



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

October 19, 1987

Docket Nos.: 50-269, 50-270
and 50-287

Mr. H. B. Tucker, Vice President
Nuclear Production Department
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

Subject: Issuance of Amendment Nos. 163, 163, and 160 to Facility Operating Licenses DPR-38, DPR-47, and DPR-55 - Oconee Nuclear Station, Units 1, 2, and 3 (TAC Nos. 60135/60136/60137)

The Nuclear Regulatory Commission has issued the enclosed Amendment Nos. 163, 163, and 160 to Facility Operating Licenses Nos. DPR-38, DPR-47 and DPR-55 for the Oconee Nuclear Station, Units 1, 2, and 3. These amendments consist of changes to the Station's common Technical Specifications (TSs) in response to your request dated August 22, 1985, as supplemented on February 11, 1986.

The amendments revise the TSs to correct a typographical error, delete an expired footnote, update the station organization by adding the Station Services and Integrated Scheduling areas and provide clarity and consistency through different wording. The change requested to TS 6.2.2 will be addressed in separate correspondence.

We request that additional attention be given to future amendment requests on the accuracy of the TS pages. During our review, we found an excessive number of typographical errors on text that was not being proposed for revision. Some of the errors were serious in that they had the potential of inadvertently changing operating limits.

8710270311 871019
PDR ADDCK 05000269
P PDR

Mr. H.B. Tucker

- 2 -

October 19, 1987

A copy of our Safety Evaluation is also enclosed. Notice of issuance of the enclosed amendments will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

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Helen N. Pastis, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II

Enclosures:

1. Amendment No. 163 to DPR-38
2. Amendment No. 163 to DPR-47
3. Amendment No. 160 to DPR-55
4. Safety Evaluation

cc w/enclosures: See next page

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04/24/87

PDII-3/DRP-I/II
HPastis
04/29/87

^{DSH}
PDII-3/DRP-I/II
DHood, Acting PD
10/16/87

Mr. H. B. Tucker
Duke Power Company

Oconee Nuclear Station
Units Nos. 1, 2 and 3

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Honorable James M. Phinney
County Supervisor of Oconee County
Walhalla, South Carolina 29621

DATED: October 19, 1987

AMENDMENT NO. 163 TO FACILITY OPERATING LICENSE DPR-38 - Oconee Nuclear Station, Unit 1
AMENDMENT NO. 163 TO FACILITY OPERATING LICENSE DPR-47 - Oconee Nuclear Station, Unit 2
AMENDMENT NO. 160 TO FACILITY OPERATING LICENSE DPR-55 - Oconee Nuclear Station, Unit 3

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 163
License No. DPR-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 1 (the facility) Facility Operating License No. DPR-38 filed by the Duke Power Company (the licensee) dated August 22, 1985, as supplemented on February 11, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter 1;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 3.B of Facility Operating License No. DPR-38 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 163, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Darl S. Hood, Acting Director
Project Directorate II-3
Division of Reactor Projects - I/II

Attachment:
Technical Specification
Changes

Date of Issuance: October 19, 1987

PDII-3/DRP-I/II
MDuncan/rad
04/24/87

PDII-3/DRP-I/II
HPastis
04/29/87

*copy notated memo to SE
check STATE & SECY
ref. issuance*
OGC-Bethesda
M/King
04/6/87

DSH
PDII-3/DRP-I/II
DHood, Acting PD
10/16/87



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY
DOCKET NO. 50-270
OCONEE NUCLEAR STATION, UNIT 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 163
License No. DPR-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 2 (the facility) Facility Operating License No. DPR-47 filed by the Duke Power Company (the licensee) dated August 22, 1985, as supplemented on February 11, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter 1;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 3.B of Facility Operating License No. DPR-47 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 163, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Darl S. Hood, Acting Director
Project Directorate II-3
Division of Reactor Projects - I/II

Attachment:
Technical Specification
Changes

Date of Issuance: October 19, 1987

PDII-3/DRP-I/II
MDuncan/rad
04/24/87

PDII-3/DRP-I/II
HPastis
04/24/87

*1444 noted revision to SE
check SMTC & SECY
bef. issuance*
OGC-Bethesda
in process
04/26/87

DSH
PDII-3/DRP-I/II
DHood, Acting PD
10/16/87



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-287

OCONEE NUCLEAR STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 160
License No. DPR-55

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 3 (the facility) Facility Operating License No. DPR-55 filed by the Duke Power Company (the licensee) dated August 22, 1985 as supplemented on February 11, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter 1;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 3.B of Facility Operating License No. DPR-55 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 160, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

151

Darl S. Hood, Acting Director
Project Directorate II-3
Division of Reactor Projects - I/II

Attachment:
Technical Specification
Changes

Date of Issuance: October 19, 1987

PDII-3/DRP-I/II
MDuncan/rad
04/24/87

PDII-3/DRP-I/II
HPastor
04/21/87

Handwritten: 151 requested revision to SE check STATE ESCRY ref. issuance
OGC-Bethesda
M. Jones
04/16/87

PSH
PDII-3/DRP-I/II
DHood, Acting PD
10/16/87

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 163 TO DPR-38

AMENDMENT NO. 163 TO DPR-47

AMENDMENT NO. 160 TO DPR-55

DOCKET NOS. 50-269, 50-270, AND 50-287

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment numbers and contain vertical lines indicating the area of change.

Remove Pages

3.3-3
6.1-1
6.1-1a
6.1-2
6.1-3
6.2-1

Insert Pages

3.3-3
6.1-1
6.1-1a
6.1-2
6.1-3
6.2-1

- b. The BWST shall contain a minimum level of 46 feet of water having a minimum concentration of 1835*ppm boron at a minimum temperature of 50°F. The manual valve, LP-28, on the discharge line shall be locked open. If these requirements are not met, the BWST shall be considered unavailable and action initiated in accordance with Specification 3.2.

3.3.5 Reactor Building Cooling (RBC) System

- a. Prior to initiating maintenance on any component of the RBC system, the redundant component shall be tested to assure operability.
- b. When the RCS, with fuel in the core, is in a condition with pressure equal to or greater than 350 psig or temperature equal to or greater than 250°F and subcritical:
 - (1) Two independent RBC trains, each comprised of an RBC fan; associated cooling unit, and associated ESF valves shall be operable.
 - (2) Tests or maintenance shall be allowed on any component of the RBC system provided one train of the RBC and one train of the RBS are operable. If the RBC system is not restored to meet the requirements of Specification 3.3.5.b(1) above within 24 hours, the reactor shall be placed in a condition with RCS pressure below 350 psig and RCS temperature below 250°F within an additional 24 hours.
- c. When the reactor is critical:
 - (1) In addition to the requirements of Specification 3.3.5.b(1) above, the remaining RBC fan, associated cooling unit, and associated ESF valves shall be operable.
 - (2) Tests or maintenance shall be allowed on one RBC train * under either of the following conditions:
 - (a) One RBC train may be out of service for 24 hours.
 - (b) One RBC train may be out of service for 7 days provided both RBS trains are operable.
 - (c) If the inoperable RBC train is not restored to meet the requirements of Specification 3.3.5.c(1) within the time permitted by Specification 3.3.5.c(2) (a) or (b), the reactor shall be placed in a hot shutdown condition within 12 hours. If the requirements of Specification 3.3.5.c(1) are not met within an additional 24 hours following hot shutdown, the reactor shall be placed in a condition with RCS pressure below 350 psig and RCS temperature below 250°F within an additional 24 hours.

6.0 ADMINISTRATIVE CONTROLS

6.1 ORGANIZATION, REVIEW, AND AUDIT

6.1.1 Organization

6.1.1.1 The Station Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.1.2 In all matters pertaining to actual operation and maintenance and to these Technical Specifications, the Station Manager shall report to and be directly responsible to the Vice President, Nuclear Production Department, through the General Manager, Nuclear Stations. The organization is shown in Figure 6.1-2.

