MAY 1, 1992

Docket Nos. 50-269, 50-270
and 50-287

Distribution
See next page

Mr. J. W. Hampton
Vice President, Oconee Site
Duke Power Company
P. O. Box 1439
Seneca, South Carolina 29679

Dear Mr. Hampton:

The Nuclear Regulatory Commission has issued the enclosed Amendment Nos. 193 , 193 and 190 to Facility Operating Licenses DPR-38, DPR-47, and DPR-55, respectively, for the Oconee Nuclear Station, Units 1, 2, and 3. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated January 2, 1992, as supplemented February 14, 1992.

The amendments revise the Administrative Controls Section to reflect a reorganization of the Duke Power Company (DPC). The reorganization essentially decentralizes the corporate management of nuclear activities to each of DPC's three nuclear site facilities, including the Oconee Site. The revisions to the TS also reflect and are complementary to revisions to the DPC Quality Assurance Topical Report. The review of this report, based upon the guidance of Standard Review Plan 17.3 as issued in August 1990, is being addressed as a separate action from the revision of the TS.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Leonard A. Wiens, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

9205200327 920501
PDR ADOCK 05000269
P PDR

Enclosures:
1. Amendment No. 193 to DPR-38
2. Amendment No. 193to DPR-47
3. Amendment No. 190to DPR-55
4. Safety Evaluation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

May 1, 1992

Docket Nos. 50-269, 50-270
and 50-287

Mr. J. W. Hampton
Vice President, Oconee Site
Duke Power Company
P. O. Box 1439
Seneca, South Carolina 29679

Dear Mr. Hampton:

SUBJECT: ISSUANCE OF AMENDMENTS - OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (TACS M82561, M82562, AND M82563)

The Nuclear Regulatory Commission has issued the enclosed Amendment Nos. 193 , 193 and 190 to Facility Operating Licenses DPR-38, DPR-47, and DPR-55, respectively, for the Oconee Nuclear Station, Units 1, 2, and 3. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated January 2, 1992, as supplemented February 14, 1992.

The amendments revise the Administrative Controls Section to reflect a reorganization of the Duke Power Company (DPC). The reorganization essentially decentralizes the corporate management of nuclear activities to each of DPC's three nuclear site facilities, including the Oconee Site. The revisions to the TS also reflect and are complementary to revisions to the DPC Quality Assurance Topical Report. The review of this report, based upon the guidance of Standard Review Plan 17.3 as issued in August 1990, is being addressed as a separate action from the revision of the TS.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Leonard A. Wiens, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 193 to DPR-38
2. Amendment No. 193 to DPR-47
3. Amendment No. 190 to DPR-55
4. Safety Evaluation

cc w/enclosures:
See next page

Mr. J. W. Hampton
Duke Power Company

Oconee Nuclear Station

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DATED: MAY 1, 1992

AMENDMENT NO. 193 TO FACILITY OPERATING LICENSE DPR-38 - Oconee Nuclear Station, Unit 1
AMENDMENT NO. 193 TO FACILITY OPERATING LICENSE DPR-47 - Oconee Nuclear Station, Unit 2
AMENDMENT NO. 190 TO FACILITY OPERATING LICENSE DPR-55 - Oconee Nuclear Station, Unit 3

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 193
License No. DPR-38

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Oconee Nuclear Station, Unit 1 (the facility) Facility Operating License No. DPR-38 filed by the Duke Power Company (the licensee) dated January 2, 1992, as supplemented February 14, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 3.B. of Facility Operating License No. DPR-38 is hereby amended to read as follows:

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P PDR

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 193 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: May 1, 1992

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-270

OCONEE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 193
License No. DPR-47

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Oconee Nuclear Station, Unit 2 (the facility) Facility Operating License No. DPR-47 filed by the Duke Power Company (the licensee) dated January 2, 1992, as supplemented February 14, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 3.B. of Facility Operating License No. DPR-47 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 193, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: May 1, 1992

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-287

OCONEE NUCLEAR STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 190
License No. DPR-55

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Oconee Nuclear Station, Unit 3 (the facility) Facility Operating License No. DPR-55 filed by the Duke Power Company (the licensee) dated January 2, 1992, as supplemented February 14, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 3.B. of Facility Operating License No. DPR-55 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 190, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: May 1, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 193

FACILITY OPERATING LICENSE NO. DPR-38

DOCKET NO. 50-269

AND

TO LICENSE AMENDMENT NO. 193

FACILITY OPERATING LICENSE NO. DPR-47

DOCKET NO. 50-270

AND

TO LICENSE AMENDMENT NO. 190

FACILITY OPERATING LICENSE NO. DPR-55

DOCKET NO. 50-287

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

| Remove Pages | Insert Pages |
|---|---|
| 6.1-1 | 6.1-1 |
| 6.1-1a | 6.1-1a |
| 6.1-2 | 6.1-2 |
| 6.1-3 | 6.1-3 |
| 6.1-3a | 6.1-3a(1) |
| -- | 6.1-3a(2) |
| 6.1-3b | 6.1-3b |
| 6.1-4 | 6.1-4 |
| 6.1-5 | 6.1-5 |
| 6.1-5a | 6.1-5a |
| 6.1-6 | 6.1-6 |
| 6.2-1 | 6.2-1 |
| 6.3-1 | 6.3-1 |

6.0 ADMINISTRATIVE CONTROLS

6.1 ORGANIZATION, REVIEW, AND AUDIT

6.1.1 Organization

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offset organizations shall include the positions for activities affecting the safety of the nuclear power plant.

6.1.1.1 Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the FSAR.

6.1.1.2 The Station Manager shall be responsible for overall plant safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant and shall delegate in writing the succession to this responsibility during his absence.

6.1.1.3 The Vice President, Oconee Nuclear Site, shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.

The Senior Vice President, Nuclear Generation Department will be the Senior Nuclear Executive and have corporate responsibility for overall nuclear safety.

6.1.1.4 Incorporated in the staff of the station shall be personnel meeting the minimum requirements encompassing the training and experience described in Section 4 of ANSI/ANS-3.1-1978, "Selection and Training of Nuclear Power Plant Personnel" except for the Radiation Protection Manager, the Operations Superintendent and the Shift Operations Manager.

The Radiation Protection Manager shall have a bachelor's degree in a science or engineering subject or the equivalent in experience, including some formal training in radiation protection, and shall have at least five years of professional experience in applied radiation protection of which three years shall be in applied radiation protection work in one of Duke Power Company's nuclear stations.

A qualified individual who does not meet the above requirements, but who has demonstrated the required radiation protection management capabilities and professional experience in applied radiation protection work at one of Duke Power Company's multi-unit nuclear stations, may be appointed to the position of Radiation Protection Manager by the Station Manager, based on the recommendations of

the Nuclear Technical Services Manager and as approved by the Site Vice President, Nuclear Operations.

The Operations Superintendent shall have a minimum of eight years of responsible nuclear or fossil station experience, of which a minimum of three years shall be nuclear station experience. A maximum of two years of the remaining five years of experience may be fulfilled by academic training, or related technical training, on a one-for-one time basis. The Operations Superintendent shall hold or have held a Senior Reactor Operator license.

The Shift Operations Manager shall have a minimum of eight years of responsible nuclear or fossil station experience, of which a minimum of three years shall be nuclear station experience. A maximum of two years of the remaining five years of experience may be fulfilled by academic training, or related technical training on a one-for-one time basis. The Shift Operations Manager shall hold a Senior Reactor Operator license.

6.1.1.5 The individuals who train the operating staff and those who carry out radiation protection and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

6.1.1.6 Minimum operating shift crew requirements shall be as specified in Table 6.1-1.

6.1.1.7 Retraining and replacement of station personnel shall be in accordance with Section 5.5 of the ANSI/ANS-3.1-1978, "Selection and Training of Nuclear Power Plant Personnel."

6.1.1.8 A training program for the fire brigade shall meet or exceed the requirements of Section 27 of the NFPA Code-1975, except that training sessions may be held quarterly.

6.1.1.9

The Shift Manager is an experienced SRO, who has been instructed in additional academic subjects, and will be assigned on-shift to provide the accident assessment capability.

The operating experience assessment function will be provided by the Station Safety Review Group.

6.1.2 Technical Review and Control

6.1.2.1 Activities

a. Procedures required by Technical Specification 6.4 and other procedures which affect station nuclear safety, and changes (other than editorial or typographical changes) thereto, shall be prepared by a qualified individual/organization. Each such procedure, or procedure change, shall be reviewed by an individual/group other than the individual/group which prepared the procedure, or procedure change, but who may be from the same organization as the individual/group which prepared the procedure, or procedure change. Such procedures and procedure changes may be approved for temporary use by two members of the station staff, at least one of whom holds a Senior Reactor Operator's License on the unit(s) affected. Procedures and procedure changes shall be approved prior to use or within seven days of receiving temporary approval by a group manager, superintendent/manager, or one of their designated direct reports.

b. Proposed changes to the Technical Specifications shall be prepared by a qualified individual/organization. The preparation of each proposed Technical Specifications change shall be reviewed by an individual/group other than the individual/group which prepared the proposed change, but who may be from the same organization as the individual/group which prepared the proposed change. Proposed changes to the Technical Specifications shall be approved by the Station Manager.

c. Proposed modifications to station nuclear safety-related structures, systems and components shall be designed by a qualified individual/organization. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modification. Proposed modifications to nuclear safety-related structures, systems, and components shall be approved prior to implementation by the Station Manager; or for the Station Manager by the Mechanical Superintendent, the Operations Superintendent, the I and E Superintendent, or the Work Control Superintendent, as previously designated by the Station Manager.

d. Individuals responsible for reviews performed in accordance with 6.1.2.1-.a, 6.1.2.1.b, and 6.1.2.1.c shall be members of the Site supervisory staff, previously designated by the Site Vice President to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated Site review personnel.

e. Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be reviewed by the Station Manager; or for the Station Manager by the Mechanical Superintendent, the Operations Superintendent, the I and E Superintendent, or the Work Control Superintendent, as previously designated by the Station Manager.

f. Incidents reportable pursuant to Technical Specification 6.6.2.1 and violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. Such reports shall be approved by the Manager, Safety Assurance and transmitted to the Site Vice President; and to the Director of the Nuclear Safety Review Board.

g. The Manager, Safety Assurance shall assure the performance of special reviews and investigations, and the preparation and submittal of reports thereon, as requested by the Site Vice President.

h. Deleted

i. Deleted

j. The Manager, Safety Assurance shall assure that an independent fire protection and loss prevention inspection and audit shall be performed annually utilizing qualified off-site personnel and that an inspection and audit by a qualified fire consultant shall be performed at intervals no greater than three years.

k. Unplanned onsite releases of radioactive material to the environs shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. Such reports shall be approved by the Manager, Safety Assurance and transmitted to the Site Vice President, or designee, and to the Director of the Nuclear Safety Review Board.

l. Licensee-initiated changes to the Offsite Dose Calculation Manual (ODCM) shall be documented and records of reviews performed shall be retained for the duration of the unit operating license. This documentation shall contain:

1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s); and,

2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.106, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations; and,

3) Shall become effective upon review and acceptance by the Station Manager and a qualified individual/organization; and,

4) Shall be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Semiannual Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.

m. Licensee-initiated changes to the Process Control Program shall be documented and records of reviews performed shall be retained for the duration of the unit license. This documentation shall contain:

1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s);

2) A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations; and,

3) Changes shall become effective upon review and acceptance by the Station Manager and a qualified individual/organization.

6.1.2.2 Records

Records of the above activities shall be maintained.

