		43				
UP (IN)										
DOWN (OUT)										
BORE AXIAL							89.3	89.3	89.4	89.4

(1) Nozzle Boss Limitation, Tangential Scan

(2) 10°L and 50°L Designed to work together
Combined Coverage – 98.5%

ANALYST

[Signature] 9/29/99

INDIAN POINT UNIT 3

R. V. COVERAGE ESTIMATE BREAKDOWNS

DIRECTION / ORIENTATION

PARALLEL SCANS

Tangential

PERP. SCANS

Bore IN/OUT

ITEM / AREA

Outlet Nozzle to Shell at 338°

WELD NO. 26

BEAM ANGLES

BEAM DIRECTION	45°S (1)		60°S (1)		70°L (1)		10°L (2)		50°L (2)	
	WELD	VOLUME								
CCW		43		43		43				
CW		43		43		43				
UP (IN)										
DOWN (OUT)										
BORE AXIAL							89.3	89.3	89.4	89.4

- (1) Nozzle Boss Limitation, Tangential Scan
- (2) 10°L and 50°L Designed to work together
Combined Coverage – 98.5%

ANALYST *Joseph 2* 9/29/99

INDIAN POINT UNIT 3

R. V. COVERAGE ESTIMATE BREAKDOWNS

DIRECTION / ORIENTATION

PARALLEL SCANS
PERP. SCANS

Tangential
Bore IN/OUT

ITEM / AREA

Outlet Nozzle to Shell at 22°

WELD NO. 27

BEAM ANGLES

BEAM DIRECTION	45°S (1)		60°S (1)		70°L (1)		10°L (2)		50°L (2)	
	WELD	VOLUME								
CCW		43		43		43				
CW		43		43		43				
UP (IN)										
DOWN (OUT)										
BORE AXIAL							89.3	89.3	89.4	89.4

- (1) Nozzle Boss Limitation, Tangential Scan
- (2) 10°L and 50°L Designed to work together
Combined Coverage – 98.5%

ANALYST Yough 2 9/29/99

NEW YORK POWER AUTHORITY
INDIAN POINT UNIT 3
REACTOR VESSEL EXAMINATION
SITE EXIT SUMMARY

09-27-99

Introduction

The Indian Point Unit 3 reactor vessel was examined by WesDyne International in conjunction with the Westinghouse Electric Company, Nuclear Services Division from Sept. 22 through Sept. 27, 1999. Examinations were governed by the Indian Point Unit 3 Plant Technical Specifications, The ASME Boiler and Pressure Vessel Code, Section XI and Section V, 1983 Ed. including summer 1983 Addenda, USNRC Regulatory Guide 1.150 Revision 1 and operating procedure INT-ISI-254, Revision 0. Manual examinations of the flange to shell weld were conducted per procedure INT-ISI_54 Rev. 0. Manual examinations of the threads in flange were conducted per procedure INT-ISI-55 Rev. 0.

Examinations and related activities including calibrations were witnessed by the Indian Point Unit 3 technical staff, the Authorized Nuclear Inservice Inspector and representatives from the USNRC.

Remote Examinations

Instructions for calibrations, examinations and assessment of recorded data is contained in procedure INT-ISI-254 Revision 0. Considered an integral part of the procedure is the Examination Program Plan, Rev. 0. The program plan is a plant specific document detailing the calibration blocks, beam angles, arrangement of transducers on the various end-effectors and twenty-three drawings depicting the reactor vessel section profiles and scanning paths. The scanning paths depicted in the technique drawings served as the input data for the models generated in the Supreme robot motion control software.

Examinations were conducted completely in the contact technique using two WesDyne Paragon multi-channel data acquisition systems, one interfaced to each Supreme scanning platform. The exam plan specified the use of 45, and 60 degree shear wave single element, 0 degree L wave single element and 70 degree L-wave triple element transducers for all vessel shell weld exams. All shell welds were and specified adjacent examination volumes were examined in both parallel and perpendicular scanning directions.

Limitations

Certain areas of a Westinghouse design 4-loop reactor vessel have obstructions which do not allow for transducer movement thereby limiting the coverage of the volumes. Following is a summary of areas where coverage cannot be calculated by any method as being at least 90%.

Weld 4, lower head to lower shell weld.

A total of 6 core support lugs occupying a space of about 20 degrees each are positioned in elevation about 4 inches above the lower shell to lower head circumferential weld. Scans were conducted between core lugs in the perpendicular and parallel directions and the scanning boundaries were maximized by visually assisted positioning of the exam head at the starting and stopping points. Examinations were also conducted beneath the core support lugs to the extent practical in the parallel and perpendicular scanning direction. Complete coverage is not practical even with additional scans and re-positioning of transducers. Coverage is estimated at 66%. (Ref. Technique drawings sheets 1,20 and 21)

Weld 5, upper shell course long seam at 7 degrees vessel axis.

Upper shell long weld No. 5 is located in the vessel upper shell course at an angular location where it intersects the cutout for the outlet nozzle centered at 22 degrees. The effective length of the long seam is 70 inches. Scanning of the portions of the long seam near the nozzle is limited due to interference from the nozzle boss. Coverage is estimated at 76%. (Ref. technique drawings, sheets 1, 14 and 15)

Outlet nozzle to shell weld, tangential scans. Welds 22,23,26 and 27

Nozzle to shell welds are examined from the bore in axial scans and from the vessel inside diameter surface to examine for transverse defects. For the outlet nozzles, the nozzle boss or protrusion completely obstructs scanning of the portion of the examination volume in the nozzle barrel. Coverage is estimated at 100% from the bore, 43% from the vessel ID. (Ref. technique drawings, sheet 12)

D. Kurek
WesDyne International
9-27-99

Code Category B-D: **Full Penetration Welds of Nozzles in Vessels**

Item No. B3.120 **Pressurizer Nozzle Inside Radius Sections**

- Relief Request RR 2-9, rev. 1 for item B3.120 previously submitted and approved (Ref. TAC No. 72247, page 18, dated November 7, 1991)

Relief Request RR 2-Closeout-3, Rev. 0

Code Category B-D: **Full Penetration Welds of Nozzles in Vessels**

Item No. B3.140 **Steam Generators (Primary Side) Nozzle Inside Radius Sections**

Relief is requested in accordance with the provisions of 10 CFR 50.55a (g)(5)(iii) from the full Code-required extent of volumetric examination for the Steam Generator (Primary Side) Nozzle Inside Radius Sections listed in Table 3.

Examination Requirements

ASME Section XI, 1983 Edition through Summer 1983 Addenda, requires 100% of the Steam Generator Nozzle Inside Radius Sections be volumetrically examined.

Basis for Relief

Complete inspection of the Code-required volume is not possible due to restriction caused by Support Lugs on the Nozzle configuration (see attached sketch and data sheets, Enclosure 3). These welds have been examined to the maximum extent practical in accordance with the standard industry practices.

Proposed Alternative Examinations

1. No additional volumetric examinations will be performed on these welds. The components listed in this relief request have been examined to the maximum extent practical.
2. A visual inspection (VT-2) is performed in conjunction with the pressure testing conducted on these components every refuel outage (with no leakage detected) in accordance with IWA-5000 and IWB-5000, which provides reasonable assurance of component integrity.

Drawings and sketches that illustrate the restricted conditions encountered which limit examination coverage are included in Enclosure 3.

Table 3

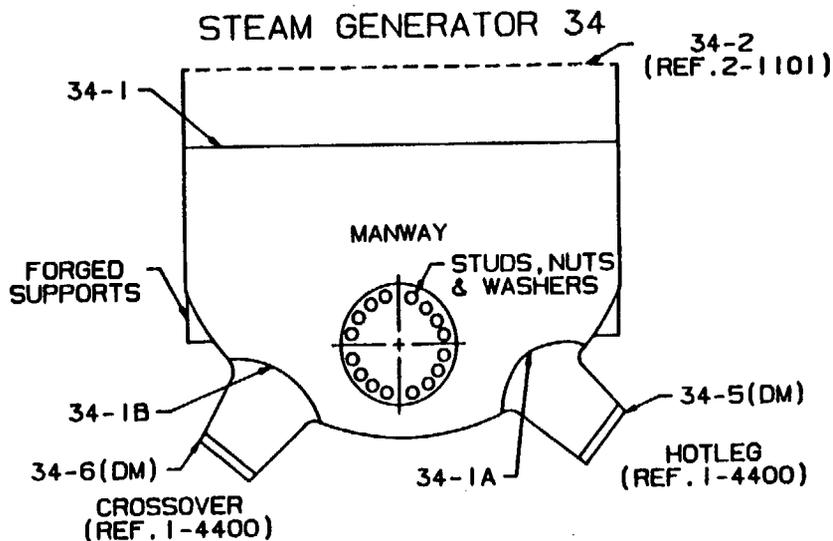
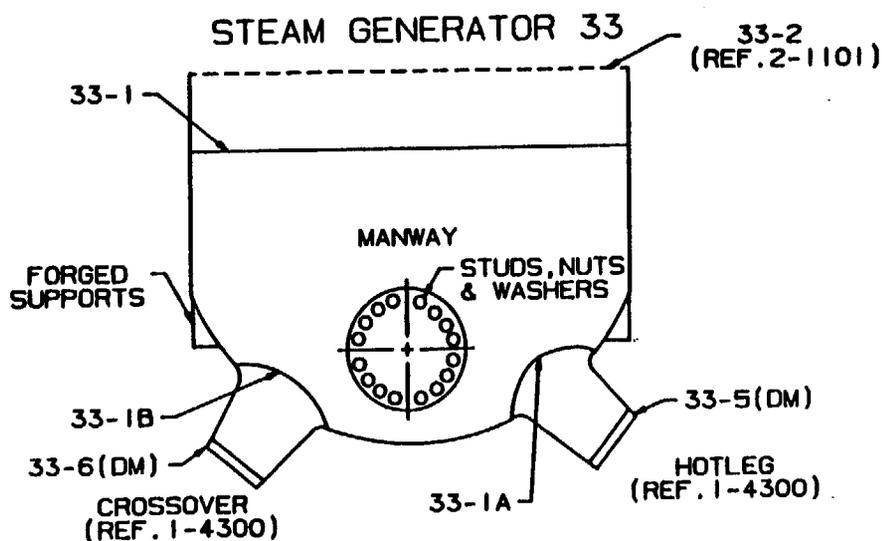
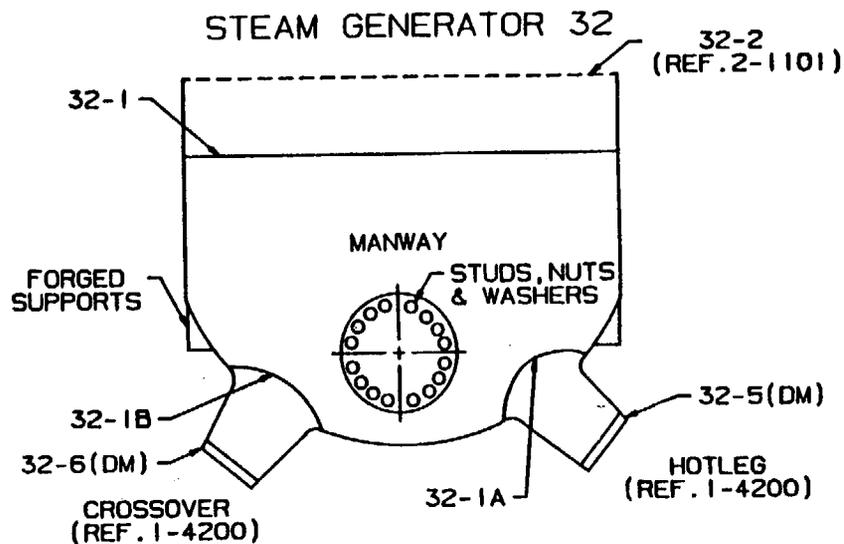
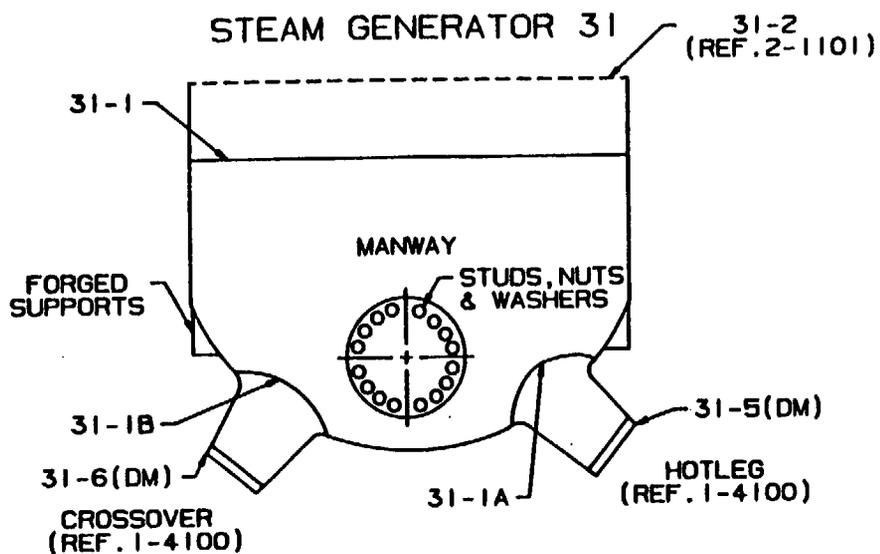
Code Category B-D /Item No.B3.140					
INT No.	Weld ID	System / Component	Extent Examined	Limitation	Remarks
1-3101	Weld 31-1A	RCS – Steam Generator	50% ⁽¹⁾	Support Lugs on the Nozzle Configuration	
1-3101	Weld 31-1B	RCS – Steam Generator	50% ⁽¹⁾	Support Lugs on the Nozzle Configuration	
1-3101	Weld 32-1A	RCS – Steam Generator	50% ⁽¹⁾	Support Lugs on the Nozzle Configuration	
1-3101	Weld 32-1B	RCS – Steam Generator	50% ⁽¹⁾	Support Lugs on the Nozzle Configuration	
1-3101	Weld 33-1A	RCS – Steam Generator	50% ⁽¹⁾	Support Lugs on the Nozzle Configuration	

NOTES:

(1) A 50% exam coverage credit is taken for these types of geometry limitations.

ENCLOSURE 3

**B-D, B3.140
(11 pages)**



CHANNEL HEAD TO TUBESHEET WELDS 31-1, 32-1, 33-1 & 34-1:
 5.65" T SA508 CLASS 3 CARBON STEEL;
 DIAMETER: 129.25"; CIRCUMFERENCE: 405.84"
 0 REFERENCE: TOP CENTERLINE OF HOTLEG MANWAY

NOZZLE TO VESSEL WELDS: NOT APPLICABLE
 NOZZLE INSIDE RADIUS SECTIONS 31-1A, 31-1B, 32-1A, 32-1B,
 33-1A, 33-1B, 34-1A & 34-1B: SA 508 CLASS 3 CARBON STEEL
 NOZZLE TO SAFE-ENDS 31-5(DM), 31-6(DM), 32-5(DM), 32-6(DM), 33-5(DM),
 33-6(DM), 34-5(DM) & 34-6(DM): 31" I.D.
 SA508 CLASS 3 CARBON STEEL WITH TYPE 308L STAINLESS STEEL
 MANWAY STUDS, NUTS & WASHERS: HOTLEG AND COLDLEG: 16 STUDS (DIA. 1.88"),
 16 NUTS AND 32 WASHERS EACH MANWAY
 INTEGRALLY WELDED ATTACHMENTS: NOT APPLICABLE

NEW YORK POWER AUTHORITY
 INDIAN POINT UNIT NO. 3

REPLACEMENT
 STEAM GENERATORS
 RCPCSG1-31, RCPCSG2-32,
 RCPCSG3-33 & RCPCSG4-34

INT-1-3101

REV.
 1

WESTINGHOUSE NUCLEAR SERVICE DIVISION INSPECTION SERVICES ULTRASONIC INDICATION DATA	PLANT	INDIAN POINT	UNIT	NO. 3	SKETCH	INT-1-3101
	SYST/COMP	REPLACEMENT STEAM GENERATOR RCPCSG1-31			PROCEDURE	INT-ISI-248 REV.0
	EXAMINER (LEVEL II)	<i>William J Kelly Robert Harout Bill A...</i>			DATE	10-28-90
	LINEARITY AND CAL. CHECK	SAT			CAL. BLOCK THICKNESS	13.5"

WELD NUMBER	INDICATION LENGTH		MIN. 50%		MIN. 100%		PEAK			MAX. 100%		MAX. 50%		BEAM		THICKNESS			REMARKS
	FROM	TO	SWP.	IN. TO REF.	SWP.	IN. TO REF.	SWP.	IN. TO REF.	± DAC AMPL.	SWP.	IN. TO REF.	SWP.	IN. TO REF.	ANGLE	DIR.	SURF. 2	WELD	SURF. 5	
31-18	(1) 1 1/2"	7"					1.6	50 1/2"	100% +18dB					30°L	7	(2) 7.09"	NA	(3) 8.94"	(4)
31-18	(1) 1 1/2"	7"					4.5	52"	100% +4dB					30°L	7	(2) 7.09"	NA	(3) 8.94"	(4)
31-18	(1) 1 1/2"	7"					1.6	54 1/2"	100% +18dB					30°L	8	(2) 7.09"	NA	(3) 8.94"	(4)
31-18	(1) 1 1/2"	7"					4.5	55 3/4"	100% +4dB					30°L	8	(2) 7.09"	NA	(3) 8.94"	(4)
NOTE (1) FROM AND TO MEASUREMENTS TAKEN FROM CENTERLINE OF 5" BLENDED RADIUS. FOR 0" DATUM POINT REFERENCE LIMITATION TO EXAMINATION SHEET.																			
NOTE (2) THICKNESS AT 6" FROM CENTERLINE OF BLENDED RADIUS.																			
NOTE (3) THICKNESS AT 1 1/2" FROM CENTERLINE OF BLENDED RADIUS.																			
NOTE (4) GEOMETRIC REFLECTOR FROM 0.64" DIA. DRAIN TUBE.																			
<i>[Signature]</i> 10/29/90 CITE NUC LEVEL III																			