6.1.1.3 The station organization for Operations, Technical Services, Maintenance, Station Services, and Integrated Scheduling shall be functionally as shown in Figure 6.1-1. Minimum operating shift requirements are specified in Table 6.1-1.

6.1.1.4 Incorporated in the staff of the station shall be personnel meeting the minimum requirements encompassing the training and experience described in Section 4 of ANSI/ANS-3.1-1978, "Selection and Training of Nuclear Power Plant Personnel" except for the Site Health Physicist, the Superintendent of Operations and the Operating Engineer.

The Site Health Physicist shall have a bachelor's degree in a science or engineering subject or the equivalent in experience, including some formal training in radiation protection, and shall have at least five years of professional experience in applied radiation protection of which three years shall be in applied radiation protection work in one of Duke Power Company's nuclear stations.

A qualified individual who does not meet the above requirements, but who has demonstrated the required radiation protection management capabilities and professional experience in applied radiation protection work at one of Duke Power Company's multi-unit nuclear stations, may be appointed to the position of Site Health Physicist by the Station Manager, based on the recommendations of the System Health Physicist and as approved by the General Manager, Nuclear Stations.

The Superintendent of Operations shall have a minimum of eight years of responsible nuclear or fossil station experience, of which a minimum of three years shall be nuclear station experience. A maximum of two years of the remaining five years of experience may be fulfilled by academic training, or related technical training, on a one-for-one time basis. The Superintendent of Operations shall hold or have held a Senior Reactor Operator license.

The Operating Engineer shall have a minimum of eight years of responsible nuclear or fossil station experience, of which a minimum of three years shall be nuclear station experience. A maximum of two years of the remaining five years of experience may be fulfilled by academic training, or related technical training on a one-for-one time basis. The Operating Engineer shall hold a Senior Reactor Operator license.

- 6.1.1.5 Retraining and replacement of station personnel shall be in accordance with Section 5.5 of the ANSI/ANS-3.1-1978, "Selection and Training of Nuclear Power Plant Personnel."
- 6.1.1.6 A training program for the fire brigade shall meet or exceed the requirements of Section 27 of the NFPA Code-1975, except that training sessions may be held quarterly.
- 6.1.1.7 The two functions of the Shift Technical Advisor, namely accident assessment and operating experience assessment, are fulfilled in the following manner:
 - a. An experienced SRO, who has been instructed in additional academic subjects, will be assigned on-shift to provide the accident assessment capability.
 - b. The operating experience assessment function will be provided by the Station Safety Review Group.

6.1.2 Technical Review and Control

6.1.2.1 Activities

- a. Procedures required by Technical Specification 6.4 and other procedures which affect station nuclear safety, and changes (other than editorial or typographical changes) thereto, shall be prepared by a qualified individual/organization. Each such procedure, or procedure change, shall be reviewed by an individual/group other than the individual/group which prepared the procedure, or procedure change, but who may be from the same organization as the individual/group which prepared the procedure, or procedure change. Such procedures and procedure changes may be approved for temporary use by two members of the station staff, at least one of whom holds a Senior Reactor Operator's License on the unit(s) affected. Procedures and procedure changes shall be approved prior to use or within seven days of receiving temporary approval for use by the Station Manager; or by the Operating Superintendent, the Technical Services Superintendent, the Superintendent of Integrated Scheduling, the Station Services Superintendent (for fire protection and security related procedures only), or the Maintenance Superintendent, as previously designated by the Station Manager.
- b. Proposed changes to the Technical Specifications shall be prepared by a qualified individual/organization. The preparation of each proposed Technical Specifications change shall be reviewed by an individual/group other than the individual/group which prepared the proposed change, but who may be from the same organization as the individual/group which prepared the proposed change. Proposed changes to the Technical Specifications shall be approved by the Station Manager.
- c. Proposed modifications to station nuclear safety-related structures, systems and components shall be designed by a qualified individual/organization. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modification. Proposed modifications to station nuclear safety-related structures, systems and components shall be approved prior to implementation by the Station Manager; or by the Operating Superintendent, the Technical Services Superintendent, the Superintendent of Integrated Scheduling, or the Maintenance Superintendent, as previously designated by the Station Manager.
- d. Individuals responsible for reviews performed in accordance with 6.1.2.1.a, 6.1.2.1.b, and 6.1.2.1.c shall be members of the station supervisory staff, previously designated by the Station Manager to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated station review personnel.
- e. Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be reviewed by the Station Manager; or by the Operating Superintendent, the Technical Services Superintendent, the Maintenance Superintendent, or the

Superintendent of Integrated Scheduling, as previously designated by the Station Manager.

