



50-269/270/287

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

September 19, 1997

Mr. W. R. McCollum  
Vice President, Oconee Site  
Duke Energy Corporation  
P. O. Box 1439  
Seneca, SC 29679

SUBJECT: AMENDMENT CORRECTION - OCONEE NUCLEAR STATION, UNITS 1, 2,  
AND 3 (TAC NOS. M97125, M97126, AND M97127)

Dear Mr. McCollum:

On September 15, 1997, the Commission issued Amendment Nos. 225, 225, and 222 to Facility Operating Licenses DPR-38, DPR-47, and DPR-55, respectively, for the Oconee Nuclear Station Units 1, 2, and 3. The amendments consisted of changes to the Technical Specifications (TS) in response to your application dated October 30, 1996, and supplements dated April 22, July 2, September 3, and September 4, 1997. The amendments revised the Reactor Building Structural Integrity TS regarding the tendon surveillance program.

By letter dated September 18, 1997, you informed the staff of an oversight of an editorial nature that had been made in your September 3, 1997, supplement that had been carried over into the approved amendments. The typographical error consisted of removal of a portion of item number 5 from the bottom of TS page 4.4-15. This error was introduced only in your final production of the revised TS page 4.4-15 that was included in your supplement dated September 3, 1997. The error was not included in the original submittal, a supplement dated April 22, 1997, or in the marked-up TS page contained in the September 3, 1997, supplement. In order to re-insert the desired information, you found it necessary to insert it at the top of page 4.4-16 and move the existing TS requirements onto subsequent pages. Therefore, pages 4.4-16, 4.4-16a, and 4.4-16b were affected and were enclosed. Note that this is contrary to the submittal letter description, which stated that changes were being made to pages 4.4-16 through 4.4-19. The correct page numbers were, however, submitted.

During our review of the technical details of the amendment and preparation of the safety evaluation, the staff did not detect that the information was missing because our analysis was based on page 4.4-15 that contained the correct information. As a result, the error has no

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Mr. William R. McCollum

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September 19, 1997

affect on our evaluation of the TS change and had no affect on the Notice published in the Federal Register for this amendment. Correction of the typographical error is, therefore, acceptable. The corrected TS pages are attached.

Sincerely,



David E. LaBarge, Senior Project Manager  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270,  
and 50-287

Enclosure: Amendment Correction

cc w/encl: See next page

Mr. William R. McCollum

- 2 -

September 19, 1997

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Sincerely,

ORIGINAL SIGNED BY:

David E. LaBarge, Senior Project Manager  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270,  
and 50-287

Enclosure: Amendment Correction

cc w/encl: See next page

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CORRECTION TO LICENSE AMENDMENT NO. 225

FACILITY OPERATING LICENSE NO. DPR-38

DOCKET NO. 50-269

AND

TO LICENSE AMENDMENT NO. 225

FACILITY OPERATING LICENSE NO. DPR-47

DOCKET NO. 50-270

AND

TO LICENSE AMENDMENT NO. 222

FACILITY OPERATING LICENSE NO. DPR-55

DOCKET NO. 50-287

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove

4.4-16  
4.4-16a  
4.4-16b

Insert

4.4-16  
4.4-16a  
4.4-16b

5. If the measured prestressing forces from consecutive surveillances for the same tendon, or tendons in a group, indicate a trend of prestress loss larger than expected and the resulting prestressing forces are likely to be less than the minimum required for the group before the next scheduled surveillance, additional lift-off testing shall be done so as to determine the cause and extent of such occurrence. The condition shall be considered as an indication of abnormal degradation of the reactor building. In the event of an indication of abnormal degradation, refer to Technical Specification 3.6.7.2.

b. Perform tendon detensioning, inspections, and material tests on a tendon from each group. A randomly selected tendon from each group shall be completely detensioned in order to identify any broken or damaged wires and to determine the following conditions over the entire length of a removed tendon wire sample (this wire sample should be the broken wire if so identified):

1. Tendon wires are free of corrosion, cracks, and damage, and
2. Minimum tensile strength of 240,000 psi (guaranteed ultimate tensile strength of the wire material) exists for at least three wire samples (one from each end and one at mid-length) cut from the removed wire.