MS/HSB
 11-2-90

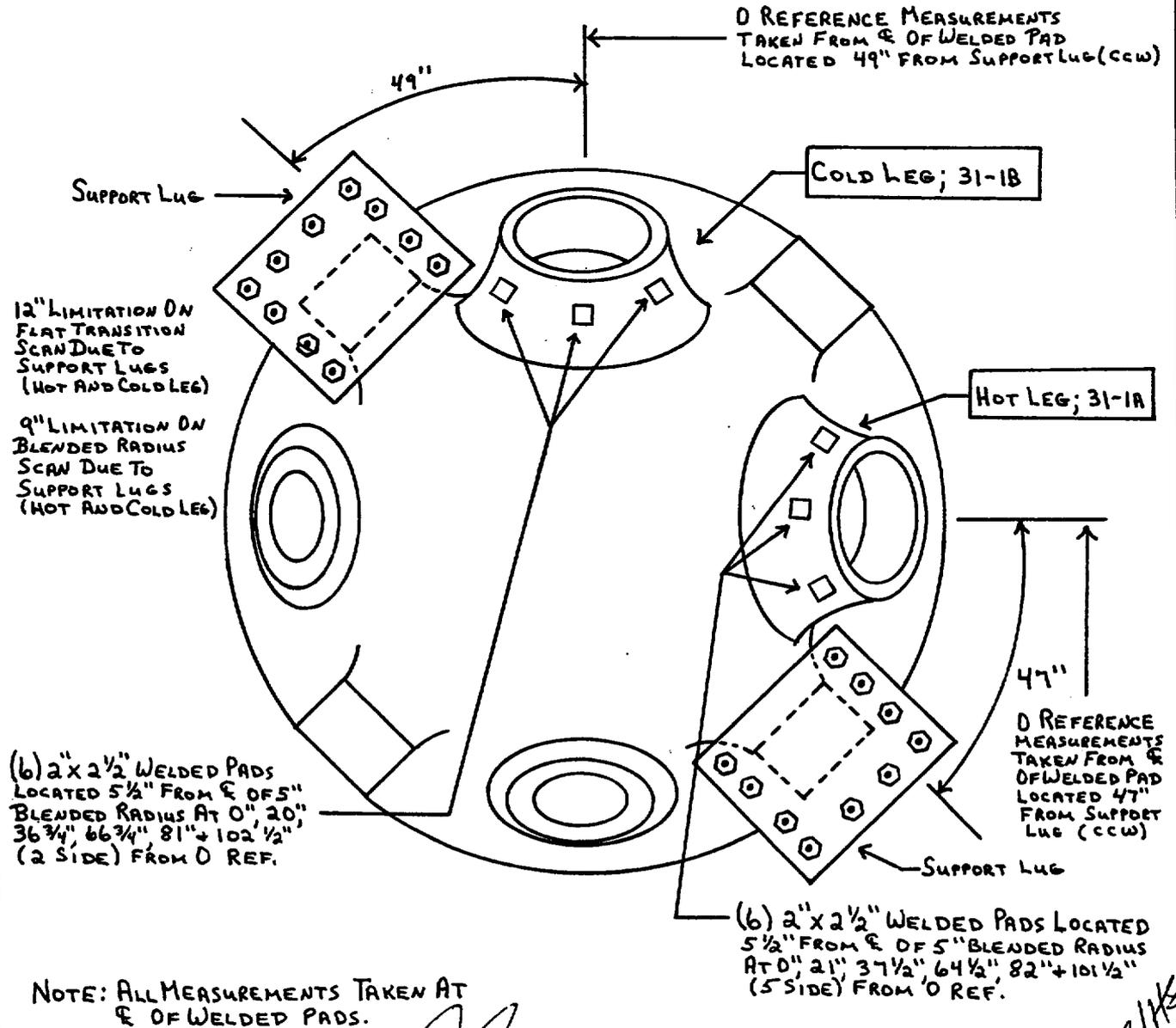
WESTINGHOUSE NUCLEAR SERVICE DIVISION
INSPECTION SERVICES

LIMITATION TO EXAMINATION

PLANT INDIAN POINT UNIT No. 3 SKETCH INT-1-3101
 REPLACEMENT STEAM GENERATOR
 SYST/COMP RCPCSG1-31 PROCEDURE INT-IST-248 REV.0
 EXAMINER William J. Kelly Robert H. Grant Bill Holt DATE 10-28-90
 LEVEL II

RELATED TO: U/T P/T _____ M/T _____ V/T _____ ITEM(S): 31-1A + 31-1B

PROVIDE GENERAL INFORMATION TO DESCRIBE APPROXIMATE SIZE, LOCATION AND TYPE OF LIMITATION.



APB/HSSB
10-28-90

WESTINGHOUSE NUCLEAR SERVICE DIVISION INSPECTION SERVICES WELD ULTRASONIC EXAMINATION INSERVICE	PLANT INDIAN POINT SYST. / COMP. REPLACEMENT STEAM GENERATOR RCPCSG2-32 EXAMINER (LEVEL II) <i>William J. Halley</i>	UNIT NO. 3	SKETCH INT-1-3101 REV. 1 PROCEDURE INT-ISI-248, REV. 0 DATE 5-31-97
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EQUIPMENT	TRANSDUCER	STRAIGHT BEAM SCAN DIRECTION 0	AXIAL SCANS DIRECTIONS 2 & 5				CIRCUMFERENTIAL SCANS DIRECTIONS 7 & 8				CALIBRATION CHECK	
INST. S / N STAVELEY SONIC/136-904K	S / N						B14653				INITIAL TIME 0939	
REP. RATE 1K	SIZE						0.5"X1.0"					
REJECT OFF	FREQ.						2.25 MHz					
	ANGLE						30° L					
DAMPING	CALIBRATION REFLECTOR LOCATION	SIGNAL AMPLITUDE	SWEEP POSITION	SIGNAL AMPLITUDE	SWEEP POSITION	DISTANCE FROM SCRIBE / REF. LINE TO 50% DAC LOCATIONS		SIGNAL AMPLITUDE	SWEEP POSITION	DISTANCE FROM SCRIBE / REF. LINE TO 50% DAC LOCATIONS		
500 OHMS						PEAK				PEAK		
FILTER 1	NOTCH A							80%	5.0	4.0"	3.7" 4.8s	4.4" 5.2s
LIN. CHECK	NOTCH B							50%	6.5	4.6"	4.2" 6.3s	5.1" 6.6s
SAT	NOTCH C							50%	8.0	5.45"	4.85" 7.9s	5.95" 8.3s
S. U. CABLE RG-174 12' BNC-BNC												
COUPLANT SONOTRACE 40 96343	CAL. GAIN							65.4dB				
												FINAL TIME 1133
												CAL. BLOCK INT-51
												THICKNESS 13.5"
												TEMPERATURE 85°F

WELD NUMBER	TEMP.	BASE METAL SCAN	SCAN DIRECTION				EXAMINATION LIMITATIONS	CROWN CONFIGURATION	RESULTS			REMARKS
			2	5	7/8	0			NI	NRI	RI	
32-1A	105°F				YES		(6) WELDED PADS (2" X 2 1/2") AND SUPPORT PLATE, SEE LIMITATION TO EXAMINATION SHEET 9% NOT EXAMINED	SMOOTH (FLAT TRANSITION)	X			SURFACE THERMOMETER S/N 10080
32-1B	105°F				YES		(6) WELDED PADS (2" X 2 1/2") AND SUPPORT PLATE, SEE LIMITATION TO EXAMINATION SHEET 9% NOT EXAMINED	SMOOTH (FLAT TRANSITION)	X			SURFACE THERMOMETER S/N 10080
												NOTE: SCRIBE/REF. LINE MEASUREMENTS TAKEN FROM NOTCHED EDGE OF CAL. BLOCK.

NEW YORK POWER AUTHORITY LEVEL III REVIEW / DATE <i>W. J. Halley</i> 6/17/97	ANII REVIEW / DATE <i>ASB</i> 6-18-97
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WESTINGHOUSE NUCLEAR SERVICE DIVISION INSPECTION SERVICES WELD ULTRASONIC EXAMINATION INSERVICE	PLANT INDIAN POINT	UNIT NO. 3	SKETCH INT-1-3101 REV. 1
	SYST. / COMP. REPLACEMENT STEAM GENERATOR RCPCSG2-32	PROCEDURE INT-ISI-248, REV. 0	
	EXAMINER (LEVEL II) <i>William H. Kelley</i>	DATE 5-31-97	

EQUIPMENT	TRANSDUCER	STRAIGHT BEAM SCAN DIRECTION 0		AXIAL SCANS DIRECTIONS 2 & 5				CIRCUMFERENTIAL SCANS DIRECTIONS 7 & 8				CALIBRATION CHECK		
INST. S / N STAVELEY SONIC/136-904K	S / N												INITIAL TIME 0935	
REP. RATE 600 Hz	SIZE													
REJECT OFF	FREQ.													
	ANGLE													
DAMPING 600 OHMS	CALIBRATION REFLECTOR LOCATION	SIGNAL AMPLITUDE	SWEEP POSITION	SIGNAL AMPLITUDE	SWEEP POSITION	DISTANCE FROM SCRIBE / REF. LINE TO 50% DAC LOCATIONS		SIGNAL AMPLITUDE	SWEEP POSITION	DISTANCE FROM SCRIBE / REF. LINE TO 50% DAC LOCATIONS				
FILTER 1	NOTCH D							80%	5.0	5.3"	4.9"	5.8"		
LIN. CHECK SAT	NOTCH E							50%	6.5	6.0"	4.9s	5.2s		
S. U. CABLE RG-174 12' BNC-BNC	NOTCH F							50%	8.0	6.85"	5.5"	6.5"	FINAL TIME 1135	
COUPLANT SONOTRACE 40 96343	CAL. GAIN													
								66dB						CAL. BLOCK INT-51
														THICKNESS 13.5"
														TEMPERATURE 85°F

WELD NUMBER	TEMP.	BASE METAL SCAN	SCAN DIRECTION				EXAMINATION LIMITATIONS	CROWN CONFIGURATION	RESULTS			REMARKS
			2	5	7/8	0			NI	NRI	RI	
32-1A	105°F				YES		SUPPORT LUG & PLATE, SEE LIMITATION TO EXAMINATION SHEET, 4.7% NOT EXAMINED	ROUNDED (BLENDED RADIUS)	X			SURFACE THERMOMETER S/N 10080
32-1B	105°F				YES		SUPPORT LUG & PLATE, SEE LIMITATION TO EXAMINATION SHEET, 4.7% NOT EXAMINED	ROUNDED (BLENDED RADIUS)	X			SURFACE THERMOMETER S/N 10080

NOTE: SCRIBE/REF. LINE MEASUREMENTS TAKEN FROM NOTCHED EDGE OF CAL. BLOCK.

NEW YORK POWER AUTHORITY LEVEL III REVIEW / DATE <i>W. H. Kelley</i> INT 6/17/97	ANR REVIEW / DATE <i>MSB</i> 6-18-97
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WESTINGHOUSE NUCLEAR SERVICE DIVISION
INSPECTION SERVICES

LIMITATION TO EXAMINATION

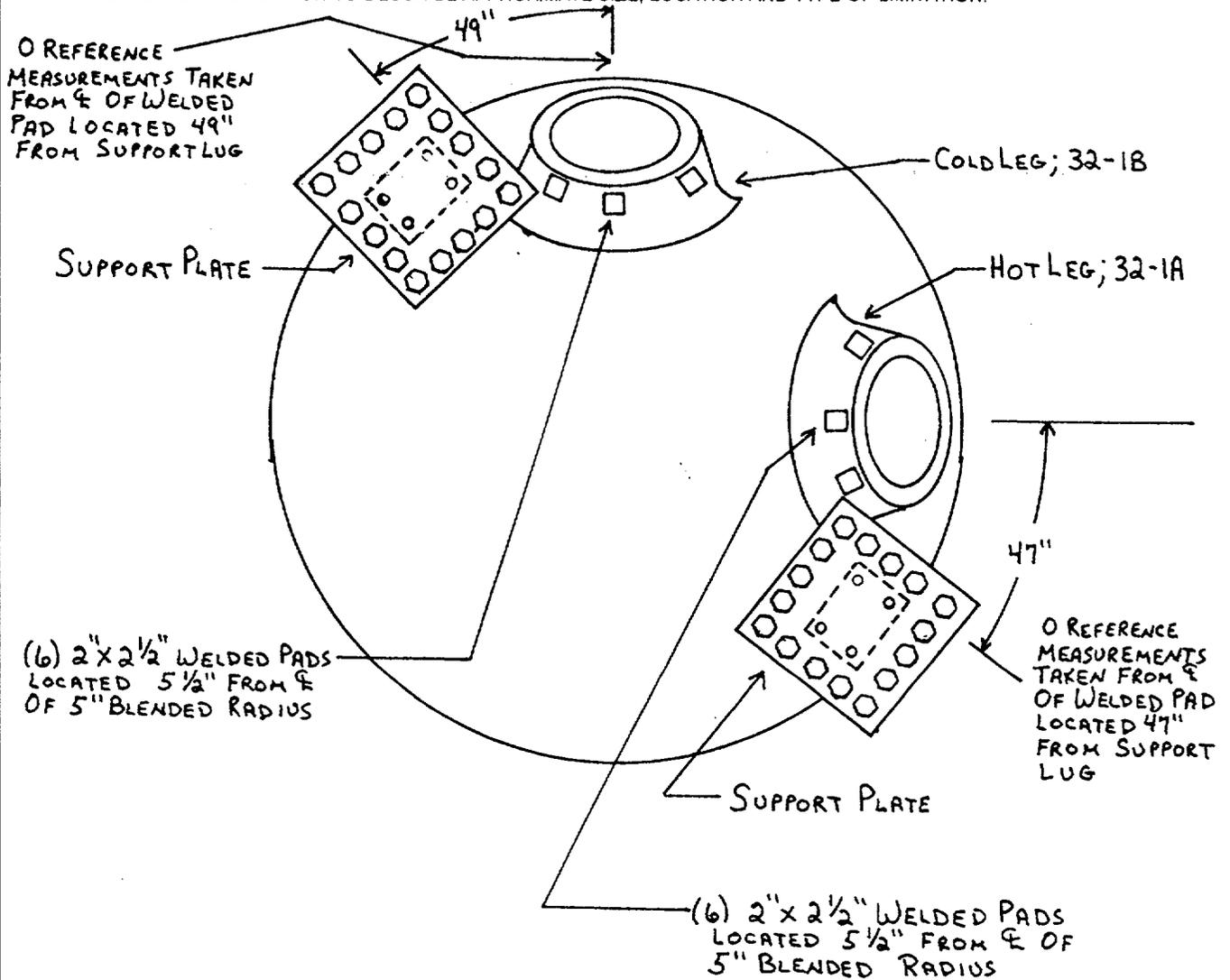
PLANT INDIAN POINT UNIT NO. 3 SKETCH INT-1-3101 REV. 1

SYST./COMP. REPLACEMENT STEAM GENERATOR RCPCSG2-32 PROCEDURE INT-4SI-248, REV. 0

EXAMINER William J. Halley DATE 5-31-97
LEVEL II

RELATED TO: U/T X P/T _____ M/T _____ V/T _____ ITEM(S) 32-1A & 32-1B

PROVIDE GENERAL INFORMATION TO DESCRIBE APPROXIMATE SIZE, LOCATION AND TYPE OF LIMITATION.



NEW YORK POWER AUTHORITY LEVEL III REVIEW / DATE

W. J. Halley LIII 6/17/97

ANII REVIEW / DATE

HSB HSB 6-18-97

WESTINGHOUSE NUCLEAR SERVICE DIVISION INSPECTION SERVICES				PLANT INDIAN POINT				UNIT NO. 3				SKETCH INT-1-3101 REV. 1													
WELD ULTRASONIC EXAMINATION INSERVICE				SYST. / COMP. REPLACEMENT STEAM GENERATOR RCPCSG3-33				PROCEDURE INT-ISI-248, REV. 0																	
				EXAMINER (LEVEL II) <i>William J. Holly</i>				DATE 5-31-97																	
				EQUIPMENT				TRANSducer				STRAIGHT BEAM SCAN DIRECTION 0													
INST. S / N STAVELEY SONIC/136-904K				S / N				AXIAL SCANS DIRECTIONS 2 & 5				CIRCUMFERENTIAL SCANS DIRECTIONS 7 & 8				CALIBRATION CHECK									
REP. RATE				SIZE								B14656				INITIAL TIME 0935									
500 Hz				FREQ.								0.5"X1.0"													
REJECT				ANGLE								2.25 MHz													
OFF												30° L													
DAMPING				CALIBRATION REFLECTOR LOCATION		SIGNAL AMPLITUDE		SWEEP POSITION		SIGNAL AMPLITUDE		SWEEP POSITION		DISTANCE FROM SCRIBE / REF. LINE TO PEAK		50% DAC LOCATIONS		SIGNAL AMPLITUDE		SWEEP POSITION		DISTANCE FROM SCRIBE / REF. LINE TO PEAK		50% DAC LOCATIONS	
600 OHMS																									
FILTER				NOTCH D														80%		5.0		5.3"		4.9" 5.8" 4.9s 5.2s	
1																		50%		6.5		6.0"		5.5" 6.5" 6.4s 6.8s	
LIN. CHECK				NOTCH E														50%		8.0		6.85"		6.3" 7.3" 7.9s 8.2s	
SAT				NOTCH F																				FINAL TIME 1135	
S. U. CABLE RG-174 12' BNC-BNC																									
COUPLANT																								CAL. BLOCK INT-51	
SONOTRACE 40 96343				CAL. GAIN														66dB						THICKNESS 13.5"	
																								TEMPERATURE 86°F	
WELD NUMBER		TEMP.		BASE METAL SCAN		SCAN DIRECTION				EXAMINATION LIMITATIONS				CROWN CONFIGURATION		RESULTS			REMARKS						
						2 5 7/8 0										NI NRI RI									
33-1A		105°F				YES				SUPPORT LUG & PLATE, SEE LIMITATION TO EXAMINATION SHEET, 4.7% NOT EXAMINED				ROUNDED (BLENDED RADIUS)		X			SURFACE THERMOMETER S/N 10080						
																			NOTE: SCRIBE/REF. LINE MEASUREMENTS TAKEN FROM NOTCHED EDGE OF CAL. BLOCK.						
NEW YORK POWER AUTHORITY / LEVEL III REVIEW / DATE <i>WJH</i> 6/17/97												ANII REVIEW / DATE <i>ASB</i> 6-18-97													

WESTINGHOUSE NUCLEAR SERVICE DIVISION INSPECTION SERVICES WELD ULTRASONIC EXAMINATION INSERVICE	PLANT INDIAN POINT SYST. / COMP. REPLACEMENT STEAM GENERATOR RCPGSG3-33 EXAMINER (LEVEL II) <i>William J. Hall</i>	UNIT NO. 3	SKETCH INT-1-3101 REV. 1 PROCEDURE INT-ISI-248, REV. 0 DATE 5-31-97
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EQUIPMENT	TRANSDUCER	STRAIGHT BEAM SCAN DIRECTION 0	AXIAL SCANS DIRECTIONS 2 & 5				CIRCUMFERENTIAL SCANS DIRECTIONS 7 & 8				CALIBRATION CHECK		
INST. S / N STAVELEY SONIC/136-904K	S / N						B14653				INITIAL TIME 0939		
REP. RATE 1K	SIZE FREQ.						0.5"X1.0" 2.25 MHz						
REJECT OFF	ANGLE						30° L						
DAMPING	CALIBRATION REFLECTOR LOCATION	SIGNAL AMPLITUDE	SWEEP POSITION	SIGNAL AMPLITUDE	SWEEP POSITION	DISTANCE FROM SCRIBE / REF. LINE TO 50% DAC LOCATIONS		SIGNAL AMPLITUDE	SWEEP POSITION	DISTANCE FROM SCRIBE / REF. LINE TO 50% DAC LOCATIONS			
600 OHMS						PEAK				PEAK			
FILTER 1	NOTCH A							80%	5.0	4.0"	3.7" 4.8s 4.2"	4.4" 5.2s 5.1"	
LIN. CHECK SAT	NOTCH B							50%	6.5	4.6"	6.3s 4.85"	8.6s 5.95"	
S. U. CABLE RG-174 12' BNC-BNC	NOTCH C							50%	8.0	5.45"	7.9s	8.3s	FINAL TIME 1133
COUPLANT SONOTRACE 40 96343	CAL. GAIN							65.4dB					CAL. BLOCK INT-51 THICKNESS 13.5" TEMPERATURE 85°F

WELD NUMBER	TEMP.	BASE METAL SCAN	SCAN DIRECTION				EXAMINATION LIMITATIONS	CROWN CONFIGURATION	RESULTS			REMARKS
			2	5	7/8	0			NI	NRI	RI	
33-1A	105°F				YES		(6) WELDED PADS (2" X 2 1/2") AND SUPPORT PLATE, SEE LIMITATION TO EXAMINATION SHEET 9% NOT EXAMINED	SMOOTH (FLAT TRANSITION)	X			SURFACE THERMOMETER S/N 10080
												NOTE: SCRIBE/REF. LINE MEASUREMENTS TAKEN FROM NOTCHED EDGE OF CAL. BLOCK.