- f. Incidents reportable pursuant to Technical Specification 6.6.2.1 and violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. Such reports shall be approved by the Station Manager and transmitted to the Vice President, Nuclear Production Department, or his designee; and to the Director of the Nuclear Safety Review Board.
- g. The Station Manager shall assure the performance of special reviews and investigations, and the preparation and submittal of reports thereon, as requested by the Vice President, Nuclear Production Department.
- h. The station security program, and implementing procedures, shall be reviewed at least once per 12 months. Changes determined to be necessary, as a result of such review shall be approved by the Station Manager or Station Services Superintendent and transmitted to the Vice President, Nuclear Production Department, or his designee; and to the Director of the Nuclear Safety Review Board.
- i. The station emergency plan, and implementing procedures, shall be reviewed at least once per 12 months. Changes determined to be necessary as a result of such review shall be approved by the Station Manager and transmitted to the Vice President, Nuclear Production Department, or his designee; and the Director of the Nuclear Safety Review Board.
- j. The Station Manager shall assure that an independent fire protection and loss prevention inspection and audit shall be performed annually utilizing qualified off-site personnel and that an inspection and audit by a qualified fire consultant shall be performed at intervals no greater than three years.
- k. Unplanned onsite releases of radioactive material to the environs shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. Such reports shall be approved by the Station Manager and transmitted to the Vice President, Nuclear Production Department, or designee, and to the Director of the Nuclear Safety Review Board.
- l. Proposed changes to the Offsite Dose Calculation Manual (ODCM) shall be prepared by a qualified individual/organization. Each proposed change shall be reviewed by an individual/group other than the individual group which prepared the proposed change, but who may be from the same organization as the individual/group which prepared the proposed change. Proposed changes to the ODCM shall be approved by the Station Manager prior to implementation.

6.1.2.2 Records

Records of the above activities shall be maintained.

6.2 ACTION TO BE TAKEN IN THE EVENT OF A REPORTABLE OCCURRENCE

6.2.1 The Station Manager shall assure that any reportable event is promptly investigated.

6.2.2 The Vice President, Nuclear Production Department shall be notified of any reportable event. A written report shall be prepared which describes the circumstances leading up to and resulting from the incident and shall recommend appropriate action to prevent or minimize the probability of a recurrence. The report shall be submitted to the Vice President, Nuclear Production Department, and the Director of the Nuclear Safety Review Board.

6.2.3 The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.6.2 and 10 CFR 50.73.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 163 TO FACILITY OPERATING LICENSE NO. DPR-38

AMENDMENT NO. 163 TO FACILITY OPERATING LICENSE NO. DPR-47

AMENDMENT NO. 160 TO FACILITY OPERATING LICENSE NO. DPR-55

DUKE POWER COMPANY

OCONEE NUCLEAR STATION, UNITS 1, 2, and 3

DOCKET NOS. 50-269, 50-270 AND 50-287

I. INTRODUCTION

By application dated August 22, 1985, as supplemented on February 11, 1986, Duke Power Company (the licensee) requested amendments to Facility Operating Licenses Nos. DPR-38, DPR-47 and DPR-55 for the Oconee Nuclear Station, Units 1, 2, and 3. These amendments would correct a typographical error, delete an expired footnote, update the station organization by adding the Station Services and Integrated Scheduling areas and provide clarity and consistency through different wording.

