

AUG 1 1973

Docket No. 50-269

Duke Power Company  
ATTN: Mr. A. C. Thies  
Senior Vice President  
Production and Transmission  
422 South Church Street  
P. O. Box 2178  
Charlotte, North Carolina 28201

Gentlemen:

On June 26, 1973, you requested and were granted by phone a change in Technical Specification 6.7 "Radiological Controls" of Technical Specification, Appendix A to Operating License No. DPR-38.

Prior to granting this request we determined that the change does not present significant hazards considerations not described or implicit in the Safety Analysis Report and there is reasonable assurance that the health and safety of the public will not be endangered.

Accordingly, pursuant to Section 20.501 of 10 CFR Part 20 and Section 50.59 of 10 CFR Part 50, Appendix A to Operating License No. DPR-38, Technical Specification 6.7 "Radiological Controls" is revised as the enclosure to this letter. This change will be known as Change No. 4 to the Technical Specifications, License DPR-38 and was effective on June 26, 1973.

Sincerely,

R. C. DeYoung, Assistant Director  
for Pressurized Water Reactors  
Directorate of Licensing

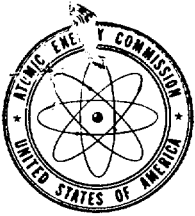
Enclosure:

Change No. 4 to DPR-38

bcc: J. R. Buchanan, ORNL  
T. B. Abernathy, DTIE

cc: William L. Porter, Esquire  
Duke Power Company  
422 South Church Street  
Charlotte, North Carolina 28201

OFFICE ▶	Charlotte, North Carolina 28201			RO	L:DD/TR	L:C/PWR-4	L:AD/PWRs
SURNAME ▶	IAPeltier	JGallo	Jeppler	JHendrie	ASchwencer	RCDeYoung	
DATE ▶	7/ 27 /73	7/ 27 /73	7/ 30 /73	7/ 30 /73	7/ 31 /73	7/ 31 /73	7/ 31 /73



UNITED STATES  
ATOMIC ENERGY COMMISSION  
WASHINGTON, D.C. 20545

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Gentlemen:

On June 26, 1973, you requested and were granted by phone a change in Technical Specification 6.7 "Radiological Controls" of Technical Specification, Appendix A to Operating License No. DPR-38.

Prior to granting this request we determined that the change does not present significant hazards considerations not described or implicit in the Safety Analysis Report and there is reasonable assurance that the health and safety of the public will not be endangered.

Accordingly, pursuant to Section 20.501 of 10 CFR Part 20 and Section 50.59 of 10 CFR Part 50, Appendix A to Operating License No. DPR-38, Technical Specification 6.7 "Radiological Controls" is revised as the enclosure to this letter. This change will be known as Change No. 4 to the Technical Specifications, License DPR-38 and was effective on June 26, 1973.

Sincerely,

A handwritten signature in cursive script, reading "R. C. DeYoung", is positioned above the typed name.

R. C. DeYoung, Assistant Director  
for Pressurized Water Reactors  
Directorate of Licensing

Enclosure:  
Change No. 4 to DPR-38

cc: William L. Porter, Esquire  
Duke Power Company  
422 South Church Street  
Charlotte, North Carolina 28201

- (6) Bioassays and/or whole body counts of individuals, and other surveys, as appropriate, to evaluate individual exposures and to assess protection actually provided.
  - (7) Records sufficient to permit periodic evaluation of the adequacy of the respiratory protective program.
  - (e) The licensee uses equipment approved by the U. S. Bureau of Mines under its appropriate Approval Schedules as set forth in Table 6.7-1 below. Equipment not approved under U. S. Bureau of Mines Approval Schedules may be used only if the licensee has evaluated the equipment and can demonstrate by testing, or on the basis of reliable test information, that the material and performance characteristics of the equipment are at least equal to those afforded by U. S. Bureau of Mines approved equipment of the same type, as specified in Table 6.7-1 below.
  - (f) Unless otherwise authorized by the Commission, the licensee does not assign protection factors in excess of those specified in Table 6.7-1 below in selecting and using respiratory protective equipment.
  - (g) These specifications with respect to the provisions of 20.103 shall be superseded by adoption of proposed changes to 10 CFR 20, Section 20.103, which would make this specification unnecessary.
- 6.7.2 Exposure of individuals to concentrations of radioactive noble gases may be controlled in accordance with the dose limits and requirements of Section 20.101, instead of 20.103 consistent with requirements of Specification 6.7.1a.2.a and footnote 2 referenced therein.

Date of Issuance: June 26 , 1973