NEW YORK POWER AUTHORITY LEVEL III REVIEW / DATE <i>W. J. Hall</i> WJH 6/17/97	ANN REVIEW / DATE <i>W. J. Hall</i> WJH 6-18-97
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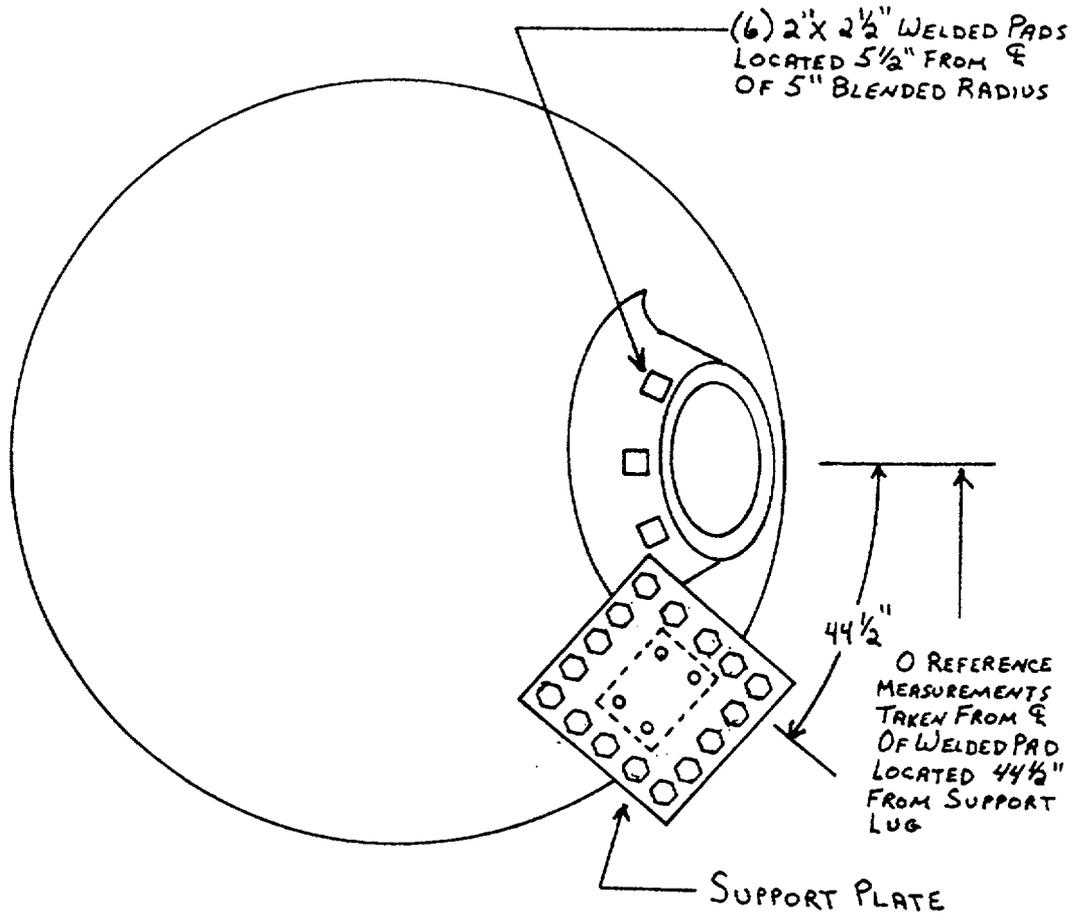
WESTINGHOUSE NUCLEAR SERVICE DIVISION
INSPECTION SERVICES

LIMITATION TO EXAMINATION

PLANT INDIAN POINT UNIT NO. 3 SKETCH INT-1-3101 REV. 1
SYST./COMP. REPLACEMENT STEAM GENERATOR RCPCSG3-33 PROCEDURE INT-4SI-248, REV. 0
EXAMINER William G. Halley DATE 5-31-97
LEVEL II

RELATED TO: U/T X P/T _____ M/T _____ V/T _____ ITEM(S) 33-1A

PROVIDE GENERAL INFORMATION TO DESCRIBE APPROXIMATE SIZE, LOCATION AND TYPE OF LIMITATION.



NEW YORK POWER AUTHORITY / LEVEL III REVIEW / DATE

William G. Halley 6/17/97

ANII REVIEW / DATE

[Signature] 6-15-97

Code Category B-F:

Pressure Retaining Dissimilar Metal Welds

Item No. B5.10

Reactor Vessel Nozzle to Safe End Butt Welds, Nominal Pipe Size ≥ 4 "

- Relief Request RR 2-6, rev. 0 for item B5.10 previously submitted and approved with Conditions (Ref. TAC No. 72247, pages 20-22, dated November 7, 1991). Examination of the Reactor Vessel Nozzle-to-Safe End Butt Welds were performed in accordance with the conditions as stipulated in the approved Relief Request.

Code Category B-F:

Pressure Retaining Dissimilar Metal Welds

Item No. B5.40

**Pressurizer Nozzle-to-Safe End Butt Welds
Nominal Pipe Size \geq 4"**

- Relief Request RR 2-23, rev. 0 for item B5.40 previously submitted and approved (Ref. TAC No. M82269, dated December 21, 1994).

Code Category B-F:

Pressure Retaining Dissimilar Metal Welds

Item No. B5.70

**Steam Generator Nozzle-to-Safe End Butt Welds
Nominal Pipe Size ≥ 4 "**

- Relief Request RR 2-24, rev. 0 for item B5.70 previously approved (Ref. TAC No. M82269, pages 16-17, dated December 21, 1994)

Code Category B-J: Pressure Retaining Welds in Piping

Item No. B9.10 Nominal Pipe Size \geq 4 in.
Item No. B9.11 Circumferential Welds

Two Relief Requests were previously submitted and approved:

- Relief Request RR 2-10, rev. 0 for item B9.11 previously approved with conditions (Ref. TAC No. 72247, pages 20-22, dated November 7, 1991) for 8 Reactor Coolant Pipe Circumferential Welds off the Hotleg and Coldleg. Examination of the Reactor Vessel Nozzle-to-Safe End Butt Welds were performed in accordance with the conditions as stipulated in the approved Relief Request.
- Relief Request RR 2-25, rev. 0 for item B9.11 previously approved for 17 piping welds (Ref. TAC No. M82269, pages 17-19, dated December 21, 1994)

Relief Request RR 2-Closeout-4, Rev. 0

Code Category B-J: **Pressure Retaining Welds in Piping**

Item No. B9.30 **Branch Pipe Connection Welds**
Item No. B9.31 **Nominal Pipe Size \geq 4"**

Relief is requested in accordance with the provisions of 10 CFR 50.55a (g)(5)(iii) from the full Code-required extent of volumetric examination for Branch Pipe Connection Welds (Nominal Pipe Size \geq 4").

Examination Requirements

ASME Section XI, 1983 Edition through Summer 1983 Addenda, requires the lower 1/3t of the weld examined volumetrically up to and including adjacent base metal 1/4" from the weld toe on either side of the weld.

Basis for Relief

Complete inspection of the Code-required volume for components listed in Table 4 was not possible due to restricted access caused by various design and geometry limitations. Approximately two thirds (2/3) of the required examinations were performed during the 1st and 2nd Periods of the Interval (from 1989 to 1992) and do not have the detailed measurements necessary to estimate the percentage of examination coverage. This was standard practice in the Industry at that time. The examinations were performed to the maximum extent possible using the available technologies. The type and location of limitations are also recorded on the data sheets, as approved for other B-J welds referenced in Tables 5A thru 5C. In most cases, the limitation is caused by the weld's proximity to other components such as a valve, integrally welded support, or an adjacent nozzle. When a one-sided exam is recorded or a limited exam is confirmed as a one-sided exam (no other restrictions) through the review of examination data and drawing verification, a 50% exam coverage credit is taken. If a limited or restricted exam is noted but not enough information exists to derive a percentage of coverage, it will be so noted.

Proposed Alternative

1. No additional volumetric and/or surface examinations will be performed on these welds. The components listed in this relief request have been examined to the maximum extent practical.
2. A visual inspection (VT-2) is performed in conjunction with the pressure testing conducted on these components every refuel outage (with no leakage detected) in accordance with IWA-5000 and IWB-5000, which provides reasonable assurance of component integrity.

Drawings and sketches which illustrate the restricted conditions encountered, (in some cases typical for a given configuration), that limited examination coverage are in Enclosure 4.

Table 4

Code Category B-J / Item No. B9.31					
INT No.	Weld ID	System / Component	Extent Examined	Limitation	Remarks
1-4101	16BC	Accumulator Discharge	50% ⁽¹⁾	Nozzle Configuration restricts 7&8 scan	10" Accumulator Discharge Line 351
1-4302	33BC	Reactor Coolant Pipe	50% ⁽¹⁾	Weld configuration limits 2 scan	Loop 33 10" PLOCAP
1-4401	13BC	Accumulator Discharge	50% ⁽¹⁾	Branch connection weld configuration limits 2, 7 & 8 scans. Nozzle configuration limits 5 scan.	10" Accumulator Discharge Line 350

NOTES:

(1) A 50% exam coverage credit is taken for these types of geometry limitations.

ENCLOSURE 4

**B-J, B9.31
(7 pages)**

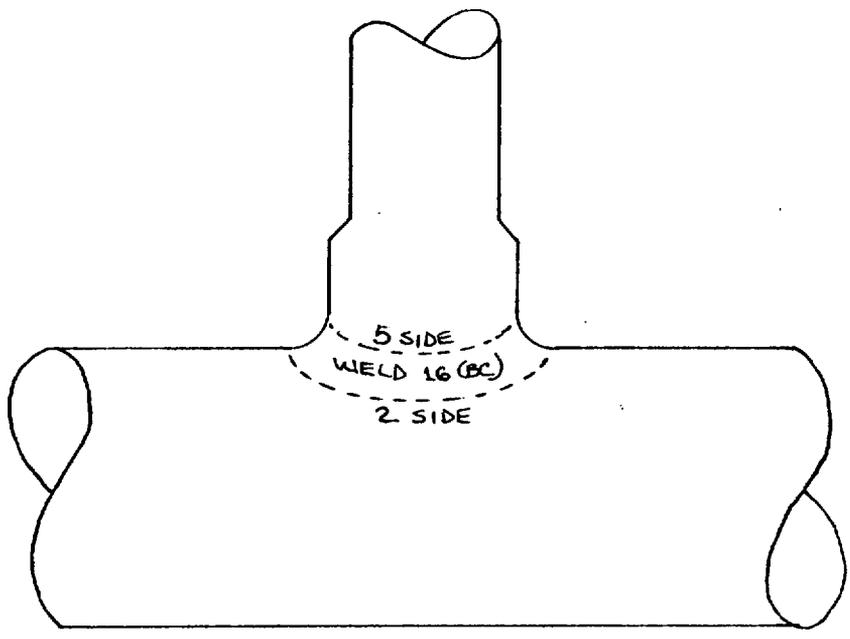
WESTINGHOUSE NUCLEAR SERVICE DIVISION
INSPECTION SERVICES

LIMITATION TO EXAMINATION

PLANT INDIAN POINT UNIT NO. 3 SKETCH INT-1-4101
10" LINE 351
SYST/COMP ACCUMULATOR DISCHARGE PROCEDURE INT-ISI-206, REV. 0
EXAMINER James R. Dellmoro DATE 2-14-89
LEVEL II

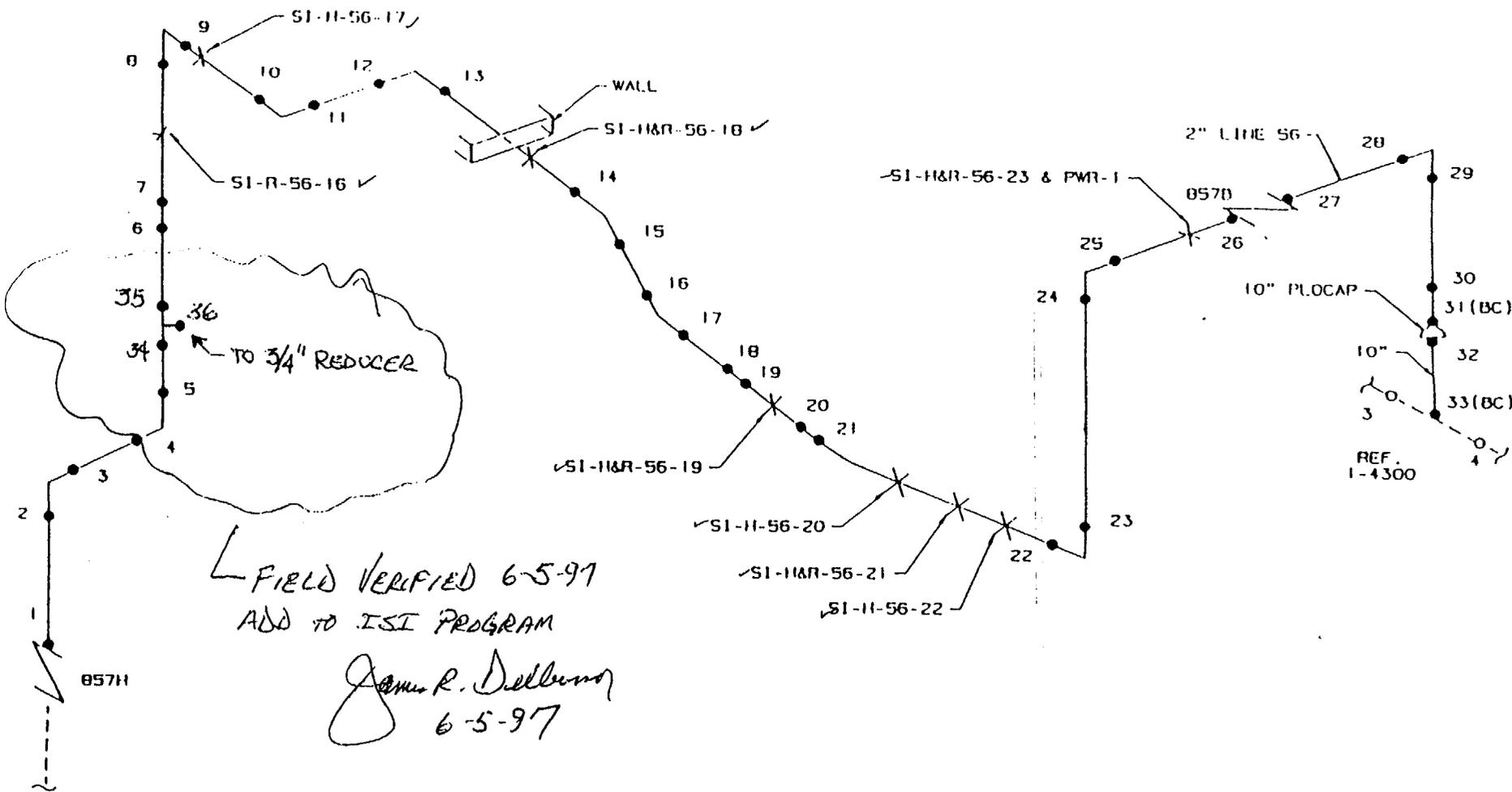
RELATED TO: U/T P/T _____ M/T _____ V/T _____ ITEM(S): WELD 16 (BC)

PROVIDE GENERAL INFORMATION TO DESCRIBE APPROXIMATE SIZE, LOCATION AND TYPE OF LIMITATION.