The February 11, 1986 submittal provided supplemental information and contained changes which reduced the review and/or approval responsibility of the Station Services Superintendent from that initially proposed. This submittal does not significantly alter the action noticed in the Federal Register on October 23, 1985 and does not affect the staff's proposed no significant hazards determination.

II. EVALUATION

2.1 T.S. 3.3.5.c(2)(b)-Deletion of Footnote and Correction of a
Typographical Error (Page 3.3-3)

A footnote would be deleted because it expired on April 20, 1985. The footnote referred to "3A" reactor building cooling (RBC) train, and authorized a one-time extension of inoperability to allow repair, provided both reactor building spray (RBS) trains are operable and that the "3A" RBC train is returned to service no later than 11:59 p.m., April 20, 1985. The proposed revision would delete this outdated footnote.

In the same sentence, the licensee is also proposing to correct a typographical error (from C to S). Therefore, both trains of the RBS system (not RBC) are required to be operable. This does not change the operability requirements of the systems but refers to the correct one and corrects a typographical error for the sentence to read, "One RBC train may be out of service for 7 days provided both RBS trains are operable."

2.2 TSs 6.1.1.3, 6.1.2.1.a, c, and e - Adding Station Services and Integrated Scheduling (Pages 6.1-1, 6.1-2 and 6.1-3)

The licensee is proposing to add to TS 6.1.1.3, Station Services and Integrated Scheduling on the station organization.

In TSs 6.1.2.1.a, c and e, the licensee is proposing to add the Superintendents of Integrated Scheduling and of Station Services to allow them to approve safety-related procedures, structures, tests and experiments. In the February 11, 1986 supplement, the licensee explained the minimum qualifications required for the Superintendent of Station Services and the Superintendent of Integrated Scheduling. The requirements for these two superintendents are equivalent to those of other station superintendents with comparable responsibilities.

TS 6.1.2.1.a would be changed to include the Station Services Superintendent and to allow this staff member to approve fire protection and security related procedures only and if so designated by the Station Manager. The Superintendent of Integrated Scheduling would also be included to allow him to review and approve safety-related procedures and others as required by TS 6.4.

TS 6.1.2.1.c would be revised to include the Superintendent of Integrated Scheduling to allow him to approve proposed modifications to station nuclear safety-related structures, systems and components, and if so designated by the Station Manager.

TS 6.1.2.1.e would be revised to include the Superintendent of Integrated Scheduling to allow him to review proposed tests and experiments which affect station nuclear safety and are not addressed in the Final Safety Analysis Report (FSAR) or TSs, and if so designated by the Station Manager.

We have reviewed the experience requirements for these two new positions and they seem to be similar to those required for other superintendents of similar responsibilities. Based on this, we find these proposed changes acceptable.

2.3 TS 6.1.1.4 - Relocating First Paragraph (Page 6.1-1a)

Because of changes to page 6.1-1, the last paragraph of TS 6.1.1.4 would be relocated in its entirety to page 6.1.1a.

2.4 TS 6.1.2.1 h and i - Clarifying the Word Annually and Including the Superintendent of Station Services

TS 6.1.2.1.h requires that the Station Manager approve all changes to the station security program and implementing procedures. The proposed revision would allow the Station Services Superintendent to also approve these changes.

TSs 6.1.2.1 h and i require annual review of the station security program, its implementing procedures, the station emergency plan and its implementing procedures. The proposed revisions would change the word from annually to once per twelve months to make the TSs consistent throughout and with other manuals.

2.5 TS 6.2.1 - Reportable Events (Page 6.2-1)

TS 6.2.1 requires that the Station Manager promptly investigate any reportable event. The proposed revisions would allow the Station Manager to assure that any reportable event is promptly investigated. These revisions would better reflect the Station Manager's role in the event of a reportable occurrence; he does not investigate the event himself, but assures that the event is investigated by appropriate personnel.

2.6 TS 6.2.3 - Reportable Events (Page 6.2-1)

TS 6.2.3 requires that the Commission be notified of a reportable event pursuant to TS 6.6.2. The proposed revisions would add that the notification be done pursuant to 10 CFR 50.73 also. The proposed revision would provide consistency between TS 6.6.2 and 6.2.3.

