

December 1, 1995

DISTRIBUTION

Mr. J. W. Hampton
Vice President, Oconee Site
Duke Power Company
P. O. Box 1439
Seneca, SC 29679

Docket File
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OGC 0-15 B18

G.Hill(4) T-5 C3
C.Grimes 0-11 F23
ACRS T-2 E26

SUBJECT: ISSUANCE OF AMENDMENTS - OCONEE NUCLEAR STATION, UNITS 1, 2,
AND 3 (TAC NOS. M91411, M91412, AND M91413)

Dear Mr. Hampton:

The Nuclear Regulatory Commission has issued the enclosed Amendment Nos. 211, 211, and 208 to Facility Operating Licenses DPR-38, DPR-47, and DPR-55, respectively, for the Oconee Nuclear Station, Units 1, 2, and 3. The amendments consist of changes to the Technical Specifications (TS) in response to Duke Power Company's application dated January 12, 1995, as supplemented by letter dated June 29, 1995. The amendments revise and clarify portions of TS Section 6.0, "Administrative Controls."

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Leonard A. Wiens, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270 and 50-287

Enclosures:

1. Amendment No. 211 to DPR-38
2. Amendment No. 211 to DPR-47
3. Amendment No. 208 to DPR-55
4. Safety Evaluation

cc w/encl: See next page

DOCUMENT NAME:

OFFICE	DRPE/PD22/LA	DRPE/PD22/PM	OGC	DRPE/PD22/D
NAME	L. BERRY	L. WIENS		H. BERKOW
DATE	11/16/95	11/16/95	11/27/95	12/01/95
COPY	YES NO	YES NO	YES NO	YES NO

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Mr. J. W. Hampton
Duke Power Company

Oconee Nuclear Station

cc:

A. V. Carr, Esquire
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242-0001

Mr. Ed Burchfield
Compliance
Duke Power Company
Oconee Nuclear Site
P. O. Box 1439
Seneca, South Carolina 29679

J. Michael McGarry, III, Esquire
Winston and Strawn
1400 L Street, NW.
Washington, DC 20005

Ms. Karen E. Long
Assistant Attorney General
North Carolina Department of
Justice
P. O. Box 629
Raleigh, North Carolina 27602

Mr. Robert B. Borsum
B&W Nuclear Technologies
Suite 525
1700 Rockville Pike
Rockville, Maryland 20852-1631

Mr. G. A. Copp
Licensing - EC050
Duke Power Company
526 South Church Street
Charlotte, North Carolina 28242-0001

Manager, LIS
NUS Corporation
2650 McCormick Drive, 3rd Floor
Clearwater, Florida 34619-1035

Dayne H. Brown, Director
Division of Radiation Protection
North Carolina Department of
Environment, Health and
Natural Resources
P. O. Box 27687
Raleigh, North Carolina 27611-7687

Senior Resident Inspector
U. S. Nuclear Regulatory Commission
Route 2, Box 610
Seneca, South Carolina 29678

Regional Administrator, Region II
U. S. Nuclear Regulatory Commission
101 Marietta Street, NW. Suite 2900
Atlanta, Georgia 30323

Max Batavia, Chief
Bureau of Radiological Health
South Carolina Department of Health
and Environmental Control
2600 Bull Street
Columbia, South Carolina 29201

County Supervisor of Oconee County
Walhalla, South Carolina 29621



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 211
License No. DPR-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 1 (the facility) Facility Operating License No. DPR-38 filed by the Duke Power Company (the licensee) dated January 12, 1995, as supplemented by letter dated June 29, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 3.B of Facility Operating License No. DPR-38 is hereby amended to read as follows:

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Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 211, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance..