NOZZLE CONFIGURATION RESTRICTS 7/8 SCANS

APS/KSB
2-23-89



FIELD VERIFIED 6-5-97
 ADD TO ISI PROGRAM

James R. Dellburn
 6-5-97

REF.
 2-2540

PIPING:
 10" SCH 140 1.00" T STAINLESS STEEL
 2" SCH 160 .344" T STAINLESS STEEL

VALVE BONNET BOLTING:
 857B: NOT APPLICABLE
 857H: NOT APPLICABLE

VALVE MANUFACTURER:
 857B: NOT APPLICABLE
 857H: NOT APPLICABLE

INTEGRALLY WELDED ATTACHMENTS:
 PWR-1: 0.250" T

NEW YORK POWER AUTHORITY INDIAN POINT UNIT NO. 3	
LOOP 33 10" PLOCAP LOOP 33 2" LINE 56 SIS HOTLEG	
INT-1-4302	REV. 5

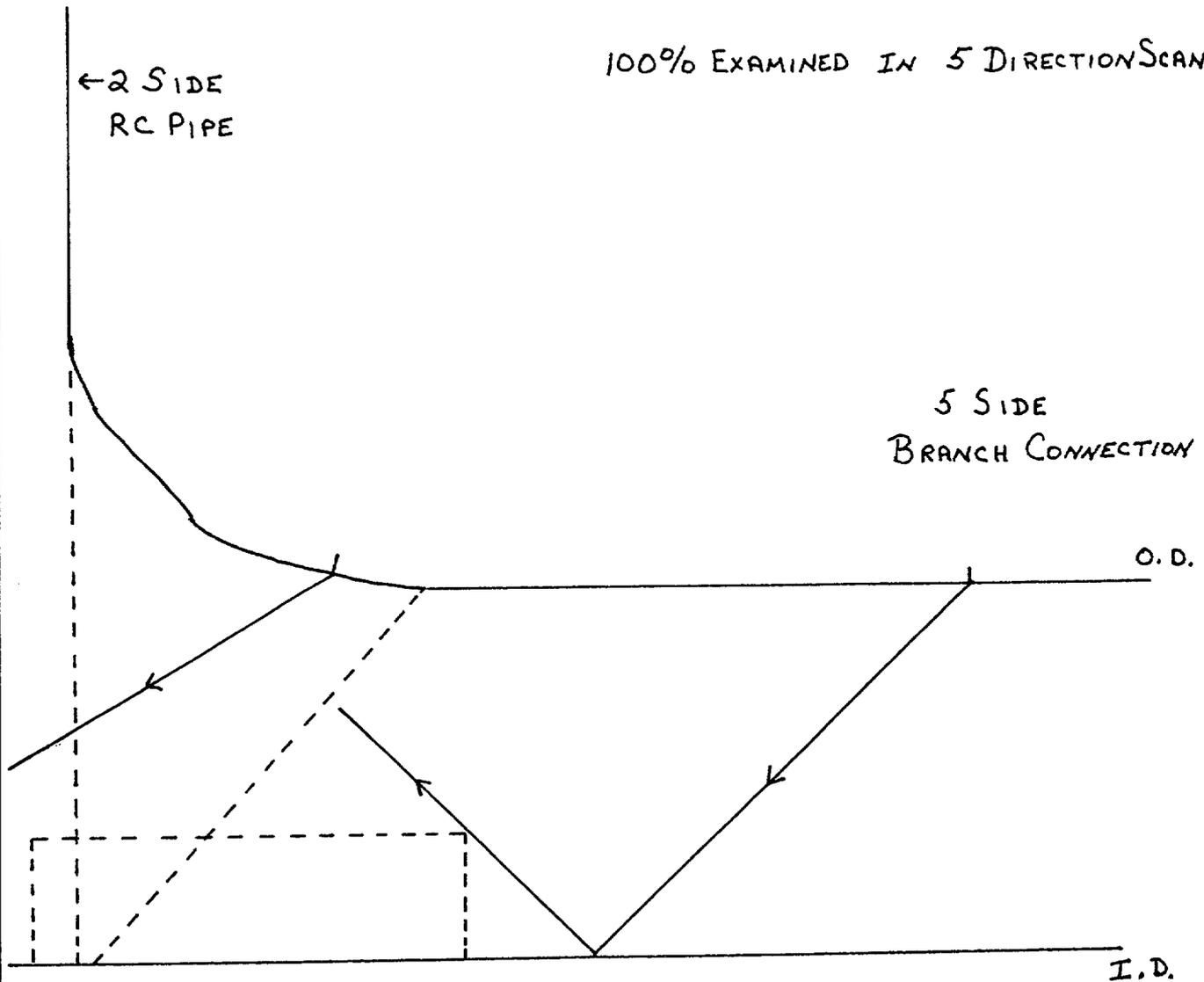
WESTINGHOUSE NUCLEAR SERVICE DIVISION
INSPECTION SERVICES

LIMITATION TO EXAMINATION

PLANT INDIAN POINT UNIT NO. 3 SKETCH INT-1-4302 REV. 5
SYST./COMP. LOOP 33 10" PLOCAP PROCEDURE INT-ISI-206, REV. 1, F.C. 01
EXAMINER *William G. Halley* DATE 5-28-97
LEVEL II

RELATED TO: U/T X P/T _____ M/T _____ V/T _____ ITEM(S) 33 (BC)

PROVIDE GENERAL INFORMATION TO DESCRIBE APPROXIMATE SIZE, LOCATION AND TYPE OF LIMITATION.



NEW YORK POWER AUTHORITY LEVEL III REVIEW / DATE
W. G. Halley 6/9/97

ANII REVIEW / DATE
SP Jett 6-13-97

WESTINGHOUSE NUCLEAR SERVICE DIVISION
INSPECTION SERVICES

LIMITATION TO EXAMINATION

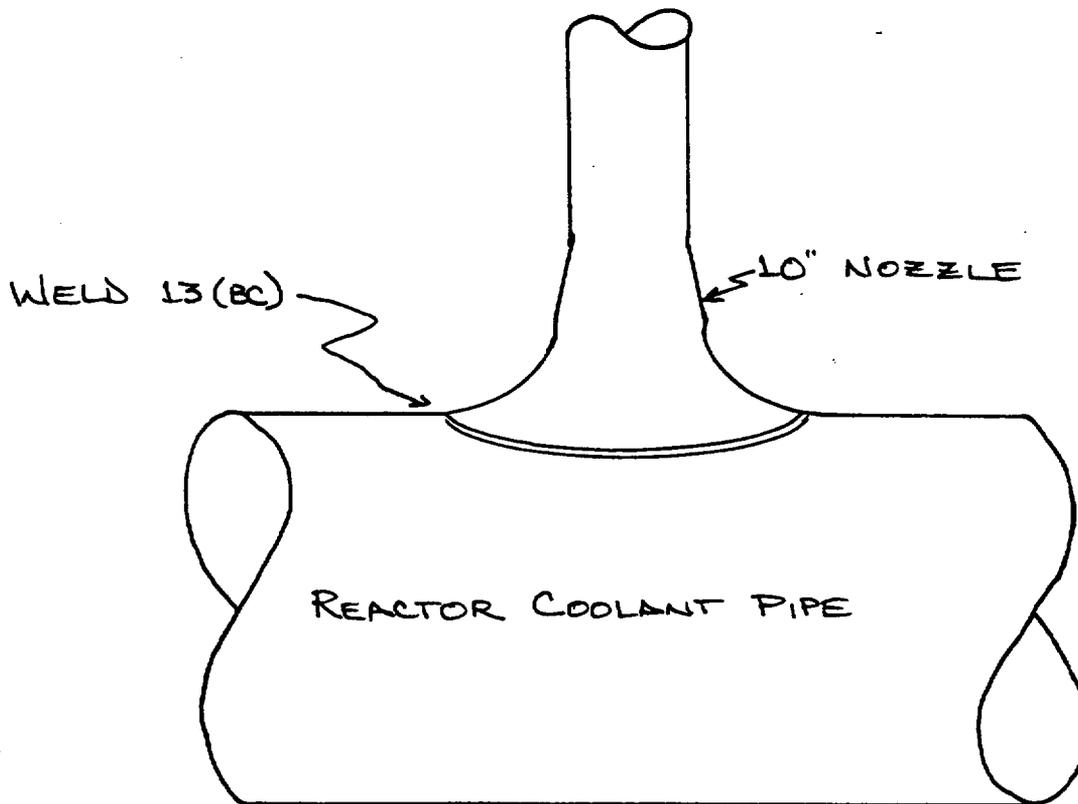
PLANT INDIAN POINT UNIT No 3 SKETCH INT-1-4401
SYST/COMP 10' ACCUMULATOR LINE 350 PROCEDURE INT-ISI-206, REV 1
EXAMINER Jamir R. Dellano William V. Talley DATE 10-2-90
LEVEL II

RELATED TO: U/T P/T _____ M/T _____ V/T _____ ITEM(S): WELD 13 (BC)

PROVIDE GENERAL INFORMATION TO DESCRIBE APPROXIMATE SIZE, LOCATION AND TYPE OF LIMITATION.

BRANCH CONNECTION WELD CONFIGURATION LIMITS 2, 7; 8 SCANS

NOZZLE CONFIGURATION RESTRICTS 5 SCAN.



MS/MSB
10-15-90

[Signature] 10/13/90
SITE NDE LEVEL III

Relief Request RR 2-Closeout-5, Rev. 0

Code Category B-K-1: Integral Attachments for Piping, Pumps, and Valves

Item No. B10.20 Pump Integrally Welded Attachments

Relief is requested pursuant to the provisions of 10 CFR 50.55a (g)(5)(iii), and 10 CFR 50.55a (a)(3)(ii) from the ASME Boiler and Pressure Vessel Code surface or volumetric examination requirements of three (3) pump integrally welded attachments listed in Table 5.

Examination Requirements

ASME Code Section XI, 1983 Edition through Summer 1983 Addenda requires examinations to include the welded attachment of pumps to be examined by Examination Category B-K-1, and includes essentially 100% of the weld length, for those attachments whose base material design thickness is 5/8 in. and greater.

Basis For Relief

Code Case N-509, which was approved by USNRC through Reg. Guide 1.147, Rev. 12 was applied in selection of the examination population size. Integrally welded attachments (3 total) to the 32 Reactor Coolant Pump was examined. Due to restrictions from the lower support structure, portions of the weld (about 15") cannot be examined by the required surface examination method.

Proposed Alternative Examinations

1. No additional volumetric and/or surface examinations will be performed on these welds. The components listed in Table 8 of this relief request have been examined to the maximum extent practical.
2. A visual inspection (VT-2) is performed in conjunction with the pressure testing conducted on these components every refuel outage (with no leakage detected) in accordance with IWA-5000 and IWB-5000, which provides reasonable assurance of component integrity.

Table 5

Code Category B-K-1 / Item No. B10.20					
INT No.	Weld ID	System / Component	Extent Examined	Limitation	Remarks
1-5100	32-1SC	Reactor Coolant	50% ⁽¹⁾	Lower support structure limits exam on bottom of weld	32 RCP
1-5100	32-2SC	Reactor Coolant	50% ⁽¹⁾	Lower support structure limits exam on bottom of weld	32 RCP
1-5100	32-3SC	Reactor Coolant	50% ⁽¹⁾	Lower support structure limits exam on bottom of weld	32 RCP

Note: 1. Surface examination was limited by the lower support structure which limits access to about 15” of the bottom of the weld. Examination was performed to the maximum extent possible and a 50% credit for coverage is taken.

ENCLOSURE 5

**B-K-1, B10.20
(3 pages)**

WESTINGHOUSE NUCLEAR SERVICE DIVISION
INSPECTION SERVICES

LIMITATION TO EXAMINATION

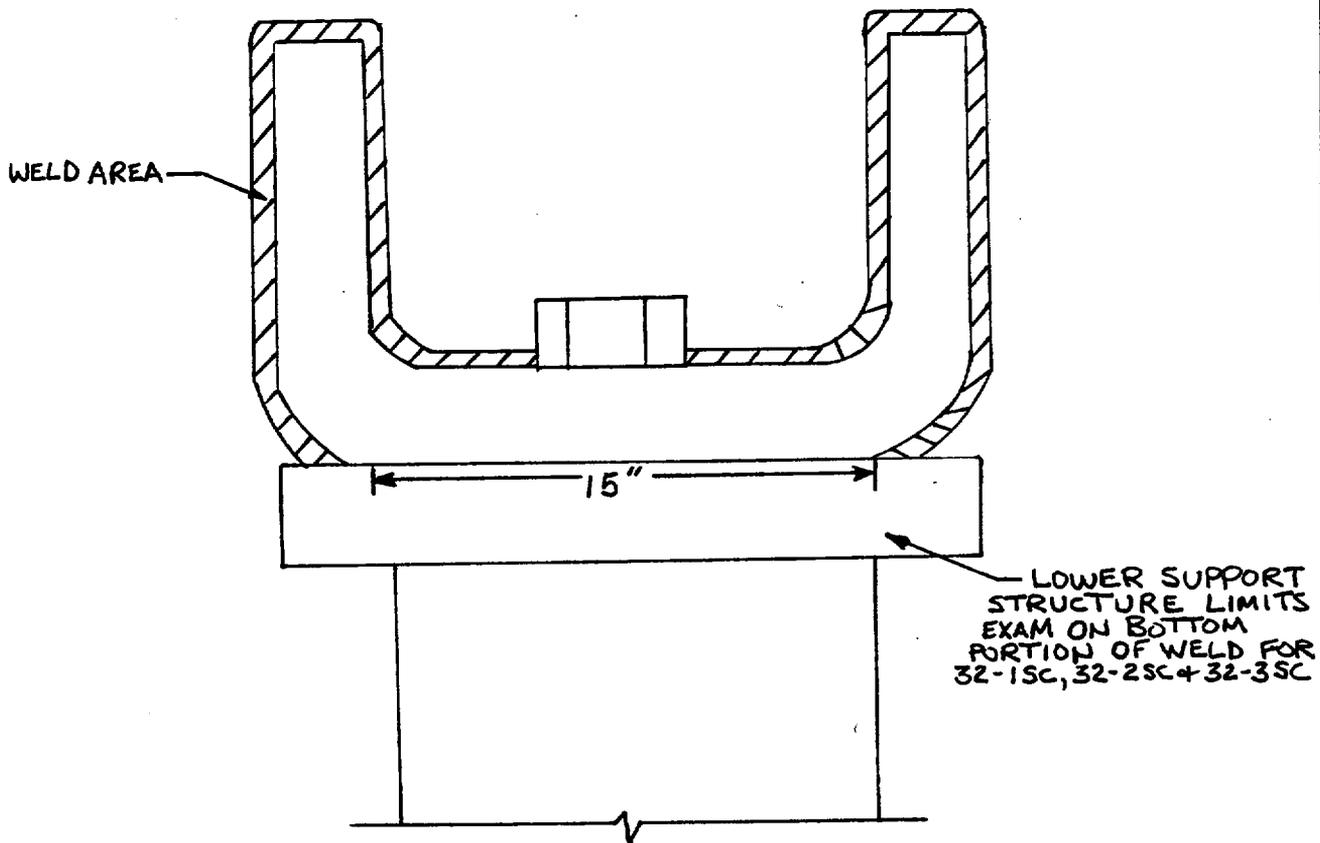
PLANT INDIAN POINT UNIT NO. 3 SKETCH INT-1-5100

SYST/COMP REACTOR COOLANT PUMP RCPCP2-32 PROCEDURE INT-TSI-11 REV. 1

EXAMINER Robert Conant Bill A. S. DATE 10-27-90 / 10-28-90
LEVEL II

RELATED TO: U/T _____ P/T M/T _____ V/T _____ ITEM(S): 32-1SC, 32-2SC + 32-3SC

PROVIDE GENERAL INFORMATION TO DESCRIBE APPROXIMATE SIZE, LOCATION AND TYPE OF LIMITATION.



SSG
10/31/90

APD/HAB
11-2-90

CLASS 2 COMPONENTS

Relief Request RR 2-Closure-6, Rev. 0

Code Category C-A: **Pressure Retaining Welds in Pressure Vessels**

Item No. C1.10 **Shell Circumferential Welds**
Item No. C1.20 **Head Circumferential Welds**
Item No. C1.30 **Tubesheet-to-Shell Welds**

Relief is requested in accordance with the provisions of 10 CFR 50.55a (g)(5)(iii) from the full Code-required extent of volumetric examination for the component welds listed in Tables 6A through 6C.

Examination Requirements

ASME Section XI requires 100% volumetric examination of this weld once per interval.

Basis For Relief

Complete inspection of the Code-required volume is not possible based on restricted access caused by interference due to the close proximity of the vessel flange bolting, nozzles and support clamps. Drawings and sketches which illustrate the restricted conditions encountered that limit examination coverage are included in Enclosure 6.

Examination Alternative

1. No additional volumetric examinations will be performed on these welds. The component listed in this relief request has been examined to the maximum extent practical.
2. A visual inspection (VT-2) was performed during the pressure testing which is conducted on this component every refuel outage (no leakage detected) in accordance with IWA-5000 and IWC-5000, which provides reasonable assurance of component integrity.

Table 6A

Code Category C-A / Item No. C1.10					
INT No.	Weld ID	System / Component	Extent Examined	Limitation	Remarks
2-1210	32-3	32 Accumulator Tank	50% ⁽¹⁾	Weld crown, welded lug, nozzle, head to shell configuration and long seam	32 Accum. Tank
2-1320	2	Seal Water Return Filter	50% ⁽¹⁾	Welded support and lower head configuration	SW Return Filter

NOTES: (1) A 50% exam coverage credit is taken for these types of geometry limitations.

Table 6B

Code Category C-A / Item No. C1.20					
INT No.	Weld ID	System / Component	Extent Examined	Limitation	Remarks
2-1110	9	31 Regen Heat Exchanger	50% ⁽¹⁾	2 adjacent nozzles	Regen HX
2-1200	2	Volume Control Tank	50% ⁽¹⁾	Head Configuration, weld crown manway, nozzle	VCT
2-1210	32-4	32 Accumulator Tank	50% ⁽¹⁾	Weld crown, welded lug, nozzle, head to shell configuration and long seam	32 Accum. Tank
2-1320	1	Seal Water Return Filter	50% ⁽¹⁾	Weld crown and flange configuration	SW Return Filter
2-1320	3	Seal Water Return Filter	50% ⁽¹⁾	Weld tag and flange configuration	SW Return Filter

NOTES: (1) A 50% exam coverage credit is taken for these types of geometry limitations.

Table 6C

Code Category C-A / Item No. C1.30					
INT No.	Weld ID	System / Component	Extent Examined	Limitation	Remarks
2-1101	31-2	31 Steam Generator	50% ⁽¹⁾	1 handholes and 2 nozzles	S/G
2-1110	10	31 Regen Heat Exchanger	50% ⁽¹⁾	2 adjacent nozzles and support clamp	Regen HX
2-1110	11	31 Regen Heat Exchanger	92% - 45° 93% - 2 scan 73% - 5 scan	1 adjacent nozzle and support clamp	Regen HX

NOTES: (1) A 50% exam coverage credit is taken for these types of geometry limitations.