2.7 Summary

On the basis of our evaluation, we conclude that the proposed TS revisions are acceptable.

III. ENVIRONMENTAL CONSIDERATION

These amendments involve changes in recordkeeping, reporting or administrative procedures or requirements. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

IV. CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (50 FR 43026) on October 23, 1985, and consulted with the state of South Carolina. No public comments were received, and the state of South Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 19, 1987

Principal Contributors: P. Moore
H. Pastis

October 19, 1987

Docket Nos.: 50-269, 50-270
and 50-287

Mr. H. B. Tucker, Vice President
Nuclear Production Department
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

Subject: Denial of Amendment Request (TACS 60135/60136/60137)

Enclosed for your information is a copy of a "Notice of Denial of Amendments to Facility Operating Licenses and Opportunity for Hearing" related to a portion of your August 22, 1985, request for amendments to the operating licenses for Oconee Nuclear Station, Units 1, 2, and 3. The denied portion of the amendment would have revised Technical Specification Section 6.2.2 (Reportable Events) to include the Superintendent of Integrated Scheduling and Station Services Superintendent and also to make the section consistent with the Technical Specifications for the McGuire and Catawba Nuclear Stations.

The notice has been forwarded to the Office of the Federal Register for publication.

Sincerely,

151

Helen N. Pastis, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II

Enclosure: As stated

cc w/encl: See next page

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04/17/87

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04/17/87

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UNITED STATES NUCLEAR REGULATORY COMMISSIONDUKE POWER COMPANYDOCKET NOS. 50-269, 50-270, AND 50-287DENIAL OF AMENDMENTS TO FACILITY OPERATINGLICENSES AND OPPORTUNITY FOR HEARING

The U.S. Nuclear Regulatory Commission (the Commission) has denied a request by the licensee for amendments to Facility Operating License Nos. DPR-38, DPR-47, and DPR-55 issued to the Duke Power Company (the licensee) for operation of the Oconee Nuclear Station, Units 1, 2, and 3 (the facility), located in Oconee County, South Carolina.

The proposed amendments would have revised Technical Specification Section 6.2.2 to make it consistent with the Technical Specifications for Catawba and McGuire Nuclear Stations and include the Superintendent of Integrated Scheduling and Station Services Superintendent. Notice of consideration of issuance of the amendments was published in the FEDERAL REGISTER on October 23, 1985 (50 FR 43026). The licensee's application for the amendments was dated August 22, 1985, and supplemented February 11, 1986.

The request was found unacceptable since the revisions requested were not supported by adequate justification.

The licensee was notified of the Commission's denial of this request by letter dated October 19, 1987.

By November 23, 1987 the licensee may demand a hearing with respect to the denial described above and any person whose interest may be affected by this proceeding may file a written petition for leave to intervene.

A request for a hearing or petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Docketing and Service Branch, or may be delivered to the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., by the above date.

A copy of any petitions should also be sent to the Office of General Counsel-Bethesda, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, and to J. Michael McGarry, III, Esq., Bishop, Liberman, Cook, Purcell, and Reynolds, 1200 Seventeenth Street, N.W., Washington, D.C. 20036, attorney for the licensee.

For further details with respect to this action, see (1) the application for amendments dated August 22, 1985, as supplemented February 11, 1986, and (2) the Commission's letter to Duke Power Company dated October 19, 1987, which are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Oconee County Library, 501 West Southbroad Street, Walhalla, South Carolina 29691. A copy of item (2) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Reactor Projects I/II.

Dated at Bethesda, Maryland this 19th day of October 1987.

FOR THE NUCLEAR REGULATORY COMMISSION

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Helen N. Pastis, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II

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04/16/87

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PDII-3/DRP-I/II
DHood, Acting PD
10/16/87

Mr. H. B. Tucker
Duke Power Company

Oconee Nuclear Station
Units Nos. 1, 2 and 3

cc:

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Honorable James M. Phinney
County Supervisor of Oconee County
Walhalla, South Carolina 29621