

April 15, 1983

Units Nos. 50-269/270/287

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Mr. H. B. Tucker, Vice President
 Nuclear Production Department
 Duke Power Company
 P. O. Box 33189
 422 South Church Street
 Charlotte, North Carolina 28242

Dear Mr. Tucker:

The Commission has issued the enclosed Amendments Nos. 120, 120, and 117 to Facility Operating Licenses Nos. DPR-38, DPR-47 and DPR-55 for the Oconee Nuclear Station, Units Nos. 1, 2 and 3. These amendments consist of changes to the Oconee Station's common Technical Specifications (TSs) in response to your request dated February 23, 1983.

These amendments revise the TS withdrawal schedule for the reactor vessel surveillance capsules.

Our acceptance of this revised Oconee capsule withdrawal schedule from Crystal River, Unit No. 3, is based on the condition that the neutron irradiation received by Capsule OCI-C must exceed the anticipated end of life neutron fluence at the inside surface for the Oconee, Unit No. 1, and the Arkansas Nuclear One, Unit No. 1, reactor vessel beltlines. By copy of this letter, we are requesting the other affected B&W licensees, especially Florida Power Corporation, to make timely application(s) for corresponding TS changes.

The reporting and/or recordkeeping requirements of this letter affect fewer than ten respondents; therefore, OMB clearance is not required under P.L. 96-511.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by

E. L. Conner, Project Manager
 Operating Reactors Branch #4
 Division of Licensing

Enclosures:

1. Amendment No. 120 to DPR-38
2. Amendment No. 120 to DPR-47
3. Amendment No. 117 to DPR-55
4. Safety Evaluation
5. Notice

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 PDR ADDCK 05000269
 PDR

cc w/enclosures: See next page
 *See previous white for concurrences.

amend and form only

OFFICE	ORB#4:DL	ORB#4:DL	G-ORB#4:DL	AD:OR:DL	OELD		
SURNAME	RIngram *	Mooney:cf	JStolz*	GL:mas	T. TRANSON		
DATE	4/5/83	4/7/83	4/5/83	4/7/83	4/13/83		

Mr. H. B. Tucker, Vice President
 Nuclear Production Department
 Duke Power Company
 P. O. Box 33189
 422 South Church Street
 Charlotte, North Carolina 28242

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Dear Mr. Tucker:

The Commission has issued the enclosed Amendments Nos. and to Facility Operating Licenses Nos. DPR-38, DPR-47 and DPR-55 for the Oconee Nuclear Station, Units Nos. 1, 2 and 3. These amendments consist of changes to the Oconee Station's common Technical Specifications (TSs) in response to your request dated February 23, 1983.

These amendments revise the TS withdrawal schedule for the reactor vessel surveillance capsules.

Our acceptance of this revised Oconee capsule withdrawal schedule from Crystal River, Unit No. 3, is based on the condition that the neutron irradiation received by Capsule OCI-C must exceed the anticipated end of life neutron fluence at the inside surface for the Oconee, Unit No. 1, and the Arkansas Nuclear One, Unit No. 1, reactor vessel belt-lines. By copy of this letter, we are requesting the other affected B&W licensees, especially Florida Power Corporation, to make timely application(s) for corresponding TS changes.

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Sincerely,

E. L. Conner, Project Manager
 Operating Reactors Branch #4
 Division of Licensing

Enclosures:

1. Amendment No. to DPR-38
2. Amendment No. to DPR-47
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4. Safety Evaluation
5. Notice

cc w/enclosures: See next page

- audit and file only

OFFICE	ORB#4:DL	ORB#4:DL	C-ORB#4:DL	AD:OR:DL	OELD		
SURNAME	RIgram	MConner	JStall	GLainas	R. Rawson		
DATE	4/5/83	4/5/83	4/5/83	4/5/83	4/13/83		



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

April 15, 1983

DISTRIBUTION:
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RIngram

Docket No. 50-269/270/287

Docketing and Service Section
Office of the Secretary of the Commission

SUBJECT: OCONEE NUCLEAR STATION, UNITS NOS. 1, 2 AND 3.

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (12) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- Notice of Availability of Applicant's Environmental Report.
- Notice of Proposed Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) or Amendment(s).
- Other: Amendments Nos. 120, 120, and 117, respectively.
Referenced documents have been provided PDR.