6.1.3 Nuclear Safety Review Board

6.1.3.1 Function

The NSRB shall function to provide independent review and audit of designated activities in the areas of:

a. Nuclear power plant operations
b. Nuclear Engineering
c. Chemistry and radiochemistry
d. Metallurgy
e. Instrumentation and control
f. Radiological safety
g. Mechanical and electrical engineering
h. Administrative control and quality assurance practices

6.1.3.2 Organization

a. The Director, members and alternate members of the NSRB shall be formally appointed by the Executive Vice President, Power Generation, and shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of five years technical experience, of which a minimum of three years shall be in one or more areas given in 6.1.3.1.

In special cases, candidates for appointment without an academic degree in engineering or a physical science may be qualified with a minimum of ten years experience in one of the areas specified in Specification 6.1.3.1.

b. The NSRB shall be composed of at least five members, including the Director, Members of the NSRB may be from the Nuclear Generation Department, from other departments within the Company or from external to the

Company. A maximum of one member of the NSRB may be from the Oconee Nuclear Site staff.

c. Consultants may be utilized by the NSRB to provide expert advice to the NSRB, as determined necessary by the Director of the NSRB.

d. Staff assistance may be provided to the NSRB in order to promote the proper, timely and expeditious performance of its functions.

e. The NSRB shall meet at least once per six months. The period between such meetings shall not exceed eight months.

f. A quorum of the NSRB shall consist of the Director, or his designated alternate, and at least two other NSRB members or alternate members. No more than a minority of the quorum shall have line responsibility for operation of Oconee Nuclear Station.

6.1.3.3 Subjects Requiring Review

The following subjects shall be reported to and reviewed by the NSRB:

a. The safety evaluations for (1) changes to procedures, equipment or systems, and (2) tests or experiments completed under the provisions of 10 CFR 50.59(a)(1) to verify that such actions did not constitute an unreviewed safety question.

b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in 10 CFR 50.59.

c. Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59.

d. Proposed changes in Technical Specifications or the Facility Operating Licenses.

e. Violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.

f. Significant operating abnormalities or deviations from normal and expected performance of station equipment that affect nuclear safety.

g. Incidents that are the subject of non-routine reports submitted to the Commission.

h. Quality Verification Department audits relating to station operations and actions taken in response to these audits.

6.1.3.4 Audits

Audits of Site activities shall be performed under the cognizance of the NSRB. These audits shall encompass:

a. The conformance of station operation to provisions contained within the Technical Specifications and applicable facility operating license conditions.

b. The performance, training and qualifications of the station staff.

c. The results of actions taken to correct deficiencies occurring in equipment, structures, systems or methods of operation that affect nuclear safety.

d. The performance of activities required by the quality assurance program to meet the criteria of Appendix B to 10 CFR 50.

e. The station emergency plan and implementing procedures.

f. The station security plan and implementing procedures.

g. Any other area of station operation considered appropriate by the NSRB or the Executive Vice President, Power Generation Department.

h. The station fire protection program and implementing procedures.

i. The Offsite Dose Calculation Manual and implementing procedures.

j. The Radiological Environmental Monitoring Program and the results thereof.

k. The Process Control Program and implementing procedures for solidification of radioactive wastes.

l. The performance of activities required by the Quality Assurance Program to meet the criteria of Regulatory Guide 1.21 Revision 1, June 1974 and Regulatory Guide 4.1 Revision 1, April 1975.

6.1.3.5 Responsibilities and Authorities

a. The NSRB shall report to and advise the Executive Vice President, Power Generation on those areas of responsibility specified in Specifications 6.1.3.3 and 6.1.3.4.

b. Minutes shall be prepared and forwarded to the Senior Vice President, Nuclear Generation, and to the Executive Vice President, Power Generation, within 14 days following each formal meeting of the NSRB.

c. Records of activities performed in accordance with Specifications 6.1.3.3 and 6.1.3.4 shall be maintained.

d. Audit reports encompassed by Section 6.1.3.4 shall be forwarded to the Senior Vice President, Nuclear Generation and to the Executive Vice President, Power Generation, and to the management position responsible for the areas audited within 30 days of completion of each audit.