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: December 1, 1995



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

DOCKET NO. 50-270

OCONEE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 211
License No. DPR-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 2 (the facility) Facility Operating License No. DPR-47 filed by the Duke Power Company (the licensee) dated January 12, 1995, as supplemented by letter dated June 29, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 3.B of Facility Operating License No. DPR-47 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 211, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: December 1, 1995



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

DOCKET NO. 50-287

OCONEE NUCLEAR STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 208
License No. DPR-55

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 3 (the facility) Facility Operating License No. DPR-55 filed by the Duke Power Company (the licensee) dated January 12, 1995, as supplemented by letter dated June 29, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 3.B of Facility Operating License No. DPR-55 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 208, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: December 1, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 211

FACILITY OPERATING LICENSE NO. DPR-38

DOCKET NO. 50-269

AND

TO LICENSE AMENDMENT NO. 211

FACILITY OPERATING LICENSE NO. DPR-47

DOCKET NO. 50-270

AND

TO LICENSE AMENDMENT NO. 208

FACILITY OPERATING LICENSE NO. DPR-55

DOCKET NO. 50-287

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove Pages

Insert Pages

v

v

6.1-2
6.1-3
6.1-3a(1)
6.1-3a(2)
6.1-3b
6.1-4

6.1-2
6.1-3
6.1-4
6.1-4a
6.1-4b

6.4-1
6.4-2
6.4-2a
6.4-2b

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6.4-5
6.4-6

6.5-1
6.5-2

6.5-1
6.5-2

<u>Section</u>	<u>Page</u>	
4.9	EMERGENCY FEEDWATER PUMP AND VALVE PERIODIC TESTING	4.9-1
4.10	REACTIVITY ANOMALIES	4.10-1
4.11	(Not Used)	4.11-1
4.12	CONTROL ROOM PRESSURIZATION AND FILTERING SYSTEM	4.12-1
	(INTENTIONALLY BLANK)	4.13-1
4.14	REACTOR BUILDING PURGE FILTERS AND SPENT FUEL VENTILATION SYSTEM	4.14-1
4.15	(Not Used)	4.15-1
4.16	RADIOACTIVE MATERIALS SOURCES	4.16-1
4.17	STEAM GENERATOR TUBING SURVEILLANCE	4.17-1
4.18	SNUBBERS	4.18-1
4.19	(NOT USED)	
4.20	STANDBY SHUTDOWN FACILITY	4.20-1
4.21	(Not Used)	4.21-1
5	<u>DESIGN FEATURES</u>	5.1-1
5.1	SITE	5.1-1
5.2	CONTAINMENT	5.2-1
5.3	REACTOR	5.3-1
5.4	NEW AND SPENT FUEL STORAGE FACILITIES	5.4-1
6	<u>ADMINISTRATIVE CONTROLS</u>	6.1-1
6.1	ORGANIZATION, REVIEW, AND AUDIT	6.1-1
6.1.1	<u>Organization</u>	6.1-1
6.1.2	<u>Technical Review and Control</u>	6.1-2
6.1.3	<u>Nuclear Safety Review Board</u>	6.1-4
6.2	ACTION TO BE TAKEN IN THE EVENT OF A REPORTABLE OCCURRENCE	6.2-1
6.3	ACTION TO BE TAKEN IN THE EVENT A SAFETY LIMIT IS EXCEEDED	6.3-1
6.4	STATION OPERATING PROCEDURES	6.4-1

OCONEE UNITS 1,2,3

	Amendment No. 211	(Unit 1)
v	Amendment No. 211	(Unit 2)
	Amendment No. 208	(Unit 3)

6.1.2 TECHNICAL REVIEW AND CONTROL

6.1.2.1 Activities

Programs shall be established for the preparation, review, approval, and retention of documents required by the activities described in Specifications 6.1.2.1a through 6.1.2.1k. Approvals shall be by the head of the appropriate site organization, the head of the appropriate station organization, the head of the appropriate site engineering organization, the head of the environmental organization, or an alternate as specified in other applicable regulatory documents or administrative controls.

- a. Each procedure and program required by Specification 6.4 and other procedures which affect nuclear safety, and changes thereto, shall be prepared by a knowledgeable individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/organization other than the individual/organization which prepared the procedure, or changes thereto. Procedures, or changes thereto, shall be approved in accordance with Specifications 6.4.2 and 6.4.3.
- b. Proposed modifications to unit nuclear safety-related structures, systems and components shall be designed by a knowledgeable individual/organization. Each modification shall be reviewed by an individual/organization other than the individual/organization which designed the modification.
- c. Individuals responsible for reviews performed in accordance with Specifications 6.1.2.1.a and 6.1.2.1.b shall be members of the supervisory staff assigned to the site, and previously designated by the Site Vice President to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated site review personnel.
- d. Proposed changes to the Technical Specifications shall be prepared by a knowledgeable individual/organization. The preparation of each proposed Technical Specification change shall be reviewed by an individual/organization other than the individual/organization which prepared the proposed change.

6.1-2

OCONEE UNITS 1,2,3

Amendment No. 211 (Unit 1)
Amendment No. 211 (Unit 2)
Amendment No. 208 (Unit 3)

- e. Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be prepared and approved in a manner identical to that of Specification 6.1.2.1.a. These proposed tests and experiments shall be reviewed by an individual/organization other than the individual/organization which prepared the proposed tests and experiments.
- f. Incidents reportable pursuant to Technical Specification 6.6.2.1 and all violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. Such reports shall be reviewed by a knowledgeable individual/organization other than the individual/organization which prepared the report.
- g. Special reviews and investigations, and the preparation and submittal of reports thereon, shall be performed by a knowledgeable individual/organization.
- h. A knowledgeable individual/organization shall review every unplanned onsite release of radioactive material to the environs, and prepare reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence.
- i. A knowledgeable individual/organization shall review changes to the Process Control Program, Offsite Dose Calculation Manual (ODCM), and Radwaste Treatment Systems.
- j. A knowledgeable individual/organization shall review the Fire Protection Program and implementing procedures.
- k. Reports documenting each of the activities performed under Specifications 6.1.2.1.a through 6.1.2.1.j shall be maintained.

6.1-3

OCONEE UNITS 1,2,3

Amendment No. 211 (Unit 1)
Amendment No. 211 (Unit 2)
Amendment No. 208 (Unit 3)

6.1.3 Nuclear Safety Review Board (NSRB)

6.1.3.1 Function

The NSRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,
- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering, and
- h. Administrative control and quality assurance practices.

6.1.3.2 Organization

- a. The Director, members and alternate members of the NSRB shall be appointed in writing by the Senior Vice President, Nuclear Generation and shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of 5 years technical experience, of which a minimum of 3 years shall be in one or more areas given in Specification 6.1.3.1.
- b. In special cases, candidates for appointment without an academic degree in engineering or physical science may be qualified with a minimum of ten years experience in one of the areas in Specification 6.1.3.1. No more than two alternates shall participate as voting members in NSRB activities at any one time.
- c. The NSRB shall be composed of at least five members, including the Director. Members of the NSRB may be from the Nuclear Generation Department, from other departments within the Company, or from external to the Company. A maximum of one member of the NSRB may be from the Oconee Nuclear Site staff.

6.1-4

OCONEE UNITS 1,2,3

Amendment No. 211 (Unit 1)
Amendment No. 211 (Unit 2)
amendment No. 208 (Unit 3)

- d. Consultants shall be utilized as determined by the NSRB Director to provide expert advice to the NSRB.
- e. Staff assistance may be provided to the NSRB in order to promote the proper, timely, and expeditious performance of its functions.
- f. The NSRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least twice per year thereafter.
- g. The quorum of the NSRB necessary for the performance of the NSRB review and audit functions of these Technical Specifications shall consist of the Director, or designated alternate, and at least four other NSRB members including alternates. No more than a minority of the quorum shall have line responsibility for operation of Oconee Nuclear Station.

6.1.3.3 Review

The NSRB shall review:

- a. The safety evaluations for: (1) changes to procedures, equipment, or systems, and (2) tests or experiments completed under the provision of Section 50.59, 10 CFR to verify that such actions did not constitute an unreviewed safety question;
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- d. Violations of Codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- e. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety;

6.1-4a

OCONEE UNITS 1,2,3

Amendment No. 211 (Unit 1)
 Amendment No. 211 (Unit 2)
 amendment No. 208 (Unit 3)

- f. ALL REPORTABLE EVENTS;
- g. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems or components that could affect nuclear safety;
- h. Quality Assurance Program audits relating to station operations and actions taken in response to these audits; and
- i. Reports of activities performed under the provisions of Specifications 6.1.2.1a through 6.1.2.1j.

6.1-4b

OCONEE UNITS 1,2,3

Amendment No. 211 (Unit 1)
Amendment No. 211 (Unit 2)
Amendment No. 208 (Unit 3)

6.4 STATION OPERATING PROCEDURES

Specification

- 6.4.1 The station shall be operated and maintained in accordance with approved procedures. Written procedures with appropriate check-off lists and instructions shall be provided for the following conditions:
- a. Normal startup, operation, and shutdown of the complete facility and of all systems and components involving nuclear safety of the facility.
 - b. Refueling operations.
 - c. Actions taken to correct specific and foreseen potential malfunctions of systems or components involving nuclear safety and radiation levels, including responses to alarms, suspected primary system leaks and abnormal reactivity changes.
 - d. Emergency procedures involving potential or actual release of radioactivity.
 - e. Preventive or corrective maintenance which could affect nuclear safety or radiation exposure to personnel.
 - f. Station survey following an earthquake.
 - g. Personnel radiation protection procedures.
 - h. Operation of radioactive waste management systems.
 - i. Control of Ph in recirculated coolant after loss-of-coolant accident. Procedure shall state that pH will be measured and the addition of appropriate caustic to coolant will commence within 30 minutes after switchover to recirculation mode of core cooling to adjust the pH to a range of 7.0 to 8.0 within 24 hours.
 - j. Nuclear safety-related periodic test procedures.

6.4-1

OCONEE UNITS 1,2,3

Amendment No. 211 (Unit 1)
Amendment No. 211 (Unit 2)
Amendment No. 208 (Unit 3)

- k. Long-term emergency core cooling systems. Procedures shall include provision for remote or local operation of system components necessary to establish high and low pressure injection within 15 minutes after a line break.
- l. Fire Protection Program implementation.
- m. Offsite Dose Calculation Manual implementation.
- n. Process Control Program implementation.
- o. Technical Review and Control Program implementation.
- p. Plant Operations Review Committee implementation.

6.4.2 Each procedure of specification 6.4.1 above, and changes thereto, shall be reviewed and approved by an appropriate division manager, superintendent/manager, or one of their designated direct reports prior to implementation and shall be reviewed periodically as set forth in administrative procedures. For procedures which implement offsite environmental, technical, and laboratory activities, the above review and approval may be performed by the General Manager, Environmental Services or designee.

6.4.3 Temporary changes to procedures of Specification 6.4.1 above may be made provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operators license on the affected unit; and
- c. The change is approved by an appropriate division manager, superintendent/manager, or one of their designated direct reports within 14 days of implementation.

6.4-2

OCONEE UNITS 1,2,3

Amendment No. 211 (Unit 1)
 Amendment No. 211 (Unit 2)
 Amendment No. 208 (Unit 3)

6.4.4 The following programs shall be established, implemented, and maintained:

- a. A respiratory protective program approved by the Commission shall be in force.
- b. Administrative procedures shall be developed and implemented to limit the working hours of station staff who perform safety-related functions, e.g., senior reactor operators, reactor operators, nuclear equipment operators, and certain maintenance personnel.

Any deviations from the above procedures shall be authorized by the Station Manager (or designee) in accordance with established procedures and with documentation of the basis for granting the deviation. Individual overtime shall be periodically reviewed to assure that excessive hours have not been worked. Routine deviation from the above guidelines is not authorized.

- c. The station shall have a program that ensures the capability to obtain and analyze reactor coolant and containment atmosphere samples under accident conditions which includes training of personnel, procedures for sampling and analysis, and provisions for testing and required maintenance of sampling and analysis equipment.
- d. The station shall have a program that ensures the capability to collect and analyze or measure representative samples of radioactive iodines and particulates in plant gaseous effluents during and following an accident which includes training of personnel, procedures for sampling and analysis, and provisions for testing and required maintenance of sampling and analysis equipment.