ENCLOSURE 6

**C-A, C1.10, C1.20, C1.30
(34 pages)**

WESTINGHOUSE NUCLEAR SERVICE DIVISION INSPECTION SERVICES				PLANT INDIAN POINT				UNIT No. 3				SKETCH INT-2-1101								
WELD ULTRASONIC EXAMINATION INSERVICE				SYST/COMP. REPLACEMENT STEAM GENERATOR RCPCSGI-31				PROCEDURE INT-ISI-47 REV. 1												
EXAMINER (LEVEL II) <i>William J. Valley</i>				DATE 10-24-90																
EQUIPMENT	TRANSDUCER	STRAIGHT BEAM SCAN DIRECTION 0		AXIAL SCANS DIRECTIONS 2 & 5				CIRCUMFERENTIAL SCANS DIRECTIONS 7 & 8				CALIBRATION CHECK								
INST. S/N SONIC MK I SN: 11222E	S/N	C26673										INITIAL TIME								
	SIZE	1.0"										0940								
	FREQ.	2.25 MHZ																		
	ANGLE	0°																		
REP. RATE	CALIBRATION REFLECTOR LOCATION	SIGNAL AMPLITUDE	SWEEP POSITION	SIGNAL AMPLITUDE	SWEEP POSITION	DISTANCE FROM SCRIBE/REF. LINE TO		SIGNAL AMPLITUDE	SWEEP POSITION	DISTANCE FROM SCRIBE/REF. LINE TO										
1K						PEAK	50% DAC LOCATIONS			PEAK	50% DAC LOCATIONS									
REJECT	1/4T	80%	1.5																	
DAMPING	1/2T	80%	3.0																	
FILTER	3/4T	70%	4.5																	
LIN. CHECK	1T		6.0													FINAL TIME 1114				
SU CABLE													CAL. BLOCK IP-3							
COUPLANT													THICKNESS 3.5"							
BATCH #	8979				33 dB								TEMP. 80°F							
WELD NUMBER	TEMP.	BASE METAL SCAN	SCAN DIRECTION 2 5 7/8 0				EXAMINATION LIMITATIONS				CROWN CONFIGURATION			RESULTS NI NRI RI			REMARKS			
31-2	87°F					YES	1 HANDHOLE - SEE ATTACHED LIMITATION TO EXAMINATION SHEET				SLIGHTLY ROUNDED			<input checked="" type="checkbox"/>			AREA EXAMINED 0" CLOCKWISE TO 135" FROM 0 REFERENCE EXAMINED 100%			
31-3	87°F					YES	2 HANDHOLES AND TRANSITION SEE ATTACHED LIMITATION TO EXAMINATION SHEET				SLIGHTLY ROUNDED			<input checked="" type="checkbox"/>						
																	SURFACE THERMOMETER SN: 1979			
<i>ASD</i>																				
10/31/90																				
SITE NDE LEVEL III																				

WESTINGHOUSE NUCLEAR SERVICE DIVISION INSPECTION SERVICES WELD ULTRASONIC EXAMINATION INSERVICE				PLANT		UNIT		SKETCH											
				INDIAN POINT		NO. 3		INT-2-1101											
				SYST/COMP.		PROCEDURE		REV. 1											
				REPLACEMENT S/G RCP CSG 1-31		INT-IST-47		DATE											
				EXAMINER (LEVEL II)		10-24-90													
				ROBERT S. CASATI JAMES R. DEBBARDY CLYDE W. HALL															
EQUIPMENT		TRANSUCER		STRAIGHT BEAM SCAN DIRECTION 0				AXIAL SCANS DIRECTIONS 2 & 5		CIRCUMFERENTIAL SCANS DIRECTIONS 7 & 8		CALIBRATION CHECK							
INST. S/N SONIC MK 1		S/N		B06627				B06627		INITIAL TIME		0910							
		SIZE		0.5" X 1.0"				0.5" X 1.0"											
		FREQ.		2.25 MHZ				2.25 MHZ											
S/N 05940E		ANGLE		60°				60°											
REP. RATE		CALIBRATION REFLECTOR LOCATION		SIGNAL AMPLITUDE		SWEEP POSITION		DISTANCE FROM SCRIBE/REF. LINE TO PEAK		50% DAC LOCATIONS		SIGNAL AMPLITUDE		SWEEP POSITION		DISTANCE FROM SCRIBE/REF. LINE TO PEAK		50% DAC LOCATIONS	
3K		1/4 T		80%		1.5		1.55"		1.25" 1.9"		80%		1.5		1.55"		1.25" 1.9"	
REJECT																			
OFF																			
DAMPING		1/2 T		42%		3.0		3.15"		2.55" 3.7"		42%		3.0		3.15"		2.55" 3.7"	
MIN																			
FILTER		3/4 T		26%		4.5		4.6"		2.6s 3.5s		26%		4.5		4.6"		2.6s 3.5s	
HI																			
LIN. CHECK		1 T (NOTCH)		10%		6.1		6.05"		4.1s 5.3s		10%		6.1		6.05"		4.1s 5.3s	
SAT																			
SU CABLE		1/4 T		15%		7.5						15%		7.5					
12' BNC-BNC																			
COUPLANT		CAL. GAIN		48dB				48dB				CAL. BLOCK		IP-3					
SONOTRACE 40																			
BATCH # 8979																		THICKNESS 3.5"	
																		TEMP. 80°F	
WELD NUMBER		TEMP.		SCAN DIRECTION				EXAMINATION LIMITATIONS				CROWN CONFIGURATION		RESULTS			REMARKS		
				2 5 7/8 0										NI NRI RI					
31-2		87°F		YES YES YES				4" NOZZLE, 2" NOZZLE + HANDHOLE.				SLIGHTLY ROUNDED		X			EXAMINED AREA FROM 0° COUNTERCLOCKWISE TO 135° FROM 0 REFERENCE.		
31-3		87°F		YES YES YES				2 HANDHOLES + TRANSITION				SLIGHTLY ROUNDED		X			EXAMINED 100%		
								SEE ATTACHED LIMITATION TO EXAMINATION SHEET FOR WELDS 31-2 + 31-3									SURFACE THERMOMETER S/N 1979		

SS

12/31/90

SITE NDE LEVEL III

MSH/B

11/2-90

WESTINGHOUSE NUCLEAR SERVICE DIVISION
INSPECTION SERVICES

LIMITATION TO EXAMINATION

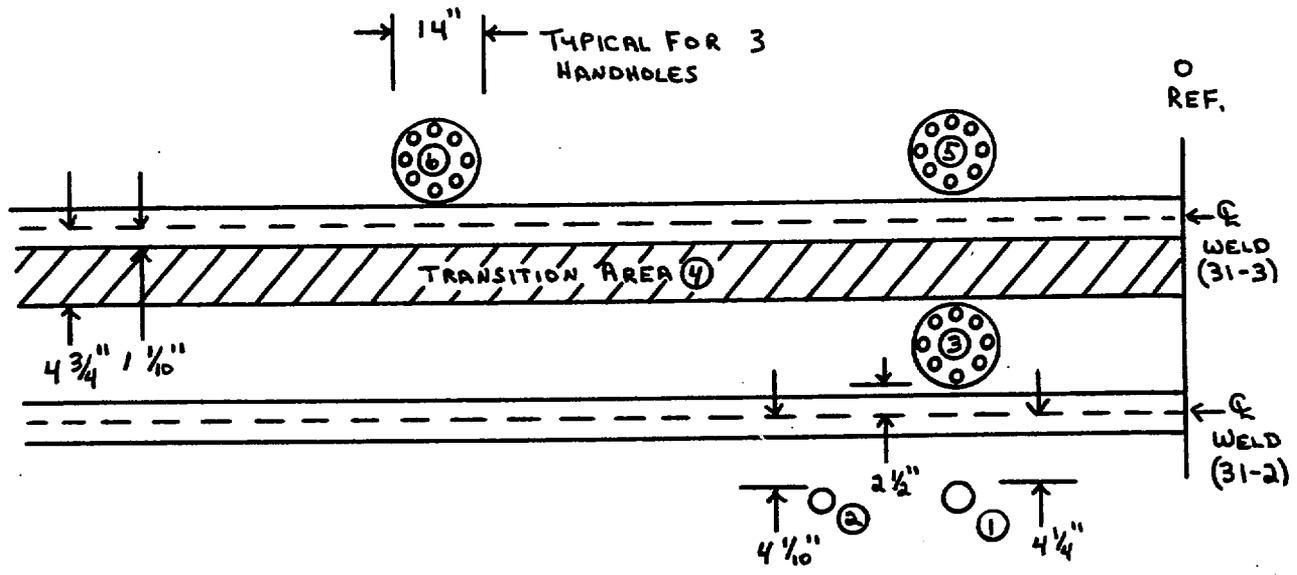
PLANT INDIAN POINT UNIT No. 3 SKETCH INT-2-1101

SYST/COMP REPLACEMENT STEAM GENERATOR RCPCSG1-31 PROCEDURE INT-ISI-47 REV. 1

EXAMINER William J. Kelly James E. Dellinger DATE 10-24-90
LEVEL II Robert J. Casant

RELATED TO: U/T P/T _____ M/T _____ V/T _____ ITEM(S): 31-2 ; 31-3

PROVIDE GENERAL INFORMATION TO DESCRIBE APPROXIMATE SIZE, LOCATION AND TYPE OF LIMITATION.



- ① 4" NOZZLE 4'5" FROM O REF; LIMITS 45°+60° 2 SCANS (WELD 2)
- ② 2" NOZZLE 9'5 1/2" FROM O REF; LIMITS 45°+60° 2 SCANS (WELD 2)
- ③ HANDHOLE 4'5" FROM O REF; LIMITS 0°, 45°+60° 5, 7+8 SCANS (WELD 2)
- ④ TRANSITION LIMITS 0°, 45°+60° 2, 7+8 SCANS (WELD 3)
- ⑤ HANDHOLE 4'5" FROM O REF; LIMITS 0°, 45°+60° 5, 7+8 SCANS (WELD 3)
- ⑥ HANDHOLE 21'2 1/2" FROM O REF; LIMITS 0°, 45°+60° 5, 7+8 SCANS (WELD 3)

[Signature]
10/31/90

ADS/HKB
11-2-90

152

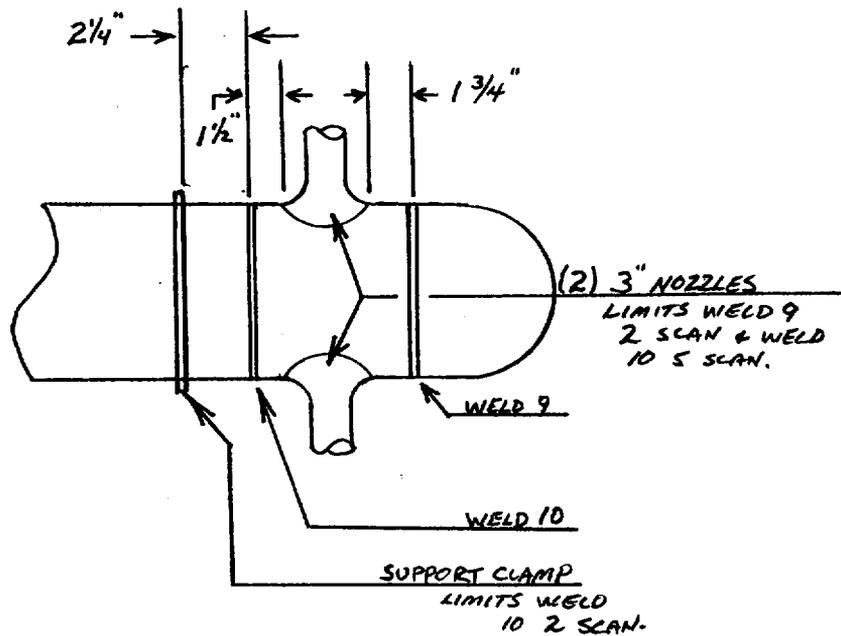
WESTINGHOUSE NUCLEAR SERVICE DIVISION
INSPECTION SERVICES

LIMITATION TO EXAMINATION

PLANT INDIAN POINT UNIT No. 3 SKETCH INT-2-1110
CSAHRG1-31
SYST/COMP REGENERATIVE HEAT EXCHANGER PROCEDURE INT-151-206 REV. 0
EXAMINER Kenneth J. Dellmann DATE 2-14-89
LEVEL II

RELATED TO: U/T P/T _____ M/T _____ V/T _____ ITEM(S): 9 & 10

PROVIDE GENERAL INFORMATION TO DESCRIBE APPROXIMATE SIZE, LOCATION AND TYPE OF LIMITATION.



AKS/MSB
2-28-89

WESTINGHOUSE NUCLEAR SERVICE DIVISION INSPECTION SERVICES WELD ULTRASONIC EXAMINATION INSERVICE	PLANT	UNIT	SKETCH
	INDIAN POINT	NO. 3	INT-2-1110 REV. 4
	SYST. / COMP	PROCEDURE	
	REGENERATIVE HEAT EXCHANGER CSAHRG1-31	INT-ISI-206 REV. 1 F.C. 01	
EXAMINER (LEVEL II)	<i>James R. Dellorusso</i>		DATE
			6-28-97

EQUIPMENT	TRANSDUCER	STRAIGHT BEAM SCAN DIRECTION 0	AXIAL SCANS DIRECTIONS 2 & 5				CIRCUMFERENTIAL SCANS DIRECTIONS 7 & 8				CALIBRATION CHECK		
INST. S / N STAVELEY SONIC/136-896K	S / N	59698									INITIAL TIME	0748	
REP. RATE	SIZE	0.5"											
1K	FREQ.	5.0 MHZ											
REJECT	ANGLE	0°											
OFF													
DAMPING	CALIBRATION REFLECTOR LOCATION	SIGNAL AMPLITUDE	SWEEP POSITION	SIGNAL AMPLITUDE	SWEEP POSITION	DISTANCE FROM SCRIBE / REF. LINE TO 50% DAC LOCATIONS		SIGNAL AMPLITUDE	SWEEP POSITION	DISTANCE FROM SCRIBE / REF. LINE TO 50% DAC LOCATIONS			
500 OHMS						PEAK				PEAK			
FILTER	1/4T	80%	3.0										
1	3/4T	40%	6.0										
LIN. CHECK												FINAL TIME	1110
SAT													
S. U. CABLE RG-174 6' DUAL													
COUPLANT												CAL. BLOCK	INT-17
SONOTRACE 40 96343	CAL. GAIN	46.6dB										THICKNESS	0.906"
												TEMPERATURE	80°F

WELD NUMBER	TEMP.	BASE METAL SCAN	SCAN DIRECTION				EXAMINATION LIMITATIONS	CROWN CONFIGURATION	RESULTS			REMARKS
			2	5	7/8	0			NI	NRI	RI	
11	74°F	Y-A				YES	SEE LIMITATION TO EXAMINATION SHEET	FLAT	X			SURFACE THERMOMETER S/N 10063

NEW YORK POWER AUTHORITY LEVEL III REVIEW / DATE <i>W. Maly III</i> 6/30/97	ANI REVIEW / DATE <i>SP JH</i> 7-1-97
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WESTINGHOUSE NUCLEAR SERVICE DIVISION INSPECTION SERVICES WELD ULTRASONIC EXAMINATION INSERVICE	PLANT INDIAN POINT	UNIT NO. 3	SKETCH INT-2-1110 REV. 4
	SYST. / COMP REGENERATIVE HEAT EXCHANGER CSAHRG1-31		PROCEDURE INT-ISI-206 REV. 1 F.C. 01
	EXAMINER (LEVEL II) <i>Sam R. Dellano</i>		DATE 6-28-97

EQUIPMENT	TRANSDUCER	STRAIGHT BEAM SCAN DIRECTION 0	AXIAL SCANS DIRECTIONS 2 & 5				CIRCUMFERENTIAL SCANS DIRECTIONS 7 & 8				CALIBRATION CHECK		
INST. S / N STAVELEY SONIC/136-896K	S / N		002MB6				002MB6				INITIAL TIME	0749	
REP. RATE	SIZE		0.5"				0.5"						
1K	FREQ.		2.25 MHz				2.25 MHz						
REJECT	ANGLE		45°				45°						
OFF			45°				45°						
DAMPING	CALIBRATION REFLECTOR LOCATION	SIGNAL AMPLITUDE	SWEEP POSITION	SIGNAL AMPLITUDE	SWEEP POSITION	DISTANCE FROM SCRIBE / REF. LINE TO		SIGNAL AMPLITUDE	SWEEP POSITION	DISTANCE FROM SCRIBE / REF. LINE TO			
						PEAK	50% DAC LOCATIONS			PEAK	50% DAC LOCATIONS		
500 OHMS													
FILTER	1T			80%	3.0			80%	3.3				
1	2T			48%	6.0			50%	6.2				
LIN. CHECK	3T			28%	9.0								
SAT												FINAL TIME	1112
S. U. CABLE RG-174 12' BNC-MCD													
COUPLANT SONOTRACE 40 96343	CAL. GAIN			19dB				28.4dB				CAL. BLOCK	INT-17
												THICKNESS	0.906"
												TEMPERATURE	80°F

WELD NUMBER	TEMP.	BASE METAL SCAN	SCAN DIRECTION				EXAMINATION LIMITATIONS	CROWN CONFIGURATION	RESULTS			REMARKS
			2	5	7/8	0			NI	NRI	RI	
11	74°F		YES	NO	YES		SEE LIMITATION TO EXAMINATION SHEET	FLAT	X			SURFACE THERMOMETER S/N 10063

NEW YORK POWER AUTHORITY LEVEL III REVIEW / DATE <i>SPromley III 6/30/97</i>	ANII REVIEW / DATE <i>SP Jutta 7-1-97</i>
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WESTINGHOUSE NUCLEAR SERVICE DIVISION INSPECTION SERVICES	PLANT	INDIAN POINT	UNIT	NO. 3	SKETCH	INT-2-1110 REV. 4
	SYST. / COMP	REGENERATIVE HEAT EXCHANGER CSAHRG1-31			PROCEDURE	INT-ISI-206 REV. 1 F.C. 01
	EXAMINER (LEVEL II)	<i>Jamie R. Bellon</i>			DATE	6-28-97