Division of Licensing, ORB#4
Office of Nuclear Reactor Regulation

Enclosure:
As Stated

OFFICE	ORB#4:DL					
SURNAME	RIngram;cf					
DATE	4/15/83					

Duke Power Company

cc w/enclosure(s):

Mr. William L. Porter
Duke Power Company
P. O. Box 33189
422 South Church Street
Charlotte, North Carolina 28242

Office of Intergovernmental Relations
116 West Jones Street
Raleigh, North Carolina 27603

Honorable James M. Phinney
County Supervisor of Oconee County
Walhalla, South Carolina 29621

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission, Region II
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303

Regional Radiation Representative
EPA Region IV
345 Courtland Street, N.E.
Atlanta, Georgia 30308

William T. Orders
Senior Resident Inspector
U.S. Nuclear Regulatory Commission
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Seneca, South Carolina 29678

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Nuclear Power Generation Division
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DeBevoise & Liberman
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Washington, D. C. 20036

Mr. John M. Griffin, Vice President
Nuclear Operations
Arkansas Power & Light Company
P. O. Box 551
Little Rock, Arkansas 72203

Mr. Walter S. Wilgus
Vice President, Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear Licensing
& Fuel Management
P. O. Box 14042, M.A.C. H-2
St. Petersburg, Florida 33733



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.120
License No. DPR-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duke Power Company (the licensee) dated February 23, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-38 is hereby amended to read as follows:

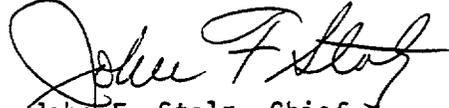
3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.120, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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P PDR

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 15, 1983



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-270

OCONEE NUCLEAR STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 120
License No. DPR-47

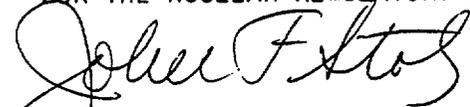
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duke Power Company (the licensee) dated February 23, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-47 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 120, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 15, 1983.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-287

OCONEE NUCLEAR STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 117
License No. DPR-55

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duke Power Company (the licensee) dated February 23, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-55 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 117, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 15, 1983

ATTACHMENTS TO LICENSE AMENDMENTS

AMENDMENT NO.120 TO DPR-38

AMENDMENT NO.120 TO DPR-47

AMENDMENT NO.117 TO DPR-55

DOCKETS NOS. 50-269, 50-270 AND 50-287

Replace the following page of the Appendix "A" Technical Specifications with the attached page. The revised page is identified by amendment numbers and contains vertical lines indicating the area of change.

Remove Page

4.2-3

Insert Page

4.2-3

Table 4.2-1

OCONEE NUCLEAR STATION CAPSULE ASSEMBLY
WITHDRAWAL SCHEDULE AT CRYSTAL RIVER UNIT NO. 3

<u>Capsule Designation</u>	<u>Insertion</u>	<u>Withdrawal</u>
OCI-A	End of 1st Cycle	End of 4th Cycle
OCI-B	End of 4th Cycle	End of 16th Cycle
OCI-C	End of 2nd Cycle	End of 11th Cycle
OCI-D	End of 9th Cycle	End of 18th Cycle
OCII-A	End of 1st Cycle	End of 2nd Cycle
OCII-B	End of 4th Cycle	End of 9th Cycle
OCII-D	End of 9th Cycle	End of 18th Cycle
OCII-E	End of 1st Cycle	End of 9th Cycle
OCII-F	End of 9th Cycle	End of 18th Cycle
OCIII-B	End of 1st Cycle	End of 2nd Cycle
OCIII-C	End of 4th Cycle	End of 10th Cycle
OCIII-D	End of 1st Cycle	End of 9th Cycle
OCIII-E	End of 5th Cycle	End of 18th Cycle
OCIII-F	End of 11th Cycle	End of 20th Cycle

NOTE: OCI - Capsules are from Unit No. 1
 OCII - Capsules are from Unit No. 2
 OCIII - Capsules are from Unit No. 3



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 120 TO FACILITY OPERATING LICENSE NO. DPR-38

AMENDMENT NO. 120 TO FACILITY OPERATING LICENSE NO. DPR-47

AMENDMENT NO. 117 TO FACILITY OPERATING LICENSE NO. DPR-55

DUKE POWER COMPANY

OCONEE NUCLEAR STATION, UNITS NOS. 1, 2 AND 3

DOCKETS NOS. 50-269, 50-270 AND 50-287

Introduction

By letter dated February 23, 1983, Duke Power Company (DPC or the licensee) proposed changes to the Oconee Station's common Technical Specifications (TSs) to revise the withdrawal schedule for the reactor vessel surveillance capsules. The licensee requests that Capsule OCI-A, which is being irradiated in the reactor vessel of Crystal River, Unit No. 3, be removed and be tested at the end of the fourth fuel cycle instead of the seventh fuel cycle. The licensee estimates that the capsule's accumulated neutron fluence at the end of the fourth fuel cycle would be 7.1×10^{18} n/cm² (E>1MeV) and at the end of the seventh fuel cycle would be 1.4×10^{19} n/cm².