6.4-3

OCONEE UNITS 1,2,3

Amendment No. 211 (Unit 1)
Amendment No. 211 (Unit 2)
Amendment No. 208 (Unit 3)

e. The station shall have a program conforming with 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to MEMBERS OF THE PUBLIC from radioactive effluents as low as reasonably achievable. The program (1) shall be contained in FSAR Chapter 16, (2) shall be implemented by operating procedures, and (3) shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

1. Limitations on the operability of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and set-point determination in accordance with the methodology in the ODCM,
2. Limitations on the concentrations of radioactive material released in liquid effluents to UNRESTRICTED AREAS conforming to 10 CFR Part 20.1001-20.2401, Appendix B, Table 2, Column 2,
3. Monitoring, sampling, and analysis of radioactive liquid and gaseous effluents in accordance with 10 CFR 20.1302 and with the methodology and parameters in the ODCM.
4. Limitations on the annual and quarterly doses or dose commitment to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released from each unit to UNRESTRICTED AREAS conforming to Appendix I to 10 CFR Part 50,
5. Determination of cumulative and projected dose contributions from radioactive effluents for the current calendar quarter and current calendar year in accordance with the methodology and parameters in the ODCM at least every 31 days.

6.4-4

OCONEE UNITS 1,2,3

Amendment No. 211 (Unit 1)
Amendment No. 211 (Unit 2)
Amendment No. 208 (Unit 3)

6. Limitations on the operability and use of the liquid and gaseous effluent treatment systems to ensure that the appropriate portions of these systems are used to reduce releases of radioactivity when the projected doses in a 31-day period would exceed 2 percent of the guidelines for the annual dose or dose commitment conforming to Appendix I to 10 CFR Part 50 as clarified by FSAR Chapter 16.
7. Limitations on the dose rate resulting from radioactive material released in gaseous effluents from the site to areas at or beyond the SITE BOUNDARY shall be limited to the following:
 - a. For noble gases: Less than or equal to a dose rate of 500 mrems/yr to the total body and less than or equal to a dose rate of 3000 mrems/yr to the skin, and
 - b. For Iodine-131, for Iodine-133, for tritium, and for all radionuclides in particulate form with half-lives greater than 8 days; Less than or equal to a dose rate of 1500 mrems/yr to any organ.
8. Limitations on the annual and quarterly air doses resulting from noble gases released in gaseous effluents from each unit to areas beyond the SITE BOUNDARY conforming to Appendix I to 10 CFR Part 50,
9. Limitations on the annual and quarterly doses to a MEMBER OF THE PUBLIC from Iodine-131, Iodine-133, tritium, and all radionuclides in particulate form with half-lives greater than 8 days in gaseous effluents released from each unit to areas beyond the SITE BOUNDARY conforming to Appendix I to 10 CFR Part 50; and,
10. Limitations on the annual dose or dose commitment to any MEMBER OF THE PUBLIC due to releases of radioactivity and to radiation from uranium fuel cycle sources conforming to 40 CFR Part 190.

6.4-5

OCONEE UNITS 1,2,3

Amendment No. 211 (Unit 1)
Amendment No. 211 (Unit 2)
Amendment No. 208 (Unit 3)

f. The station shall have a program to monitor the radiation and radionuclides in the environs of the plant. The program shall provide (1) representative measurements of radioactivity in the highest potential exposure pathways, and (2) verification of the accuracy of the effluent monitoring program and modeling of environmental exposure pathways. The program shall (1) be contained in FSAR Chapter 16, (2) conform to the guidance of Appendix I to 10 CFR Part 50, and (3) include the following:

1. Monitoring, sampling, analysis, and reporting of radiation and radionuclides in the environment in accordance with the methodology and parameters in the ODCM;
2. A Land Use Census to ensure that changes in the use of areas at and beyond the SITE BOUNDARY are identified and that modifications to the monitoring program are made if required by the results of this census; and,
3. Participation in an Interlaboratory Comparison Program to ensure that independent checks on the precision and accuracy of the measurements of radioactive materials in environmental sample matrices are performed as part of the quality assurance program for environmental monitoring.