**WELD ULTRASONIC EXAMINATION
INSERVICE**

EQUIPMENT	TRANSDUCER	STRAIGHT BEAM SCAN DIRECTION 0	AXIAL SCANS DIRECTIONS 2 & 5				CIRCUMFERENTIAL SCANS DIRECTIONS 7 & 8				CALIBRATION CHECK	
INST. S / N STAVELEY SONIC/138-896K	S / N		94 - 528								INITIAL TIME 0750	
REF. RATE	SIZE		2(4X7)mm									
1K	FREQ.		4.0 MHz									
REJECT OFF	ANGLE		60° L									
DAMPING	CALIBRATION REFLECTOR LOCATION	SIGNAL AMPLITUDE	SWEEP POSITION	SIGNAL AMPLITUDE	SWEEP POSITION	DISTANCE FROM SCRIBE / REF. LINE TO 50% DAC LOCATIONS		SIGNAL AMPLITUDE	SWEEP POSITION	DISTANCE FROM SCRIBE / REF. LINE TO 50% DAC LOCATIONS		
500 OHMS	1/4T			80%	1.8							
	3/4T			65%	5.7							
	NOTCH			20%	8.6							FINAL TIME 1114
FILTER 1												
LIN. CHECK SAT												
S. U. CABLE RG-174 12' BNC-MCD												
COUPLANT SONOTRACE 40 96343	CAL. GAIN			67dB								CAL. BLOCK INT-17
												THICKNESS 0.906"
												TEMPERATURE 80°F

WELD NUMBER	TEMP.	BASE METAL SCAN	SCAN DIRECTION				EXAMINATION LIMITATIONS	CROWN CONFIGURATION	RESULTS			REMARKS
			2	5	7/B	0			NI	NRI	RI	
11	74°F		YES	NO			SEE LIMITATION TO EXAMINATION SHEET	FLAT	X			SURFACE THERMOMETER S/N 10063

NEW YORK POWER AUTHORITY LEVEL III REVIEW / DATE

ANI REVIEW / DATE

Jamie R. Bellon LIA 6/30/97

SP JTA 7-1-97

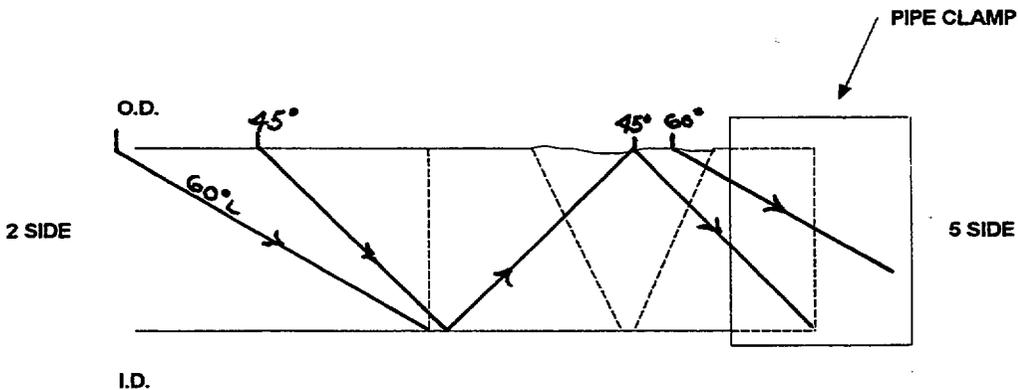
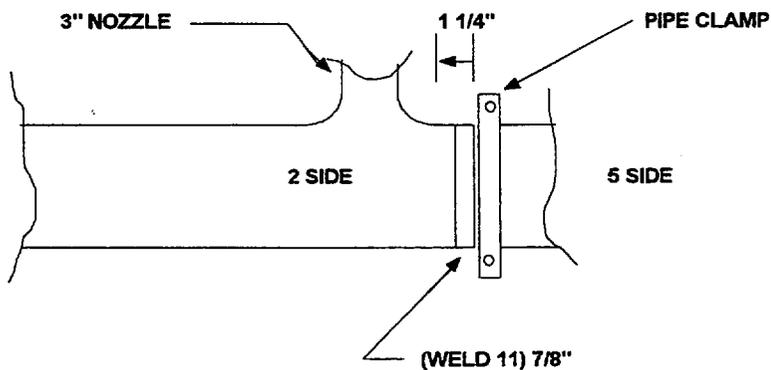
WESTINGHOUSE NUCLEAR SERVICE DIVISION
INSPECTION SERVICES

LIMITATION TO EXAMINATION

PLANT INDIAN POINT UNIT NO. 3 SKETCH INT-2-1110 REV. 4
SYST./COMP. REGENERATIVE HEAT EXCHANGER CSAHRG1-31 PROCEDURE INT-ISI-206, REV. 1 F.C. 01
EXAMINER *James R. DeLburne* DATE 6-28-97
LEVEL II

RELATED TO: U/T X P/T _____ M/T _____ V/T _____ ITEM(S) 11

PROVIDE GENERAL INFORMATION TO DESCRIBE APPROXIMATE SIZE, LOCATION AND TYPE OF LIMITATION.



NOZZLE CONFIGURATION LIMITS 2 SCAN FOR 2.5" FOR 45°, AND 2" FOR 60° L
PIPE CLAMP RESTRICTS ALL SCANS

92% EXAMINED 45°
93° IN 2 DIRECTION FOR 60° L
27% (BASE METAL 5 SIDE) NOT EXAMINED 0°

NEW YORK POWER AUTHORITY / LEVEL III REVIEW / DATE
[Signature] LIII 6/30/97

ANII REVIEW / DATE
[Signature] 7-1-97

WESTINGHOUSE NUCLEAR SERVICE DIVISION INSPECTION SERVICES			PLANT INDIAN POINT				UNIT NO. 3		SKETCH INT-2-1200			
WELD ULTRASONIC EXAMINATION INSERVICE			SYST/COMP. VOLUME CONTROL TANK CSATVCI-31				PROCEDURE INT-ISI-206 REV. 1					
			EXAMINER (LEVEL II) <i>Robert J. Casant</i>				DATE 10-2-90					
EQUIPMENT	TRANSDUCER	STRAIGHT BEAM SCAN DIRECTION 0	AXIAL SCANS DIRECTIONS 2 & 5				CIRCUMFERENTIAL SCANS DIRECTIONS 7 & 8				CALIBRATION CHECK	
INST. S/N SONIC MK 1	S/N	E09490									INITIAL TIME 1505	
	SIZE	0.25"										
	FREQ.	5.0 MHZ										
<i>SN 06208E</i>	ANGLE	0° PC										
REP. RATE 3K	CALIBRATION REFLECTOR LOCATION	SIGNAL AMPLITUDE	SWEEP POSITION	SIGNAL AMPLITUDE	SWEEP POSITION	DISTANCE FROM SCRIBE/REF. LINE TO		SIGNAL AMPLITUDE	SWEEP POSITION	DISTANCE FROM SCRIBE/REF. LINE TO		
REJECT OFF						PEAK	50% DAC LOCATIONS			PEAK	50% DAC LOCATIONS	
DAMPING MIN	1/2 T	80%	4.4									
FILTER HI	1 T		6.0									
LIN. CHECK SAT												FINAL TIME 1610
SU CABLE 6' BNC TO PC												CAL. BLOCK INT-36
COUPLANT SONOTRACE 40 BATCH # 8979												THICKNESS .250"
	CAL. GAIN	80dB										TEMP. 80°F
WELD NUMBER	TEMP.	BASE METAL SCAN	SCAN DIRECTION				EXAMINATION LIMITATIONS	CROWN CONFIGURATION	RESULTS			REMARKS
			2	5	7/8	0			NI	NRI	RI	
1	81°F					YES	WELD CONFIGURATION + WELDED SUPPORTS	ROUNDED	X			EXAMINED 100" CLOCKWISE TO 200" FROM 0 REFERENCE.
2	81°F					YES	WELD CONFIGURATION	ROUNDED	X			EXAMINED 100" CLOCKWISE TO 200" FROM 0 REFERENCE.
							SEE ATTACHED LIMITATION TO EXAMINATION SHEET FOR WELDS 1+2					SURFACE THERMOMETER SN 1973 <i>AGS/HES</i> 10-15-90

WESTINGHOUSE NUCLEAR SERVICE DIVISION
INSPECTION SERVICES

LIMITATION TO EXAMINATION

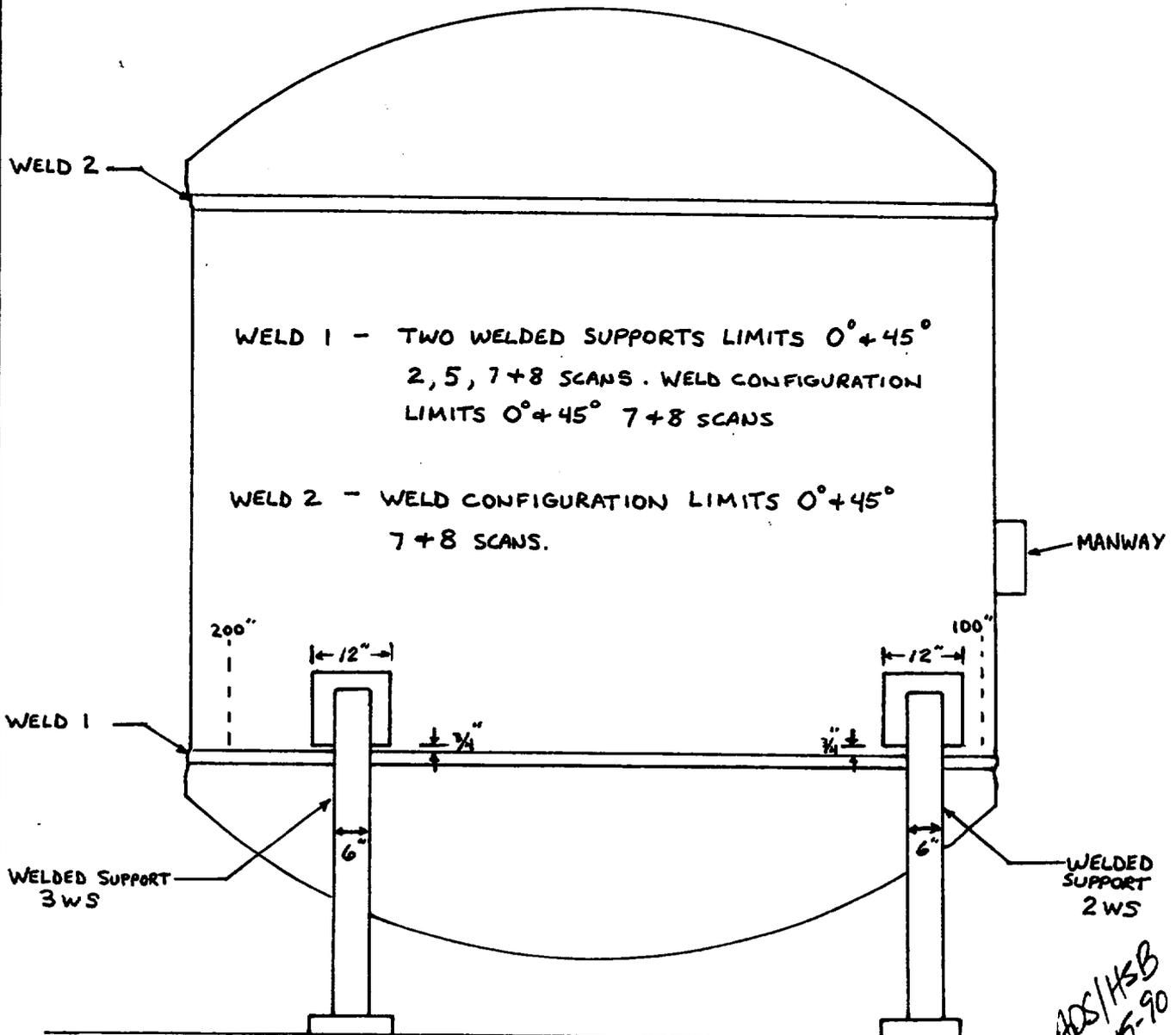
PLANT INDIAN POINT UNIT NO. 3 SKETCH INT-2-1200

SYST/COMP VOLUME CONTROL TANK CSATVCI-31 PROCEDURE INT-ISI-206 REV. 1

EXAMINER Robert J. Conant Bill HSB DATE 10-2-90
LEVEL II

RELATED TO: U/T P/T _____ M/T _____ V/T _____ ITEM(S): 1 + 2

PROVIDE GENERAL INFORMATION TO DESCRIBE APPROXIMATE SIZE, LOCATION AND TYPE OF LIMITATION.



WELD 1 - TWO WELDED SUPPORTS LIMITS 0° + 45°
2, 5, 7 + 8 SCANS. WELD CONFIGURATION
LIMITS 0° + 45° 7 + 8 SCANS

WELD 2 - WELD CONFIGURATION LIMITS 0° + 45°
7 + 8 SCANS.

HSB/HSB
10-15-90

JOSB
10/13/90
SITE NDE LEVEL III

WESTINGHOUSE NUCLEAR SERVICE DIVISION
INSPECTION SERVICES

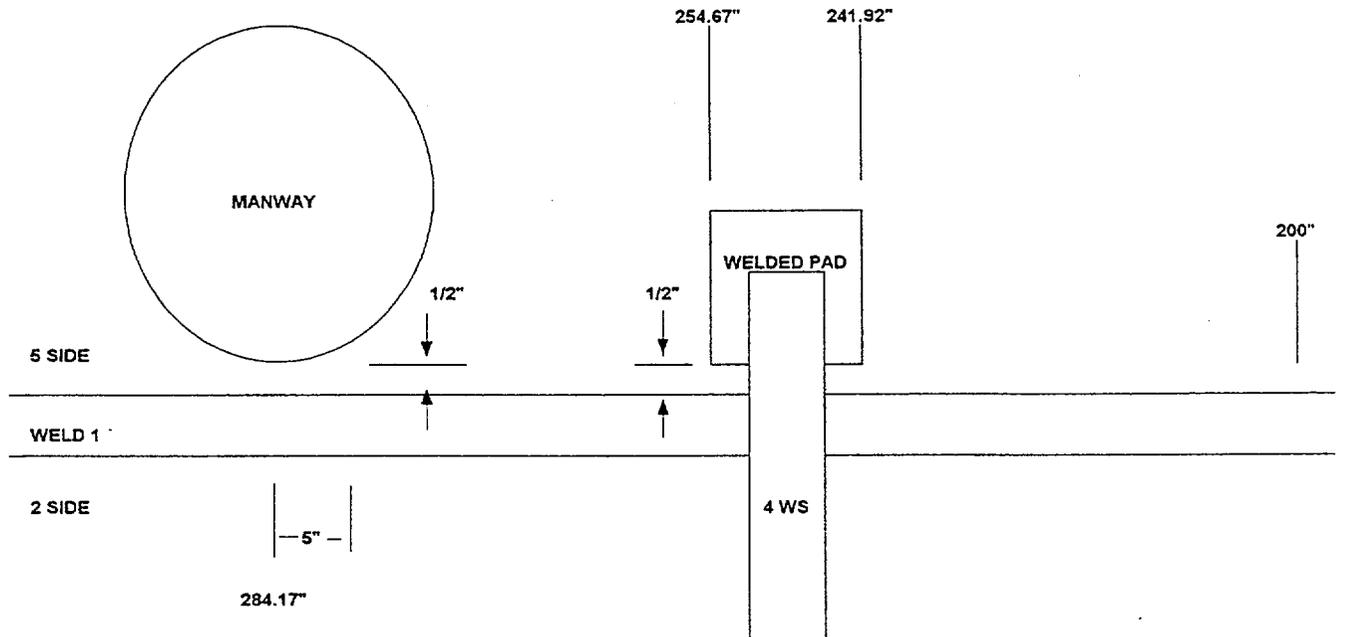
LIMITATION TO EXAMINATION

PLANT INDIAN POINT UNIT NO. 3 SKETCH INT-2-1200 REV. 3
SYST./COMP. VOLUME CONTROL TANK CSATVC1-31 PROCEDURE INT-4SI-206, REV. 1 F.C. 01
EXAMINER William H. Halley DATE 6-13-97
LEVEL II

RELATED TO: U/T X P/T _____ M/T _____ V/T _____ ITEM(S) 1

PROVIDE GENERAL INFORMATION TO DESCRIBE APPROXIMATE SIZE, LOCATION AND TYPE OF LIMITATION.

0 REFERENCE: CENTERLINE OF MANWAY



LIMITATION: MANWAY, SUPPORT (4 WS) AND WELDED PAD. 14% NOT EXAMINED

NEW YORK POWER AUTHORITY LEVEL III REVIEW / DATE
W. Halley 6/28/97

ANII REVIEW / DATE
JP JH 6-26-97

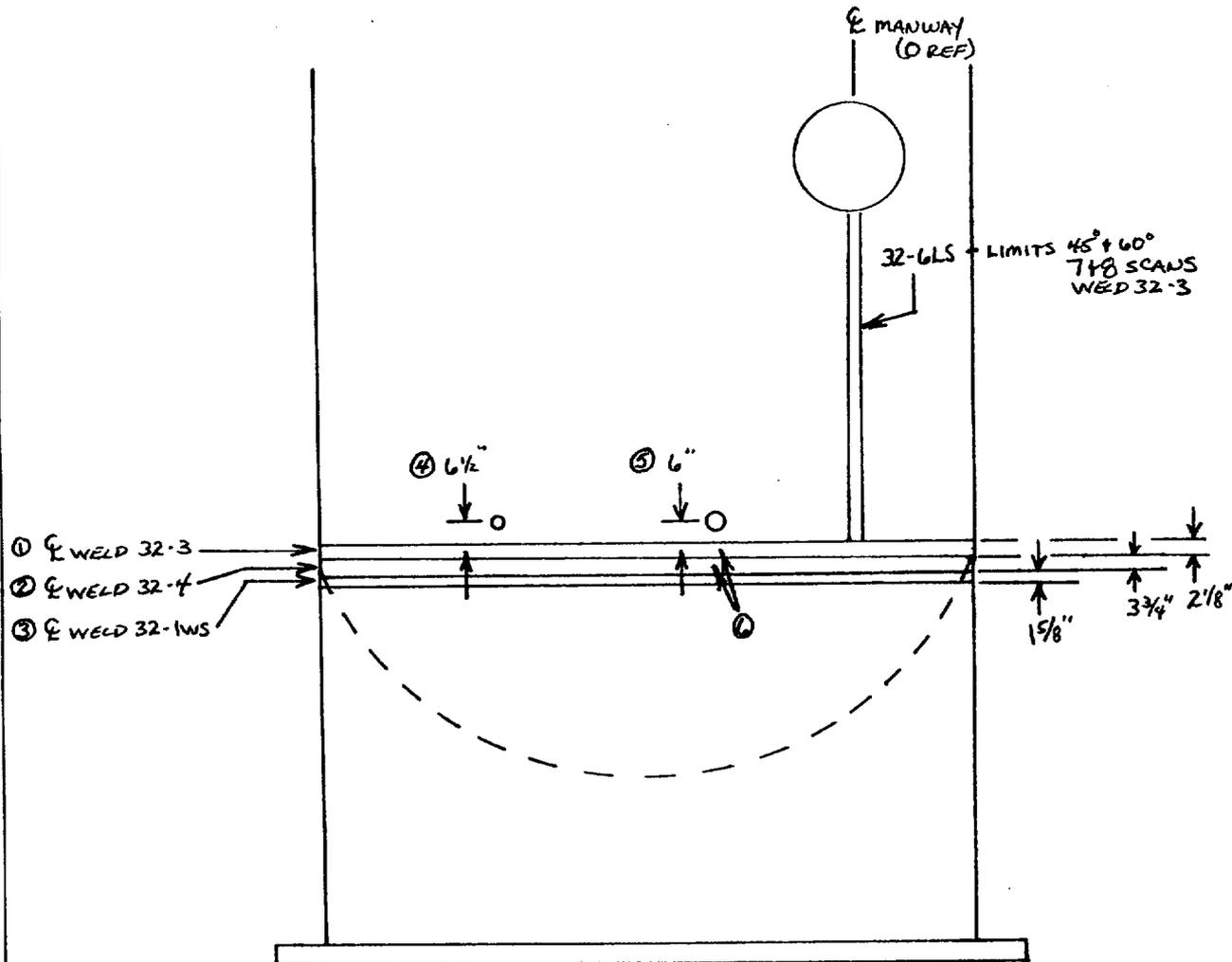
WESTINGHOUSE NUCLEAR SERVICE DIVISION
INSPECTION SERVICES

LIMITATION TO EXAMINATION

PLANT INDIAN POINT UNIT No. 3 SKETCH INT-2-1210
 SYST/COMP ACCUMULATOR TANK INTSIATAT2-32 PROCEDURE INT-151-47 REV.0
 EXAMINER Kurt J. Hostetler DATE 2-24-89
 LEVEL II

RELATED TO: U/T P/T _____ M/T _____ V/T _____ ITEM(S): 32-3; 32-4

PROVIDE GENERAL INFORMATION TO DESCRIBE APPROXIMATE SIZE, LOCATION AND TYPE OF LIMITATION.



- ① WELD 32-3 - LIMITS WELD 32-4 EXAM 0°, 45° + 60° 5, 7 + 8 SCANS.
- ② WELD 32-4 - LIMITS WELD 32-3 EXAM 0°, 45° + 60° 2, 7 + 8 SCANS.
- ③ WELD 32-1WS LIMITS WELD 32-4 EXAM, 0°, 45° + 60° 2, 7 + 8 SCANS.
- ④ 1 1/2" NOZZLE, LIMITS WELD 32-3 EXAM, 5 SCAN (60°)
- ⑤ 3" NOZZLE, LIMITS WELD 32-3 EXAM 5 SCAN (60°)
- ⑥ WELD CROWNS ON BOTH 32-3 + 32-4 LIMIT ALL SCANS.

ADS/MBB
3-2-89

WESTINGHOUSE NUCLEAR SERVICE DIVISION INSPECTION SERVICES WELD ULTRASONIC EXAMINATION INSERVICE				PLANT				UNIT				SKETCH											
				INDIAN POINT				NO.3				INT-2-1210											
				SYST/COMP.				ACCUMULATOR TANK INTSIATAT 2-32				PROCEDURE											
				EXAMINER (LEVEL II)				<i>Robert A. Casper, Bill Acton</i>				DATE											
								9-30-90															
EQUIPMENT		TRANSDUCER		STRAIGHT BEAM SCAN DIRECTION 0				AXIAL SCANS DIRECTIONS 2 & 5				CIRCUMFERENTIAL SCANS DIRECTIONS 7 & 8				CALIBRATION CHECK							
INST. S/N SONIC MK 1		S/N		C26682												INITIAL TIME 0917							
		SIZE		1.0"																			
		FREQ.		2.25 MHz																			
S/N 08079E		ANGLE		0°																			
REP. RATE		CALIBRATION REFLECTOR LOCATION		SIGNAL AMPLITUDE		SWEEP POSITION		SIGNAL AMPLITUDE		SWEEP POSITION		DISTANCE FROM SCRIBE/REF. LINE TO PEAK		50% DAC LOCATIONS		SIGNAL AMPLITUDE		SWEEP POSITION		DISTANCE FROM SCRIBE/REF. LINE TO PEAK		50% DAC LOCATIONS	
3 K		1/4 T		70%		2.0																	
REJECT		1/2 T		70%		4.0																	
OFF		3/4 T		80%		6.0																	
DAMPING		1 T				9.0																	
MIN																							
FILTER																							
HI																							
LIN. CHECK																							
SAT																							
SU CABLE																							
10' BNC TO BNC																							
COUPLANT		CAL. GAIN		18 dB												CAL. BLOCK							
SONOTRACE 40																INT-42							
BATCH # 8979																THICKNESS 2.6"							
																TEMP. 79°F							
WELD NUMBER		TEMP.		BASE METAL SCAN		SCAN DIRECTION				EXAMINATION LIMITATIONS				CROWN CONFIGURATION		RESULTS				REMARKS			
						2 5 7/8 0										NI NRI RI							
32-1		78°F								YES WELD CONFIGURATION				SLIGHTLY ROUNDED		X				EXAMINED FROM 126" CLOCKWISE TO 252" FROM O REFERENCE.			
32-3		78°F								YES WELD CONFIGURATION AND ADJACENT WELD 32-4				SLIGHTLY ROUNDED		X				EXAMINED FROM 126" CLOCKWISE TO 252" FROM O REFERENCE.			
32-4		78°F								YES WELD CONFIGURATION AND ADJACENT WELDS 32-3 + 32-1WS				ROUNDED		X				EXAMINED FROM 126" CLOCKWISE TO 252" FROM O REFERENCE.			
										SEE ATTACHED LIMITATION TO EXAMINATION SHEET FOR WELDS 32-1, 32-3 + 32-4										SURFACE THERMOMETER S/N 1979			

RSB
10/13/90
SITE IND. LEVEL III

MS/HES
10-15-90

WESTINGHOUSE NUCLEAR SERVICE DIVISION INSPECTION SERVICES WELD ULTRASONIC EXAMINATION INSERVICE				PLANT		UNIT		SKETCH							
				INDIAN POINT		NO. 3		INT-2-1210							
				SYST/COMP.		ACCUMULATOR TANK INTSEATAT2-32		PROCEDURE		INT-ISE-47, REV 1					
EXAMINER (LEVEL II)				DATE											
James R. Delbasso				9-30-90											
EQUIPMENT		TRANSDUCER		STRAIGHT BEAM SCAN DIRECTION 0				CIRCUMFERENTIAL SCANS DIRECTIONS 7 & 8				CALIBRATION CHECK			
INST. S/N		S/N		D25351				D25351				INITIAL TIME			
SONIC MARK 1		SIZE		0.5" x 1.0"				0.5" x 1.0"				0837			
05940E		FREQ.		2.25 MHZ				2.25 MHZ							
		ANGLE		45°				45°							
REP. RATE		CALIBRATION REFLECTOR LOCATION		SIGNAL AMPLITUDE		SWEEP POSITION		DISTANCE FROM SCRIBE/REF. LINE TO		SIGNAL AMPLITUDE		SWEEP POSITION		DISTANCE FROM SCRIBE/REF. LINE TO	
3K		1/4T		80%		2.0		PEAK 1 1/16" 50% DAC LOCATIONS 9/16" 1 3/16"		80%		2.0		PEAK 1 1/16" 50% DAC LOCATIONS 9/16" 1 3/16"	
REJECT															
OFF															
DAMPING															
MIN															
FILTER															
OFF															
LIN. CHECK														FINAL TIME	
SAT		1T (NOTCH)		16%		8.7				16%		8.7		1049	
SU CABLE															
12' BNC TO BNC		1 1/4T		26%		10.0				26%		10.0		CAL. BLOCK INT-42	
COUPLANT SONOPLANT 40 BATCH # 8979		CAL. GAIN		44 dB				44 dB				THICKNESS 2.6"			
												TEMP. 79° F			
WELD NUMBER	TEMP.	BASE METAL SCAN	SCAN DIRECTION				EXAMINATION LIMITATIONS		CROWN CONFIGURATION	RESULTS			REMARKS		
			2	5	7/8	0				NI	NRI	RI			
32-1	78°F		YES	YES	YES		WELD AND HEAD CONFIGURATION		SLIGHTLY ROUNDED		X		EXAMINED FROM 126" CLOCKWISE TO 252" FROM 0 REFERENCE		
32-3	78°F		YES	YES	YES		WELD CONFIGURATION, WELDS 32-4 & 32-5 LS		SLIGHTLY ROUNDED	X			EXAMINED FROM 126" CLOCKWISE TO 252" FROM 0 REFERENCE		
32-4	78°F		YES	YES	YES		WELD CONFIGURATION, WELDS 32-3 & 32-1WS, AND SUPPORT SKIRT		ROUNDED	X			EXAMINED FROM 126" CLOCKWISE TO 252" FROM 0 REFERENCE		
							SEE LIMITATION TO EXAMINATION SHEET FOR WELDS 32-1, 32-3 & 32-4						FROM 0 REFERENCE		
													SURFACE THERMOMETER		
													SN 1979		

10/13/90
 SITE NDE LEVEL III

PDS/HOB
 10-15-90

WESTINGHOUSE NUCLEAR SERVICE DIVISION INSPECTION SERVICES WELD ULTRASONIC EXAMINATION INSERVICE				PLANT		UNIT		SKETCH						
				INDIAN POINT		No. 3		INT-2-1210						
				SYST/COMP.		PROCEDURE								
				Accumulator Tank		INTSIATAT2-32		INT-IST-47 Rev.1						
				EXAMINER (LEVEL II)				DATE						
				William J. Halley		Clyde W. Kich I		9-30-90						
EQUIPMENT	TRANSDUCER	STRAIGHT BEAM SCAN DIRECTION 0	AXIAL SCANS DIRECTIONS 2 & 5				CIRCUMFERENTIAL SCANS DIRECTIONS 7 & 8			CALIBRATION CHECK				
INST. S/N	S/N		D15542				D15542			INITIAL TIME				
SONIC MARK 2	SIZE		0.5" X 1.0"				0.5" X 1.0"			IC: 0902				
11222E	FREQ.		2.25 MHz				2.25 MHz			CC: 1152				
REP. RATE	CALIBRATION REFLECTOR LOCATION	SIGNAL AMPLITUDE	SWEEP POSITION	SIGNAL AMPLITUDE	SWEEP POSITION	DISTANCE FROM SCRIBE/REF. LINE TO		SIGNAL AMPLITUDE	SWEEP POSITION	DISTANCE FROM SCRIBE/REF. LINE TO				
3K	1/4 T	80%	2.0	80%	2.0	1 3/16"	1 1/32"	1 1/2"	80%	2.0	1 3/16"	1 1/32"	1 1/2"	
REJECT						PEAK	50% DAC LOCATIONS				PEAK	50% DAC LOCATIONS		
OFF														
DAMPING														
MIN	1/2 T	45%	4.0	45%	4.0	2 3/8"	2 1/16"	2 13/16"	45%	4.0	2 3/8"	2 1/16"	2 13/16"	
FILTER														
HI	3/4 T	28%	6.0	28%	6.0	3 17/32"	3 5/32"	4 3/16"	28%	6.0	3 17/32"	3 5/32"	4 3/16"	
LIN. CHECK														
SAT	1 T	8%	8.5	8%	8.5				8%	8.5	5.55	6.95	FINAL TIME	
SU CABLE														
12' BNC-BNC	1 1/4 T	11%	10.0	11%	10.0				11%	10.0				
COUPLANT	CAL. GAIN													
SONOTRACE 40	56dB								56dB				CAL. BLOCK	
BATCH # 8979													INT-42	
													THICKNESS 2.6"	
													TEMP. 79°F	
WELD NUMBER	TEMP.	BASE METAL SCAN	SCAN DIRECTION				EXAMINATION LIMITATIONS			CROWN CONFIGURATION	RESULTS			REMARKS
			2	5	7/8	0				NI	NRI	RI		
32-1	78°F		YES	YES	YES		WELD CONFIGURATION + HEAD CONFIGURATION			SLIGHTLY ROUNDED		X	EXAMINED FROM 126"	
													CLOCKWISE TO 252"	
													FROM O REFERENCE	
32-3	78°F		YES	YES	YES		WELD CONFIGURATION, WELD 32-4, WELD 32-5LS + SUPPORT SKIRT			SLIGHTLY ROUNDED	X		EXAMINED FROM 126"	
													CLOCKWISE TO 252"	
													FROM O REFERENCE	
32-4	78°F		YES	YES	YES		WELD CONFIGURATION, WELD 32-3, WELD 32-1WS + SUPPORT SKIRT			ROUNDED	X		EXAMINED FROM 126"	
													CLOCKWISE TO 252"	
													FROM O REFERENCE	
													SURFACE THERMOMETER	
													S/N 1979	

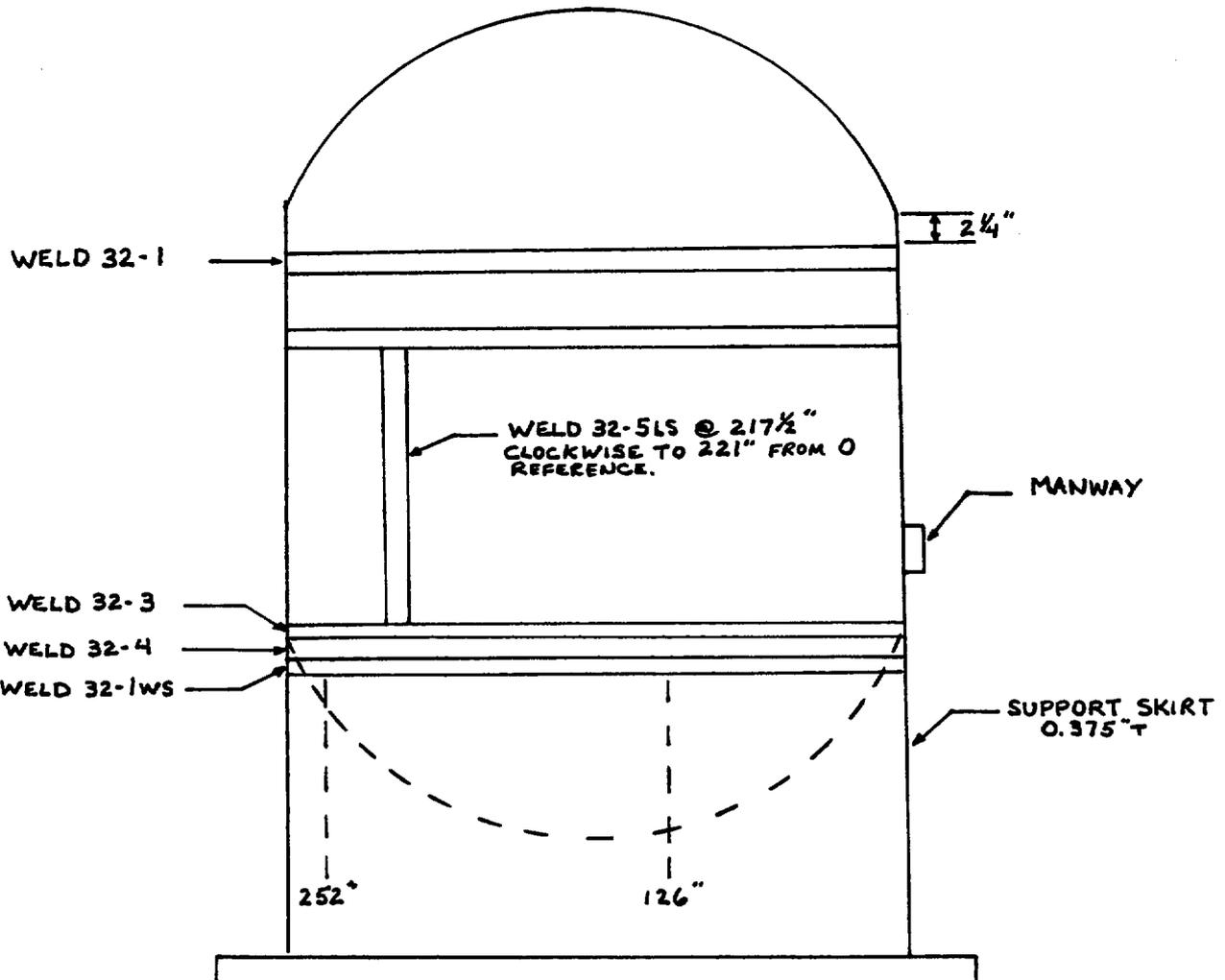
WESTINGHOUSE NUCLEAR SERVICE DIVISION
INSPECTION SERVICES

LIMITATION TO EXAMINATION

PLANT INDIAN POINT UNIT NO. 3 SKETCH INT-2-1210
 SYST/COMP ACCUMULATOR TANK INTSIATAT 2-32 PROCEDURE INT-ISI-47 REV. 1
 EXAMINER Robert L. Carnot William L. Kelly DATE 9-30-90
 LEVEL II
James R. DeLorenzo

RELATED TO: U/T P/T _____ M/T _____ V/T _____ ITEM(S): 32-1, 32-3 + 32-4

PROVIDE GENERAL INFORMATION TO DESCRIBE APPROXIMATE SIZE, LOCATION AND TYPE OF LIMITATION.



WELD 32-1 HEAD CONFIGURATION LIMITS 45°+60° 5 SCANS. WELD CONFIGURATION LIMITS 0°, 45°+60° 2, 5, 7+8 SCANS.
 WELD 32-3 WELD 32-5LS LIMITS 45°+60° 5, 7+8 SCANS. WELD CONFIGURATION LIMITS 0°, 45°+60° 2, 5, 7+8 SCANS. WELD 32-4 LIMITS 0°, 45°+60° 2, 7+8 SCANS. SUPPORT SKIRT LIMITS 60° 2 SCAN.
 WELD 32-4 WELD CONFIGURATION LIMITS 0°, 45°+60° 2, 5, 7+8 SCANS. WELD 32-3 LIMITS 0°, 45°+60° 5, 7+8 SCANS. WELD 32-1WS LIMITS 0°, 45°+60° 2, 7+8 SCANS. SUPPORT SKIRT LIMITS 45°+60° 2 SCANS.

SITE NDF LEVEL III
 02/12/90
 12/13/90

ARSHKSB
 15-90

WESTINGHOUSE NUCLEAR SERVICE DIVISION INSPECTION SERVICES				PLANT INDIAN POINT				UNIT NO. 3				SKETCH INT-2-1210, REV. 1													
WELD ULTRASONIC EXAMINATION INSERVICE				SYST. / COMP. ACCUMULATOR TANK INTSIATAT2-32				PROCEDURE INT-ISI-47, REV 1																	
				EXAMINER (LEVEL II) <i>James R. Dellano</i>				DATE 9-23-99																	
				EQUIPMENT				TRANSducer				STRAIGHT BEAM SCAN DIRECTION 0				AXIAL SCANS DIRECTIONS 2 & 5				CIRCUMFERENTIAL SCANS DIRECTIONS 7 & 8				CALIBRATION CHECK	
INST. S / N STAVELEY 136 W16498				S / N C26672				H30049				H30049				INITIAL TIME 0° - 1110 45° - 1150									
REP. RATE 4K				SIZE 1.0"				0.5" X 1.0"				0.5" X 1.0"													
REJECT OFF				ANGLE 0°				45°				45°													
DAMPING 500 OHMS				CALIBRATION REFLECTOR LOCATION		SIGNAL AMPLITUDE		SWEEP POSITION		SIGNAL AMPLITUDE		SWEEP POSITION		DISTANCE FROM SCRIBE / REF. LINE TO 50% DAC LOCATIONS		SIGNAL AMPLITUDE		SWEEP POSITION		DISTANCE FROM SCRIBE / REF. LINE TO 50% DAC LOCATIONS					
FILTER FULLWAY				1/4 T		80%		2.0		80%		2.0		0.68"		80%		2.0		0.68"					
LIN. CHECK SAT				1/2 T		80%		4.0		72%		4.0		1.4"		72%		4.0		1.4"					
S. U. CABLE 12' BNC to BNC				1 T NOTCH				15%		8.7						15%		8.7							
COUPLANT SONOTRACE 40 95343D				1-1/4 T				10%		10.0						10%		10.0				FINAL TIME 0° - 1530 45° - 1535			
				CAL. GAIN		46.6 dB		37 dB						37 dB						CAL. BLOCK INT-42					
				THICKNESS		2.6"														TEMPERATU 76°					
WELD NUMBER		TEMP.		BASE METAL SCAN		SCAN DIRECTION 2 5 7/8 0				EXAMINATION LIMITATIONS				CROWN CONFIGURATION		RESULTS NI NRI RI			REMARKS						
32-3		80°		NO		YES YES YES YES				SEE LIMITATION DATA SHEET				ROUNDED		X			SEE NOTE BELOW						
32-4		80°		NO		YES YES YES YES				SEE LIMITATION DATA SHEET				ROUNDED		X			SEE NOTE BELOW						
																			NOTE: EXAMINED FROM 252" THRU 376.8" CLOCKWISE FROM 0 REFERENCE SURFACE THERMOMETER SN 38081						
NEW YORK POWER AUTHORITY LEVEL III REVIEW / DATE <i>PE Deed</i> 10/4/99										ANI REVIEW / DATE <i>Shelby Evans</i> 10/6/99															

WESTINGHOUSE NUCLEAR SERVICE DIVISION
INSPECTION SERVICES

LIMITATION TO EXAMINATION

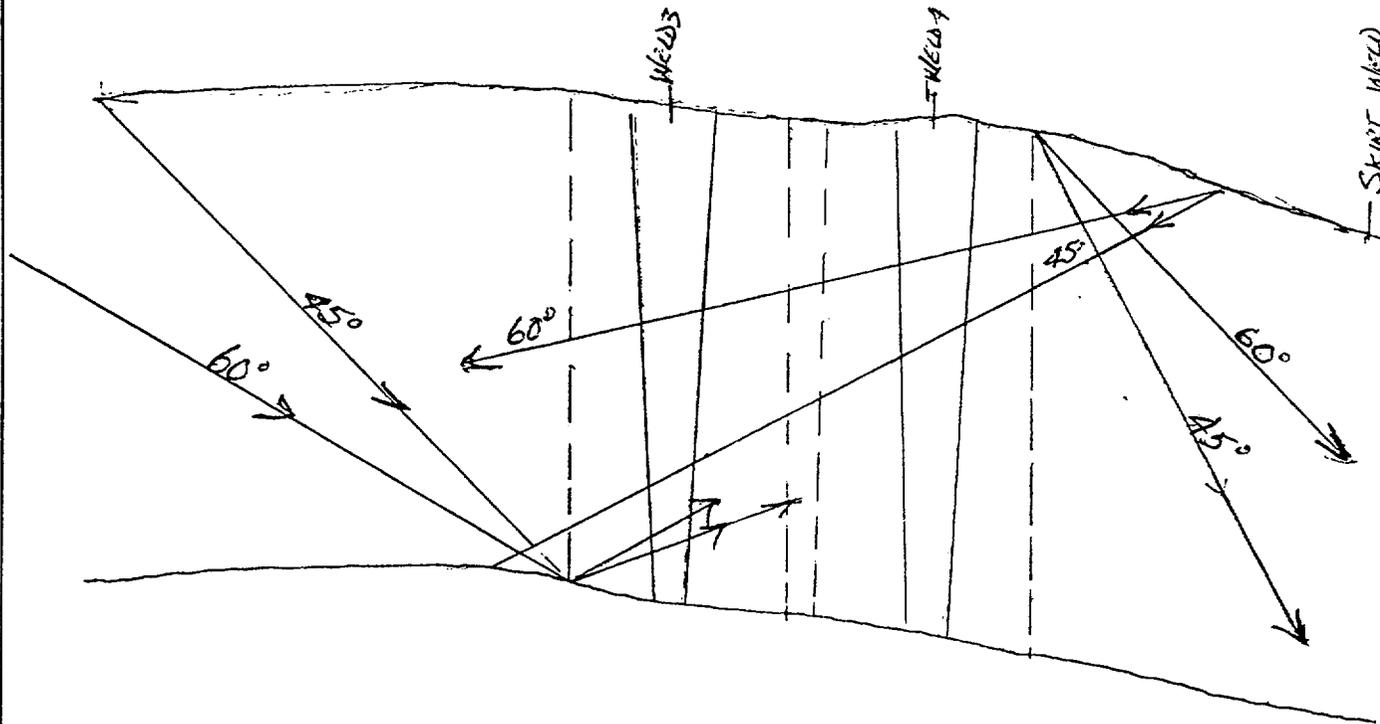
PLANT INDIAN POINT UNIT NO. 3 SKETCH INT-2-1210 REV. 1

SYST./COMP. ACCUMULATOR TANK INTSIATAT2-32 PROCEDURE INT-ISI-47, REV. 1

EXAMINER *James R. DeLorenzo* DATE 9-23-99
LEVEL II

RELATED TO: U/T X P/T _____ M/T _____ V/T _____ ITEM(S) 32-3 AND 32-4

PROVIDE GENERAL INFORMATION TO DESCRIBE APPROXIMATE SIZE, LOCATION AND TYPE OF LIMITATION.



WELD 3 94% EXAM VOLUME COVERED WITH 45° AND 88% WITH 60°

WELD 4 89% EXAM VOLUME COVERED WITH 45° AND 66% WITH 60°

NEW YORK POWER AUTHORITY LEVEL III REVIEW / DATE

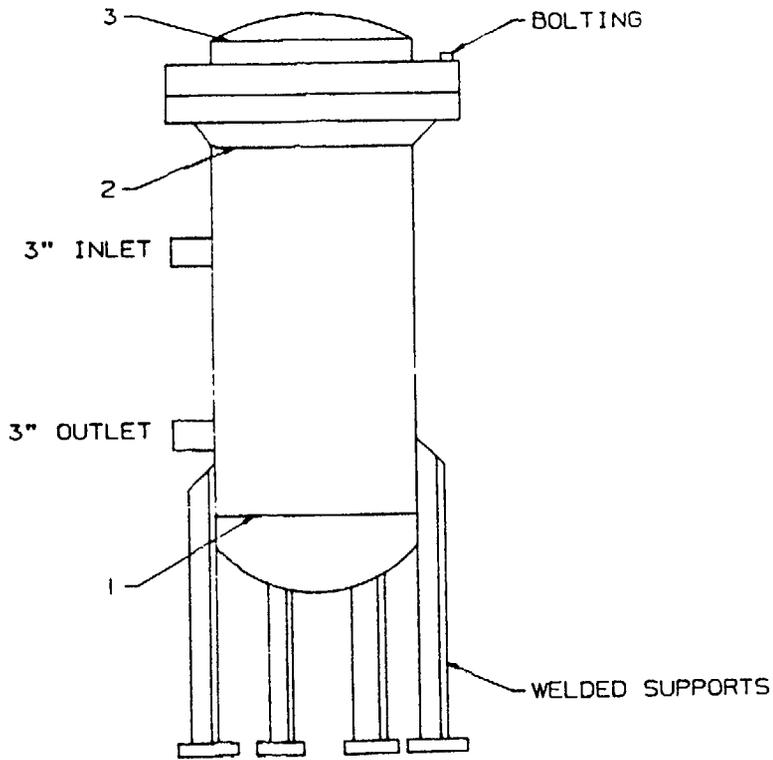
P. E. Deedy 10/14/99

ANII REVIEW / DATE

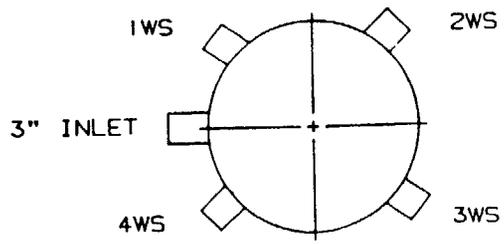
Philip Evans 10/04/99

365

SEAL WATER RETURN FILTER
CSFLSWI-31



WELDS 1, 2 & 3: 0.188" T TP304 STAINLESS STEEL
 DIAMETER 16.0"; CIRCUMFERENCE: 50.24"
 NOZZLE IN VESSEL WELDS: 3" DIAMETER
 INTEGRALLY WELDED ATTACHMENTS: 4-0.4375" T
 COMPONENT SUPPORTS: 4 WELDED ATTACHMENTS
 BOLTING: 8-0.750" DIAMETER
 0 REFERENCE: CENTERLINE OF 3" INLET NOZZLE



NEW YORK POWER AUTHORITY INDIAN POINT UNIT NO.3	
SEAL WATER RETURN FILTER CSFLSWI-31	
INT-2-1320	REV. 2

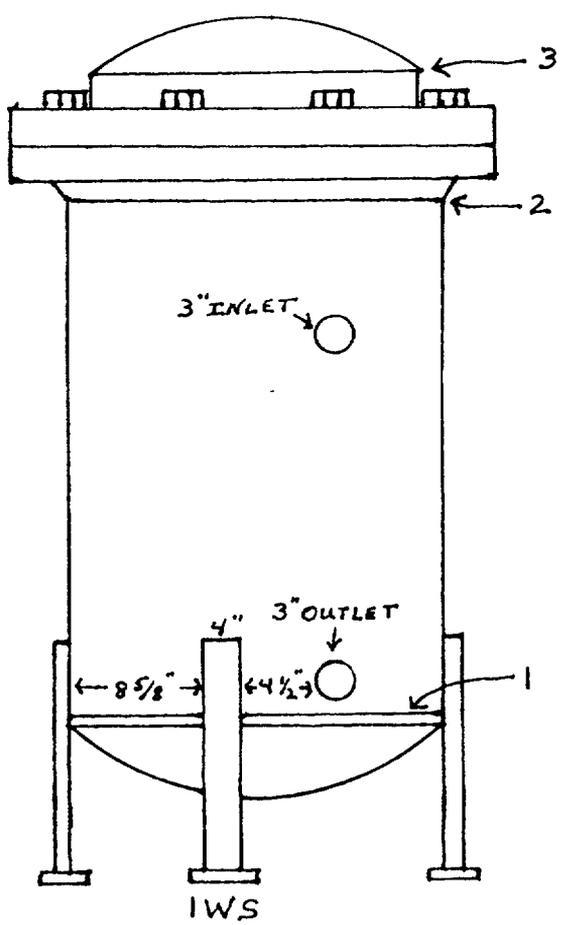
WESTINGHOUSE NUCLEAR SERVICE DIVISION
INSPECTION SERVICES

LIMITATION TO EXAMINATION

PLANT INDIAN POINT UNIT No. 3 SKETCH INT-2-1320
SYST/COMP SEAL WATER RETURN FILTER CSELSW/3 PROCEDURE INT-IST-11 REV. 0
EXAMINER Kenneth H. Cline DATE 3-23-89
LEVEL II

RELATED TO: U/T _____ P/T M/T _____ V/T _____ ITEM(S): 1

PROVIDE GENERAL INFORMATION TO DESCRIBE APPROXIMATE SIZE, LOCATION AND TYPE OF LIMITATION.



4" SUPPORT LIMITS PT EXAMINATION ON WELD 1

ADS/H5B
5/24/89

369

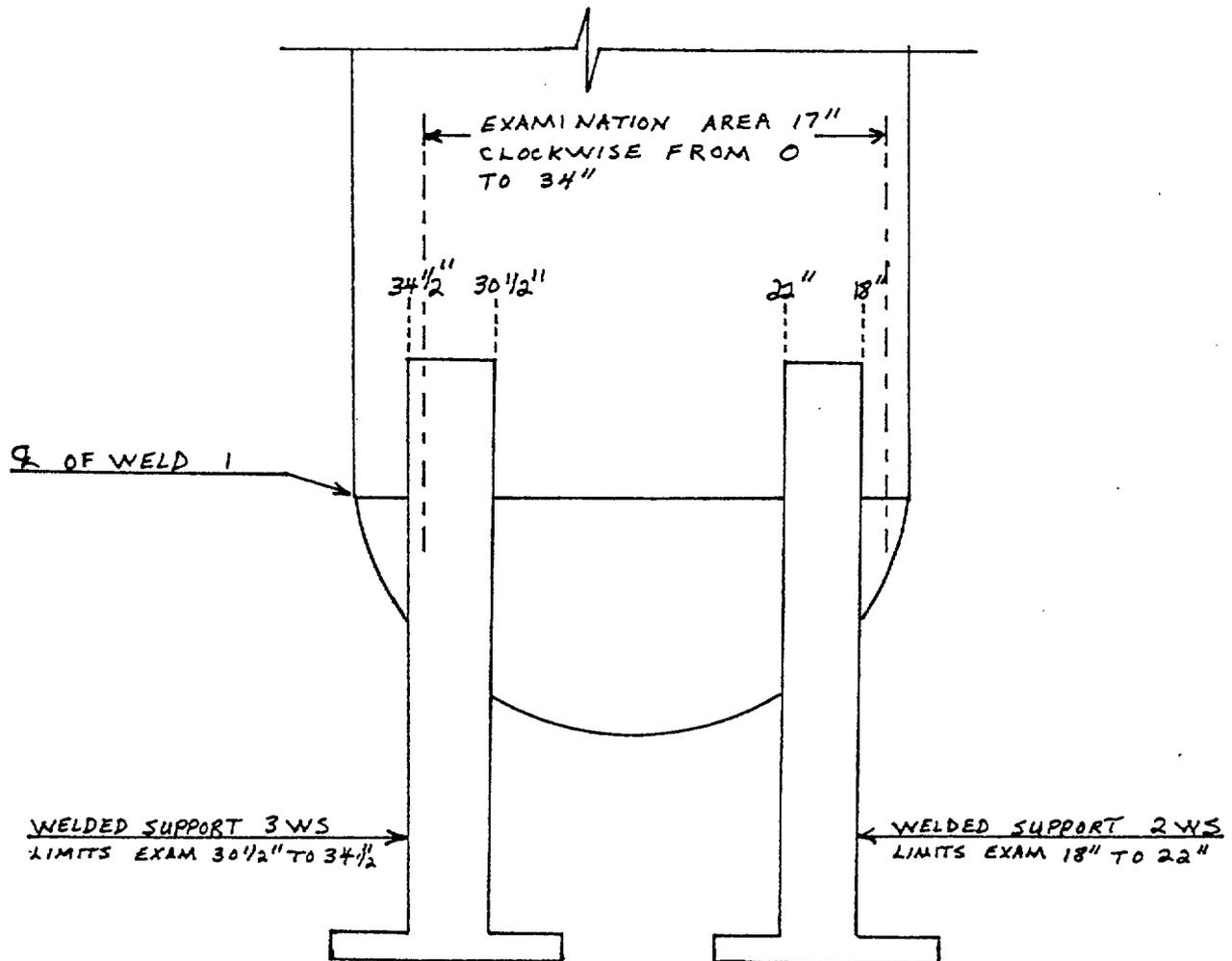
WESTINGHOUSE NUCLEAR SERVICE DIVISION
INSPECTION SERVICES

LIMITATION TO EXAMINATION

PLANT INDIAN POINT UNIT NO. 3 SKETCH INT-2-1320
SYST/COMP SEAL WATER RETURN FILTER CSFSLSW1-31 PROCEDURE INT-ISI-11 REV 1
EXAMINER [Signature] DATE 11-4-90
LEVEL II

RELATED TO: U/T _____ P/T M/T _____ V/T _____ ITEM(S): 1

PROVIDE GENERAL INFORMATION TO DESCRIBE APPROXIMATE SIZE, LOCATION AND TYPE OF LIMITATION.



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[Signature] 11/13/90

MS/1/55
11-13-90

Code Category C-B:

Pressure Retaining Nozzle Welds in Vessels

Item No. C2.20

Steam Generator

Item No. C2.21

Nozzle-to-Shell (or Head) Weld

- Relief Request RR 2-27, Rev. 0 previously granted for weld 31-9 (Ref. TAC No. M82269, pages 21-23, dated December 21, 1994)

Code Category C-B:

Pressure Retaining Nozzle Welds in Vessels

Item No. C2.20

Item No. C2.22

Steam Generator Nozzle Inside Radius Section

- Relief Request RR 2-16, rev. 0 for Item C2.22 previously granted via SER (TAC No. 72247, pages 35-37, dated November 7, 1991).