Failure to meet requirements of 4.4.2.2.b shall be considered as an indication of abnormal degradation of the reactor building. In the event of an indication of abnormal degradation, refer to Technical Specification 3.6.7.2.

c. Retension tendons detensioned for inspection to a force at least equal the force recorded prior to detensioning or the predicted value at the time of inspection, whichever is greater, but do not exceed 70% of the guaranteed ultimate tensile strength of the tendon wire material. Tendon seating force tolerance shall be -0 / +6%. During retensioning of these tendons, change in load versus elongation should be measured at varying levels of force. The following table provides levels of force, pressure, and elongation at which measurements should be taken:

	Force (Kips)	Pressure (psi)	Elongation (In)
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PTF

Step 1  
Step 2  
LOF  
OSF

Where:

Total Elongation (actual) = (LOF-PTF) Elongation  
PTF - Pretensioning Force necessary to bring the tendon into a slightly stressed condition to remove slack and seat the buttonheads.

Step 1-2 - An intermediate force approximately equally spaced between PTF and LOF.

LOF - Lock Off Force at which the tendon is seated on the shims.

OSF - Overstress Force at which the maximum elongation is measured.

If the elongation corresponding to a specific load differs by more than 10% from that recorded during the original installation, an investigation should be made to ensure that the difference is not related to wire failures or slip of wires at anchorages. This condition shall be considered as an indication of abnormal degradation of the reactor building. In the event of an indication of abnormal degradation, refer to Technical Specification 3.6.7.2.

d. Verify acceptability of the sheathing filler grease by assuring that:

1. No free water is present and no changes in the presence or physical appearance of the sheathing filler grease occur.
2. Amount of grease replaced does not exceed 5% of the net duct volume when injected at +/-10% of the specified installation pressure.
3. Minimum grease coverage exists for the different parts of the anchorage system.
4. Reactor building exterior surface does not exhibit grease leakage that could affect reactor building integrity.
5. Chemical properties of the sheathing filler grease are within the following tolerance limits:

Water Content	0 - 10% (by dry wt.)
Chlorides	0 - 10 ppm
Nitrates	0 - 10 ppm
Sulfides	0 - 10 ppm

OCONEE - UNITS 1, 2, & 3 4.4-16a

Amendment No. 225 (Unit 1)  
Amendment No. 225 (Unit 2)  
Amendment No. 222 (Unit 3)

Reserve Alkalinity  
(Base Numbers)

> 50% of installed  
value;  
> 0 (for older  
grease)

Failure to meet requirements of 4.4.2.2.d shall be considered as an indication of potential abnormal degradation of the reactor building. In the event of an indication of abnormal degradation, refer to Technical Specification 3.6.7.2.

#### 4.4.2.3 End Anchorages and Adjacent Concrete Surfaces

As an assurance of the structural integrity of the reactor building(s), tendon anchorage assembly hardware (such as bearing plates, stressing washers, wedges, and buttonheads) of all tendons selected for inspection shall be visually examined. Tendon anchorages selected for inspection shall be visually examined to the extent practical without dismantling the load bearing components of the anchorages. Top and bottom grease caps of all vertical tendons shall be visually inspected to detect grease leakage or grease cap deformations. The surrounding concrete should also be checked visually for indication of any abnormal condition.

Significant grease leakage, grease cap deformation or abnormal concrete condition shall be considered as an indication of abnormal degradation of the reactor building. In the event of an indication of abnormal degradation, refer to Technical Specification 3.6.7.2.

#### 4.4.2.4 Reactor Building Surfaces

The exterior surface of the reactor building(s) should be visually examined to detect areas of large spall, severe scaling, D-cracking in an area of 25 sq. ft. or more, other surface deterioration or disintegration, or grease leakage. Each of these conditions can be considered as evidence of abnormal degradation of structural integrity of the reactor building(s). This inspection may be performed prior to the Type A containment leakage rate test (Refer to Technical Specification 4.4.1). In the event of an indication of abnormal degradation, refer to Technical Specification 3.6.7.2.