TABLE 6.1-1
MINIMUM OPERATING SHIFT REQUIREMENTS
(With Fuel in the Three Reactor Vessels)

| | One Unit Operating* | Two Units Operating* | All Units Operating* | All Units Shutdown |
|---|---|---|---|---|
| Shift Supervisor (SRO) | 1 | 1 | 1 | 1 |
| Additional SRO | 1 | 2** | 2 | NONE |
| Shift Manager (SRO) | 1 | 1 | 1 | NONE |
| Reactor Operator | 4 | 5** | 5 | 3 |
| Nuclear Equipment Operator | 4 | 5 | 5 | 5 |

* Above cold shutdown
** Only one SRO and four Reactor Operators required if both units are operated from one Control Room.

6.2 ACTION TO BE TAKEN IN THE EVENT OF A REPORTABLE OCCURRENCE

6.2.1 The Manager, Safety Assurance shall assure that any reportable event is promptly investigated.

6.2.2 The Site Vice President shall be notified of any reportable event. A written report shall be prepared which describes the circumstances leading up to and resulting from the incident and shall recommend appropriate action to prevent or minimize the probability of a recurrence. The report shall be submitted to the Site Vice President, and the Director of the Nuclear Safety Review Board.

6.2.3 The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.6.2 and 10 CFR 50.73.

6.3 ACTION TO BE TAKEN IN THE EVENT A SAFETY LIMIT IS EXCEEDED

6.3.1 If a safety limit is exceeded, the reactor shall be shut down immediately and maintained in a safe shutdown condition until the Commission authorizes resumption of operation.

6.3.2 The violation of a safety limit shall be reported to the Commission, the Site Vice President and the Director of the Nuclear Safety Review Board.

6.3.3 A report shall be prepared which describes (1) applicable circumstances preceding the violation, (2) effects of the violation upon structures, systems, or components, and (3) corrective action taken to prevent recurrence. The report shall be reviewed by the Operations Superintendent and the Station Manager. The report shall be submitted to the Site Vice President and the Director of the Nuclear Safety Review Board.

6.3.4 A report of the violation, with appropriate analyses and corrective action to prevent recurrence shall be submitted to the Commission within 10 days of the violation.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 193 TO FACILITY OPERATING LICENSE DPR-38

AMENDMENT NO. 193 TO FACILITY OPERATING LICENSE DPR-47

AMENDMENT NO. 190 TO FACILITY OPERATING LICENSE DPR-55

DUKE POWER COMPANY

OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3

DOCKET NOS. 50-269, 50-270, AND 50-287

## 1.0 INTRODUCTION

By letter dated January 2, 1992, as supplemented February 14, 1992, the Duke Power Company (the licensee) submitted a request for changes to the Oconee Nuclear Station, Units 1, 2, and 3 Technical Specifications (TS). The requested changes would revise the Administrative Controls section to reflect a reorganization of the Duke Power Company (DPC). The reorganization essentially decentralizes the corporate management of nuclear activities to each of DPC's three nuclear site facilities, including the Oconee Site. The revisions to the TS also reflect and are complementary to revisions to the DPC Quality Assurance Topical Report. The review of this report, based upon the guidance of Standard Review Plan 17.3 as issued in August 1990, is being addressed as a separate action from the revision of the TS.

## 2.0 EVALUATION

### 2.1 Administrative Controls

a. Section 6.1.1 - Organization - The designation "facility" has been changed to "plant", and the designation "health physics" has been changed to "radiation protection". The Vice President, Oconee Nuclear Site, now has responsibility for overall plant nuclear safety and the Senior Vice President, Nuclear Generation, will have responsibility for overall nuclear safety. In addition, the titles of Station Health Physicist, Superintendent of Operations, Operating Engineer, and System Health Physicist have been changed to Radiation Protection Manager, Operations Superintendent, Shift Operations Manager, and Nuclear Technical Services Manager, respectively. The position of Shift Manager has been created that subsumes the function of the Shift Technical Advisor (STA). This change in title has also been made in Table 6.1.1.

   The NRC staff finds these changes acceptable as they reflect the revised organization and reassignment of responsibilities, and the function of the STA has been retained in the position of Shift Manager.

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b. Section 6.1.2 - Technical Review and Control - The approval authority for proposed modifications to unit nuclear safety-related structures, systems, and components, and proposed tests and experiments which affect nuclear safety have been revised to conform to the revised organization. Other titles have been changed throughout this section to reflect the revised organization and the reassignment of responsibilities, and the designation "station" has been changed to "site".

c. Section 6.1.3 - Nuclear Safety Review Board (NSRB) - The titles of Vice President, Nuclear Production Department; Nuclear Production Department; Quality Assurance Department; and Executive Vice President, Nuclear Production Department have been replaced to reflect the revised organization, and the designation "station" has been changed to "site". In Subsection 6.1.3.2, the qualification requirements for NSRB members has been revised to allow, in special cases, an individual with ten years experience in a specific technical area. In Subsection 6.1.3.4, Audits, the licensee has relocated the audit frequency from the audits required by this subsection. Audit frequency requirements are now addressed in the Duke Quality Assurance Topical Report and are performance-based on the safety significance and extent of the activities except those for the Emergency and Security Plans as discussed below.

   The NRC staff finds these changes acceptable as they reflect the revised organization and reassignment of responsibilities, the appropriate acceptance criteria of Section 13.4 of NUREG-0800, the Standard Review Plan, and the commitment to performance-based audits in DPC's revised Duke Quality Assurance Topical Report, a document controlled in accordance with 50.54(a).

d. Section 6.2 - Action To Be Taken in the Event Of A Reportable Occurrence - The titles in this section have been revised to reflect the revised organization.

   The NRC staff finds these changes acceptable as they reflect the revised organization.

e. Section 6.3 - Action To Be Taken In The Event A Safety Limit Is Exceeded - The titles in this section have been revised to reflect the revised organization.

   The NRC staff finds these changes acceptable as they reflect the revised organization.

We find the above changes to the Administrative Controls Section of the Technical Specifications for the Oconee Nuclear Station, as described in the letters dated January 2, 1992, and February 14, 1992, acceptable.

## 2.2 Security and Emergency Plan

The requirements in TS 6.1.2.1.h for review of the security program and implementing procedures and in TS 6.1.2.1.i for review of the station emergency plan and implementing procedures are removed from the TS and are relocated to the respective security and emergency plans. Once they are included in the respective plans, further changes in these review requirements must be made in accordance with 10 CFR 50.54(P) for the security plan and with 10 CFR 50.54(q) for the emergency plan. The NRC staff concludes that, in conjunction with this change to the plans, the extensive requirements for emergency planning in 10 CFR 50.47 and 50.54 and for security in 10 CFR 50.54 and 73.55 for drills, exercises, testing and the maintenance of the programs provides adequate assurance that the objective of the previous TS 6.1.2.1.h and TS 6.1.2.1.i for a periodic review of the programs and changes to the programs will be met. Therefore, this change is acceptable.

The requirements of TS 6.1.3.4.e and .f for an audit on a twelve month frequency by the corporate Nuclear Safety Review Board (NSRB) of the emergency and security plans and implementing procedures is modified by deleting the specification of the twelve month frequency. This results in no change in this regulatory requirement since the frequency of such independently conducted reviews is also specified in 10 CFR 50.54(t) for the emergency plan, in 10 CFR 50.54(P)(3) for the safeguards contingency plan and in 10 CFR 73.55(g)(4) for the security program. This change is acceptable.

## 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the South Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

## 4.0 ENVIRONMENTAL CONSIDERATION

The amendments relate to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

## 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

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Date: May 1, 1992