6.4-6

OCONEE UNITS 1,2,3

Amendment No. 211 (Unit 1)
Amendment No. 211 (Unit 2)
Amendment No. 208 (Unit 3)

6.5 STATION OPERATING RECORDS

Specification

- 6.5.1 The following records shall be prepared and permanently retained in a manner convenient for review:
- a. Records of modifications to the station as described in the FSAR.
 - b. Special nuclear material physical inventory records.
 - c. Special nuclear material isotopic inventory records.
 - d. Radiation monitoring records, including records of radiation and contamination surveys.
 - e. Records of off-site environmental surveys.
 - f. Personnel radiation exposure records as required by 10CFR20.
 - g. Records of radioactive releases and waste disposal.
 - h. Records of reactor coolant system in-service inspections.
 - i. Preoperational testing records.
 - j. Records of special reactor tests or experiments.
 - k. Records of changes safety-related operating procedures.
 - l. Records for Environmental Qualification which are covered under the provisions of paragraph 6.7.
 - m. Records of the seal service lives of hydraulic snubbers.
 - n. Records of reviews performed for changes made to the ODCM and Process Control Program.
 - o. Records of meetings of the NSRB and reports required by Specification 6.1.2.1k.

6.5-1

OCONEE UNITS 1,2,3

Amendment No. 211 (Unit 1)
Amendment No. 211 (Unit 2)
Amendment No. 208 (Unit 3)

6.5.2

The following records shall be prepared and retained for a minimum of six (6) years in a manner convenient for review:

- a. Switchboard Record.
- b. Reactor Operations Logbook.
- c. Shift Supervisor Logbook.
- d. Maintenance histories for station safety-related structures, systems and components.
- e. Record of safety-related inspections, other than reactor coolant system in-service inspections.
- f. Records of reportable events.
- g. Periodic testing records and records of other periodic checks, calibrations, etc. performed in accordance with surveillance requirements for safety-related parameters, structures, systems and components.
- h. By-product material inventory records.
- i. Training records.
- j. Test results, in units of microcuries, for leak tests performed pursuant to Specification 4.16.
- k. Radioactive liquid effluent, gaseous effluent, and gaseous process monitoring instrumentation alarm/trip setpoints.



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 211 TO FACILITY OPERATING LICENSE DPR-38

AMENDMENT NO. 211 TO FACILITY OPERATING LICENSE DPR-47

AND AMENDMENT NO. 208 TO FACILITY OPERATING LICENSE DPR-55

DUKE POWER COMPANY

OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3

DOCKET NOS. 50-269, 50-270, AND 50-287

1.0 INTRODUCTION

By letter dated January 12, 1995, as supplemented by letter dated June 29, 1995, Duke Power Company (the licensee), submitted a request for changes to the Oconee Nuclear Station, Units 1, 2, and 3, Technical Specifications (TS). The requested changes would revise and clarify portions of TS Section 6.0, "Administrative Controls." The June 29, 1995, letter provided clarifying information that did not change the scope of the January 12, 1995, application and initial proposed no significant hazards consideration determination.

2.0 EVALUATION

Some of the licensee's requested changes would relocate the requirements for the review functions from the Administrative Controls section of the TS to the respective licensee-controlled documents.

Section 182a of the Atomic Energy Act (the "Act") requires applicants for nuclear power plant operating licenses to include TS as part of the license. The Commission's regulatory requirements related to the content of TS are set forth in 10 CFR 50.36. That regulation requires that the TS include items in five specific categories, including (1) safety limits, limiting safety system settings and limiting control settings, (2) limiting conditions for operation, (3) surveillance requirements, (4) design features, and (5) administrative controls. However, the regulation does not specify the particular requirements to be included in a plant's TS.

The Commission has provided guidance for the contents of TS in its "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" ("Final Policy Statement"), 58 FR 39132 (July 22, 1993), in which the Commission indicated that compliance with the Final Policy Statement satisfies §182a of the Act. In particular, the Commission indicated that certain items could be relocated from the TS to licensee-controlled documents, consistent with the standard enunciated in *Portland General Electric Co.*

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(Trojan Nuclear Plant), ALAB-531, 9 NRC 263, 273 (1979). In that case, the Atomic Safety and Licensing Appeal Board indicated that "technical specifications are to be reserved for those matters as to which the imposition of rigid conditions or limitations upon reactor operation is deemed necessary to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety."

Consistent with this approach, the Final Policy Statement identified four criteria to be used in determining whether particular limiting conditions for operation are required to be included in the TS, as follows: (1) installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary; (2) a process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (4) a structure, system, or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety. The Commission recently adopted amendments to 10 CFR 50.36, pursuant to which the rule was revised to codify and incorporate these criteria. See Final Rule, "Technical Specifications," 60 FR 36953 (July 19, 1995). The Commission indicated that the intent of these criteria can be utilized to identify the optimum set of administrative controls in the TS.

The Commission's policy statement provides that many of the existing TS limiting conditions for operation which do not satisfy these four specified criteria may be relocated to the Updated Safety Analysis Report (USAR), such that future changes could be made to these provisions pursuant to 10 CFR 50.59. Other requirements may be relocated to more appropriate documents (e.g. Security Plan, Quality Assurance (QA) plan, and Emergency Plan) and controlled by the applicable regulatory requirement. While the content of the TS administrative controls is specified in 10 CFR 50.36(c)(5), particular details of the administrative controls may be relocated to licensee-controlled documents where §50.59 or comparable regulatory controls exist.

Administrative controls in existing TS related to the review functions should be relocated to a licensee-controlled document that provides adequate control over changes to these provisions and which provides an appropriate change control mechanism. As such, these review provisions should be relocated to the Quality Assurance Program described or referenced in the facility's USAR and controlled pursuant to 10 CFR 50.54.

The licensee proposed to relocate some TS administrative control provisions to the Duke Power Company Quality Assurance Program Topical Report, Amendment 20. These particular TS provisions are not necessary to assure safe operation of the facility, given that the requirements in the QA program implement the Commission's regulations pertaining to these aspects as set forth below. As documented in the Final Policy Statement, these aspects constitute

requirements that can be relocated to the Quality Assurance plan and controlled by the applicable regulatory requirement. Such an approach would result in an equivalent level of regulatory authority while providing for a more appropriate change control process.

On this basis, the staff concludes that the relocated provisions are not required to be in the TS under 10 CFR 50.36 or §182a of the Act, and are not required to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety. In addition, the staff finds that sufficient regulatory controls exist under 10 CFR 50.54 to adequately control future modifications to these provisions. Accordingly, the staff has concluded that these requirements may be relocated from the TS to the respective licensee-controlled documents.

Specific Change Evaluations

1. Section 6.1.2 - Technical Review and Control

With respect to the review and approval of subjects within Section 6.1.2, Duke proposes to delete the titles of individuals where specified. The titles are to be replaced by a general statement that "Approvals shall be by the head of the appropriate site organization, the head of the appropriate station organization, the head of the appropriate site engineering organization, the head of the environmental organization, or an alternate as specified in other applicable regulatory documents or administrative controls." The Duke Power Quality Assurance Program Topical Report, Amendment 20 (QAP) provides amplifying details on the station organizational structure. QAP section 17.3.2.2 defines the final approval authority for station modifications. Section 17.3.2.14 of the QAP defines the approval authority for changes to station Technical Specifications and for station procedures. Future changes to the QAP are under the control of 10 CFR 50.54(a). Duke stated that there will be no decrease in the level of quality given to the review and approval activities. Further, personnel performing the reviews will continue to meet the same qualification requirements of ANSI N18.1-1971. The staff finds the use of generic titles in the TS acceptable as the QAP contains sufficient controls to ensure that the reviews are performed by equally qualified personnel.

Duke Power proposes to delete the requirement that the Manager, Safety Assurance shall assure that reports of Reportable Events are developed and transmitted to the Site Vice President, or designee, who approves such reports, and to the Director of the Nuclear Safety Review Board. The requirement was replaced with a statement that "such reports shall be reviewed by a knowledgeable individual/organization other than the individual/organization which prepared the report." The staff finds this change acceptable as we do not require that this subject be included in the TS.

Duke Power proposes to revise the statement that "The Manager, Safety Assurance shall assure the performance..." with a statement "A knowledgeable individual/organization shall review..." and delete