Background

DPC is participating in the Babcock & Wilcox (B&W) Owners Group Integrated Reactor Vessel Material Surveillance Program (B&W Report BAW-1543). The weld material in Capsule OCI-A is identified as WF-112. This weld material is the limiting material in the reactor vessel of Arkansas Nuclear One, Unit 1 (ANO-1), whose owners are also participating in the integrated program. WF-112 weld material was welded using weld wire and flux which is similar to the limiting weld material in the Oconee reactor vessels. Hence, the results of the tests on irradiated WF-112 weld material will be useful for determining the material fracture toughness of the Oconee and ANO-1 reactor vessels.

Discussion and Evaluation

The 1/4 thickness and inside surface reactor vessel beltline material properties are the critical properties for determining the operating conditions and accident limitations for a reactor vessel. In accordance with Appendix G, 10 CFR 50, pressure-temperature limits for normal operation and anticipated upset conditions are evaluated based on the 1/4 thickness reactor vessel beltline material properties. In accordance with Commission Report SECY-82-465, the accident conditions of pressurized thermal shock are limited by the inside surface reactor vessel beltline

material properties. The maximum end of life (EOL) neutron fluence at the 1/4 thickness location for the Oconee and ANO-1 reactor vessel beltlines is estimated as 7.2×10^{18} n/cm² and 7×10^{18} n/cm², respectively. The maximum EOL neutron fluence at the inside surface for the Oconee and ANO-1 reactor vessel beltlines is estimated as 1.3×10^{19} n/cm² and 1.27×10^{19} n/cm², respectively. Thus, Capsule OCI-A at the end of the fourth fuel cycle will accumulate neutron fluence (7.1×10^{18} n/cm²), which is equivalent to the maximum EOL neutron fluence at the 1/4 thickness beltline locations for the Oconee and ANO-1 reactor vessels, but less than the maximum EOL neutron fluence at the inside beltline surface for the Oconee and ANO-1 reactor vessels.

The proposed withdrawal schedule for Capsule OCI-A will provide material properties which will be useful for determining the EOL pressure temperature limits for the Oconee and ANO-1 reactor vessels, but will not be irradiated for a sufficient amount of time to provide material properties for EOL accident limits for the Oconee and ANO-1 reactor vessels. Hence, we consider the proposed withdrawal schedule for Capsule OCI-A acceptable provided the neutron irradiation received by Capsule OCI-C, which contains the same weld material as Capsule OCI-A, exceeds the anticipated maximum EOL neutron fluence at the inside surface for the Oconee and ANO-1 reactor vessel beltlines.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of an accident previously evaluated, do not create the possibility of an accident of a type different from any evaluated previously, and do not involve a significant reduction in a margin of safety, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: April 15, 1983

The following NRC personnel have contributed to this Safety Evaluation:
B. Elliot, -E. Conner.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKETS NOS. 50-269, 50-270 AND 50-287DUKE POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 120, 120 and 117 to Facility Operating Licenses Nos. DPR-38, DPR-47 and DPR-55, respectively, issued to Duke Power Company, which revised the Technical Specifications (TSs) for operation of the Oconee Nuclear Station, Units Nos. 1, 2 and 3, located in Oconee County, South Carolina. The amendments are effective as of the date of issuance.

These amendments revise the TS withdrawal schedule for the reactor vessel surveillance capsules.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

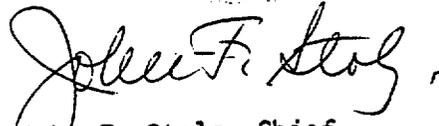
The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

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PDR

For further details with respect to this action, see (1) the application for amendments dated February 23, 1983, (2) Amendments Nos. 120 , 120 , and 117 to Licenses Nos. DPR-38, DPR-47 and DPR-55, respectively, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Oconee County Library, 501 West Southbroad Street, Walhalla, South Carolina 29691. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 15th day of April 1983.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing