September 26, 1988

Docket No.: 50-293

Mr. Ralph G. Bird
Senior Vice President - Nuclear
Boston Edison Company
Pilgrim Nuclear Power Station
RFD#1, Rocky Hill Road
Boston, Massachusetts 02199

SUBJECT: ISSUANCE OF AMENDMENT NO. 122 TO FACILITY OPERATING LICENSE NO. DPR-35 (TAC# 66526) PILGRIM NUCLEAR POWER STATION

Dear Mr. Bird:

The Commission has issued the enclosed Amendment No. 122 to Facility Operating License No. DPR-35 for the Pilgrim Nuclear Power Station. This amendment consists of changes to the Technical Specifications in response to your application dated August 6, 1988.

This amendment modifies the Technical Specifications, Section 6.0 Administrative Controls, to conform to the current nuclear organization at the Pilgrim Nuclear Power Station. The listing of the Onsite Review Committee (ORC) membership was removed to allow the Station Director to appoint in writing the ORC membership from appropriate plant disciplines. The references to specific ORC members also has been removed to eliminate the need to amend the Technical Specifications every time a member is changed.

A copy of our Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

/S/

Daniel G. McDonald, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects I/II

Enclosures:
1. Amendment No. 122 to DPR-35
2. Safety Evaluation

cc: w/enclosures:
See next page

*See previous concurrence

OFC	PDI-3	PDI-3	DIR/PDI-3	OGC			
NAME	*MRushbrook	*DMcDonald:mw	RWessman	CPW			
DATE	09/08/88	09/08/88	09/26/88	09/08/88			

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AMENDMENT NO. 122 TO FACILITY OPERATING LICENSE DPR-35-
PILGRIM NUCLEAR POWER STATION

DISTRIBUTION: w/enclosures:

Docket No. 50-293
NRC PDR
Local PDR
PDI-3 r/f
RWessman
MRushbrook
DMcDonald
OGC-15B-18
TBarnhart (4)
EJordan
BGrimes
DHagan
ACRS (10)
EButcher
Wanda Jones
OPA
LFMB
Tech. Review Branch

DF01
1/1

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

September 26, 1988

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Pilgrim Nuclear Power Station
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2. Safety Evaluation

cc: w/enclosures:
See next page

Mr. Ralph G. Bird
Boston Edison Company

Pilgrim Nuclear Power Station

cc:

Mr. K. L. Highfill
Station Director
Pilgrim Nuclear Power Station
RFD #1 Rocky Hill Road
Plymouth, Massachusetts 02360

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Secretary of Public Safety
Executive Office of Public Safety
One Ashburton Place
Boston, Massachusetts 02108

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BOSTON EDISON COMPANY

DOCKET NO. 50-293

PILGRIM NUCLEAR POWER STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 122
License No. DPR-35

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Boston Edison Company (the licensee) dated August 6, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 3.B of Facility Operating License No. DPR-35 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 122 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

R.H. Wessman

Richard H. Wessman, Director
Project Directorate I-3
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 26, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 122
FACILITY OPERATING LICENSE NO. DPR-35
DOCKET NO. 50-293

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove Pages	Insert Pages
iii	iii
208	208
208 a	208 a
Fig. 6.2-1 (pg. 209)	Fig. 6.2-1 (pg. 209)
Fig. 6.2-2 (pg. 210)	Fig. 6.2-2 (pg. 210)
212	212
213	213
216	216
217	217
226	226
227	227

		Page No.
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4.1	Sealed Source Contamination	206k
4.2	Surveillance Requirements	206k
4.3	Reports	2061
4.4	Records Retention	2061
5.0	MAJOR DESIGN FEATURES	206m
5.1	Site Features	206m
5.2	Reactor	206m
5.3	Reactor Vessel	206m
5.4	Containment	206m
5.5	Fuel Storage	207
5.6	Seismic Design	207
6.0	ADMINISTRATIVE CONTROLS	208
6.1	Responsibility	208
6.2	Organization	208
6.3	Facility Staff Qualifications	208a
6.4	Training	208a
6.5	Review and Audit	212
6.6	Reportable Event Action	216
6.7	Safety Limit Violation	217
6.8	Procedures	217
6.9	Reporting Requirements	218
6.10	Record Retention	224
6.11	Radiation Protection Program	226
6.12	(Deleted)	
6.13	High Radiation Area	226
6.14	Fire Protection Program	227

Operational Objectives		Surveillance	
7.0	RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM	8.0	229
7.1	Monitoring Program	8.1	229
7.2	Dose - Liquids	8.2	232
7.3	Dose - Noble Gases	8.3	233
7.4	Dose - Iodine-131, Iodine-133, Radioactive Material in Particulate Form, and Tritium	8.4	234
7.5	Total Dose	8.5	234

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

The Station Director shall be accountable for overall facility operation. In his absence, the Station Director shall designate, in writing, the individual to assume this responsibility.

6.2 ORGANIZATION

A. OFFSITE

The Company organization for station management and technical support shall be as shown on Figure 6.2-1.

B. FACILITY

The Facility organization shall be as shown on Figure 6.2-2 and:

1. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.

2. When the unit is in an operational mode other than cold shutdown or refueling, a person holding a Senior Reactor Operator License shall be present in the control room at all times. In addition to this Senior Operator, a Licensed Operator or Senior Operator shall be present at the controls when fuel is in the vessel.

3. At least two Licensed Operators shall be present in the control room during reactor startup, scheduled reactor shutdown and during recovery from reactor trips.

4. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.

5. ALL CORE ALTERATIONS performed while fuel is in the reactor vessel after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

6. A Fire Brigade of 5 members including a Fire Brigade Leader shall be maintained on site at all times. This excludes 3 members of the minimum shift crew necessary for safe shutdown and any personnel required for other essential functions during a fire emergency.

C. CHANGES TO THE ORGANIZATION

Changes may be made to the organization without prior license amendment provided that the NRC is notified in writing within 30 days and a revision to Section 6.2 is included in a subsequent license amendment request.

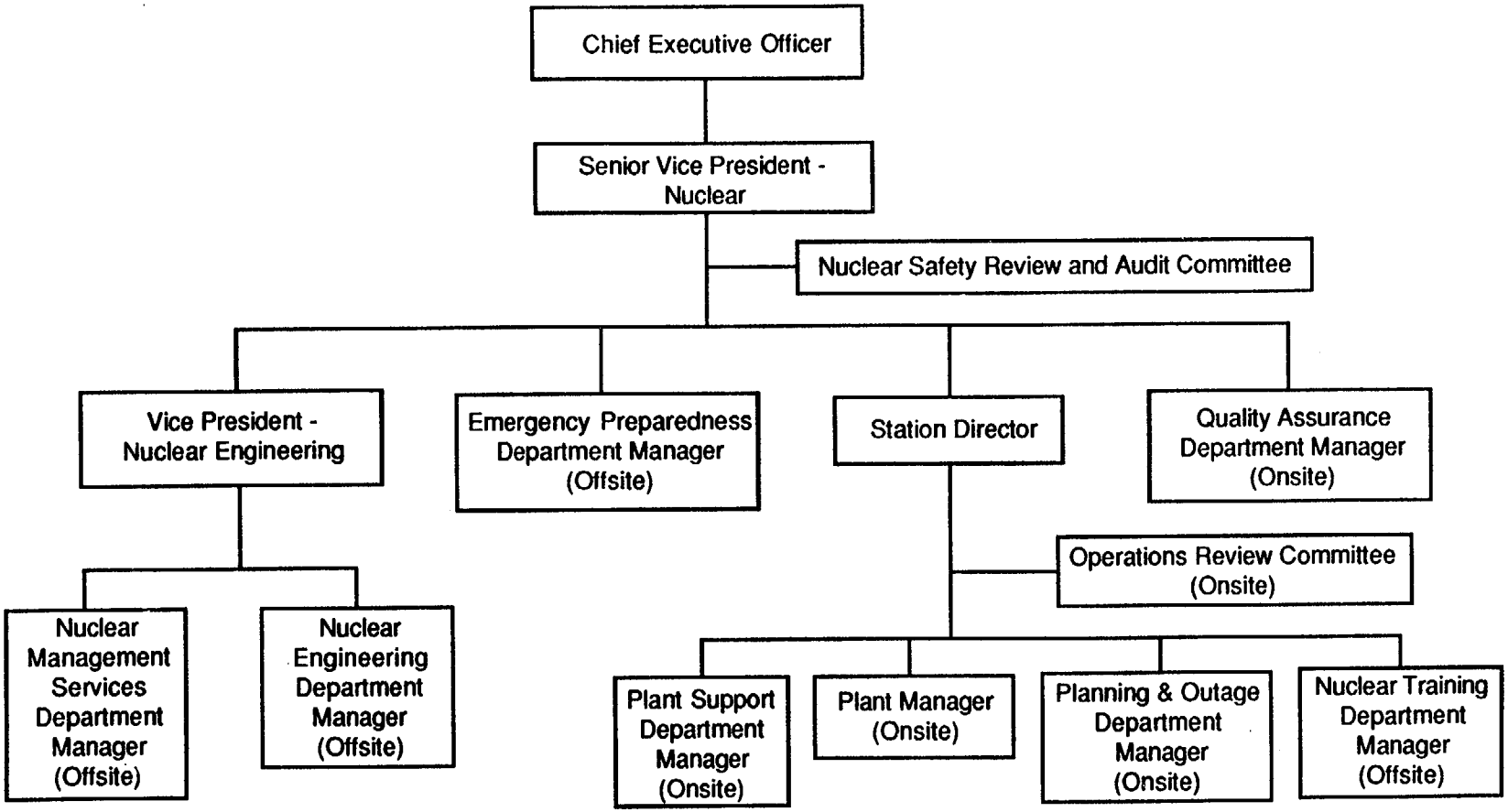
6.0 ADMINISTRATIVE CONTROLS

6.3 FACILITY STAFF QUALIFICATIONS

The qualifications with regard to educational and experience backgrounds of the facility staff at the time of appointment to the active position shall meet the requirements as described in the American National Standards Institute N18.1-1971, "Selection and Training of Personnel for Nuclear Power Plants." In addition, the individual performing the function of Radiation Protection Manager shall meet or exceed the qualifications of Regulatory Guide 1.8, September, 1975.

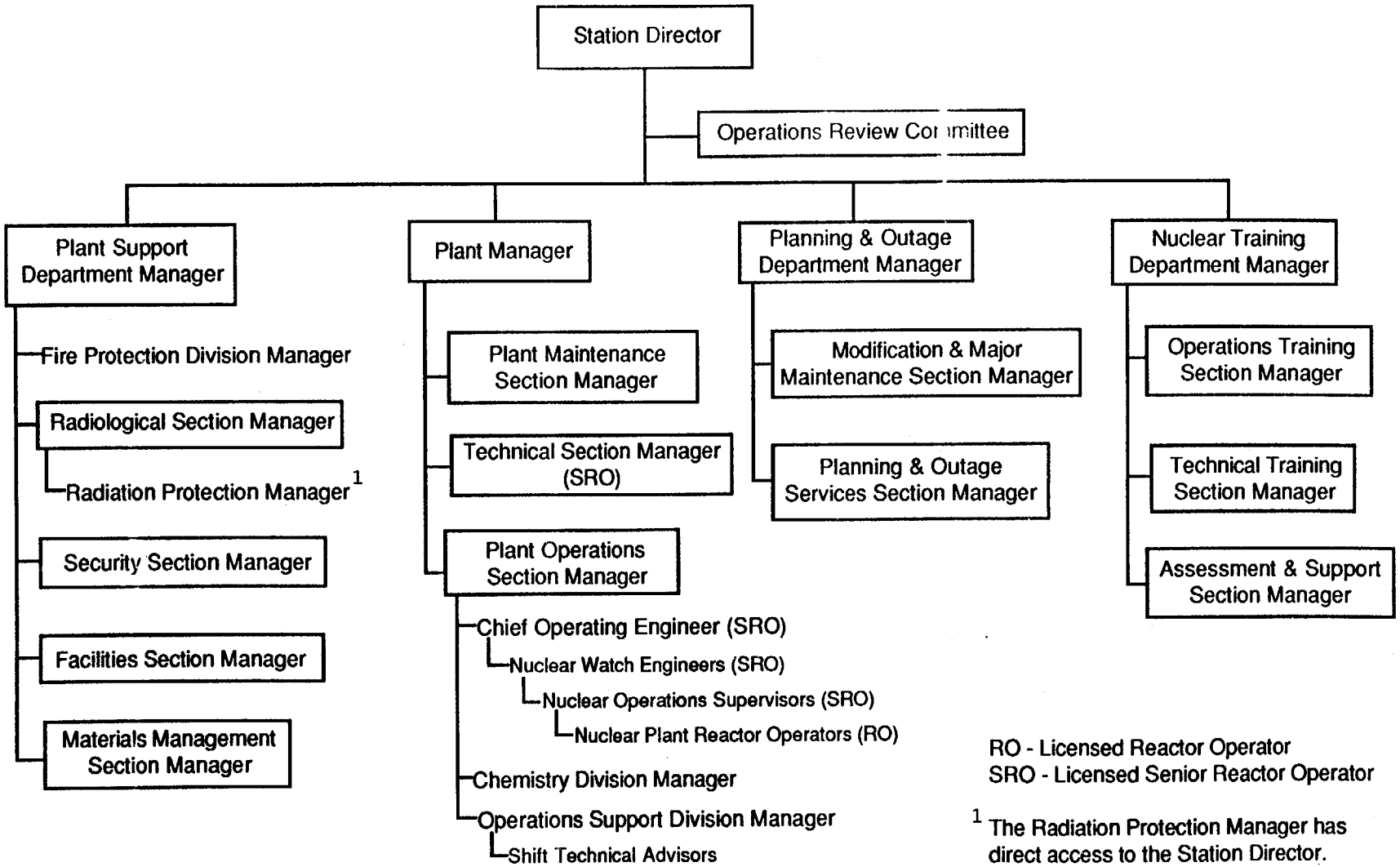
6.4 TRAINING

A retraining and replacement training program for the facility staff shall be maintained under the direction of the Nuclear Training Department Manager. The training programs for the licensed personnel shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix A of 10CFR Part 55. The training programs for the Fire Brigade shall meet or exceed the requirements of NFPA Standard No. 27-1975 "Private Fire Brigade". Fire Protection Training sessions will be held quarterly.



Boston Edison Company
Organization for Management and Technical Support of Pilgrim Station

Figure 6.2-1



Pilgrim Station Organization

Figure 6.2-2

6.5 REVIEW AND AUDIT

A. OPERATIONS REVIEW COMMITTEE (ORC)

1. FUNCTION

The ORC shall function to advise the Station Director on all matters related to safety.

2. COMPOSITION

The ORC shall be composed of the Technical Section Manager, who shall serve as ORC Chairman, and at least six members, who shall be appointed in writing by the Station Director from senior experienced onsite individuals, at the manager level or equivalent, representing each of the following disciplines: plant operations, plant maintenance, plant technical, reactor engineering, radiation protection, and chemistry.

3. ALTERNATES

Alternates shall be appointed in writing by the Station Director to serve on a temporary basis.

4. MEETING FREQUENCY

The ORC shall meet at least once per calendar month and as convened by the ORC Chairman.

5. QUORUM

A quorum of the ORC shall consist of the Chairman or designated alternate and a majority of members/designated alternates; however, no more than two alternates shall be allowed to meet quorum requirements.

6. RESPONSIBILITIES

The ORC shall be responsible for:

a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto that affect nuclear safety.

b. Review all proposed tests and experiments that affect nuclear safety.

c. Review of all proposed changes to the Technical Specifications.

d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.

6.5.A.6 RESPONSIBILITIES (Continued)

e. Review of facility operations to detect potential safety hazards.

f. Review of the Station Security Plan and implementing procedures and changes to the plan and procedures.

g. Review of the Emergency Plan and implementing procedures and changes to the plan and procedures.

h. Performance of special reviews and investigations and reports thereon as requested by the Nuclear Safety Review and Audit Committee (NSRAC) Chairman.

i. Investigation of all violations of the Technical Specifications and shall prepare and forward a report covering evaluation and recommendations to prevent recurrence to the Station Director, the NSRAC Chairman, and the Senior Vice President - Nuclear.

j. Review the Station Fire Protection Plan and implementing procedures and changes to the plan and procedures.

The ORC Chairman may appoint subcommittees composed of personnel who are not members of ORC to perform staff work necessary to the efficient functioning of ORC.

7. AUTHORITY

a. Recommend in writing to the Station Director the approval or disapproval of items considered under 6.5.A.6(a) through (d) above.

b. Render determinations in writing with regard to whether or not each item considered under 6.5.A.6(a) through (d) above constitutes an unreviewed safety question.

c. Provide immediate written notification to the Station Director, the Nuclear Safety Review and Audit Committee, and the Senior Vice President - Nuclear of disagreement between the ORC Members and the ORC Chairman. The Station Director shall have responsibility for resolution of such disagreements.

8. RECORDS

The ORC shall maintain written minutes of each meeting and copies shall be forwarded to the Station Director and the NSRAC Chairman.

6.5.B.8 AUDITS (Continued)

c. The results of all actions required by deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per six months.

d. The performance of all activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per two years.

e. The Emergency Plan and implementing procedures at least once per two years.

f. The Station Security Plan and implementing procedures at least once per two years.

g. Any other area of facility operation considered appropriate by the NSRAC or the Senior Vice President - Nuclear.

h. The Fire Protection Program and implementing procedures at least once per two years.

9. AUTHORITY

The NSRAC shall report to and advise the Senior Vice President - Nuclear on those areas of responsibility specified in Section 6.5.B.7 and 6.5.B.8.

10. RECORDS

Records of NSRAC activities shall be prepared, approved and distributed as indicated below:

a. Minutes of each NSRAC meeting shall be prepared, approved and forwarded to the Senior Vice President - Nuclear, NSRAC members, and others the Chairman may designate.

b. Reports of reviews encompassed by Section 6.5.B.7.e, f, g and h above, shall be prepared, approved and forwarded to the Senior Vice President - Nuclear, with a copy to the Station Director within 21 days following the completion of the review.

c. Audit reports encompassed by Section 6.5.B.8 above shall be forwarded to the Senior Vice President - Nuclear and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE EVENT ACTION

The following actions shall be taken for each reportable event:

A. The Commission shall be notified and/or a report submitted pursuant to the requirements of either 10 CFR 50.72 or 10 CFR 50.73.

B. Each Reportable Event Report submitted to the Commission shall be reviewed by the ORC and submitted to the NSRAC Chairman and the Station Director.

6.7 SAFETY LIMIT VIOLATION

The following actions shall be taken in the event a Safety Limit is violated:

A. The provisions of 10 CFR 50.36(c) (1) (i) shall be complied with immediately.

B. The Safety Limit Violation shall be reported to the Commission within 1 hour per 10CFR50.36(c)(6) and 50.72, and to the Station Director and the NSRAC Chairman immediately.

C. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the ORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.

D. The Safety Limit Violation Report shall be submitted to the Commission within 30 days in accordance with 10CFR50.36(c)(7) and 50.73 and to the NSRAC Chairman and the Station Director.

6.8 PROCEDURES

A. Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Sections 5.1 and 5.3 of ANSI N18.7 - 1972 and Appendix "A" of USNRC Regulatory Guide 1.33, except as provided in 6.8.B and 6.8.C below.

B. Each procedure of 6.8.A above, and changes thereto, shall be reviewed by the ORC and approved by the responsible department manager prior to implementation. These procedures shall be reviewed periodically as set forth in administrative procedures.

NOTE: ORC review and approval of procedures for vendors/contractors, who have a QA Program approved by Boston Edison Company, is not required for work performed at the vendor/contractor facility.

C. Temporary changes to procedures of 6.8.A above may be made provided:

1. The intent of the original procedure is not altered.

2. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's license on the unit affected.

3. The change is documented, subsequently reviewed by the ORC within 7 days of implementation, and approved by the responsible department manager.

D. Written procedures to implement the Fire Protection Program shall be established, implemented and maintained.

6.10.B RECORD RETENTION (Continued)

6. Records of gaseous and liquid radioactive material released to the environs.

7. Records of transient or operational cycles for those facility components designed for a limited number of transients or cycles.

8. Records of training and qualification for current members of the plant staff.

9. Records of in-service inspections performed pursuant to these Technical Specifications.

10. Records of Quality Assurance activities required by the QA Manual.

11. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.

12. Records of meetings of the ORC and the NSRAC.

13. Records for Environmental Qualification.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 (Deleted)

6.13 HIGH RADIATION AREA (OPTIONAL)

6.13.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit*. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.

* Health Physics personnel or personnel escorted by Health Physics personnel shall be exempt from the RWP issuance requirements during the performance of this assigned radiation protection duties, provided they comply with approved radiation protection procedures for entry into high radiation areas.

6.13.1 HIGH RADIATION AREA (Continued)

b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate level in the area has been established and personnel have been made knowledgeable of them.

c. An individual qualified in radiation protection procedures who is equipped with a radiation dose rate monitoring device. This individual shall be responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the Radiation Protection Manager in the Radiation Work Permit.

6.13.2 The requirements of 6.13.1, above, shall also apply to each high radiation area in which the intensity of radiation is greater than 1000 mrem/hr. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Nuclear Watch Engineer on duty and/or the Radiation Protection Manager.

6.14 FIRE PROTECTION PROGRAM

The following inspections and audits shall be performed:

1. An independent fire protection inspection and audit shall be performed annually utilizing either qualified off-site licensee personnel or an outside fire protection firm.

2. An inspection and audit by an outside qualified fire consultant shall be performed at intervals no greater than 3 years.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 122
TO FACILITY OPERATING LICENSE NO. DPR-35
BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

1.0 INTRODUCTION

The NRC staff noted in the Systematic Assessment of License Performance (SALP) Board Report No. 50-293/87-99 for the Pilgrim Nuclear Power Station (PNPS) which covered the period of February 1, 1987 through May 15, 1988, that an organizational transition had taken place. The report also noted that several temporary changes, including numerous changes in personnel, had been made to strengthen planning, control and performance at the PNPS. Many of these temporary changes were incorporated into a permanent reorganization in February 1988. The licensee continued to refine the new organization and control process through July 1988. The licensee notified the NRC of the reorganization by letter dated January 7, 1988 and subsequently requested an amendment to reflect the new organization in a letter dated August 6, 1988.

The notification and request were in accordance with the PNPS Technical Specifications (TS), Section 6.2.C, "Changes to the Organization," which allows organizational changes to be implemented without prior NRC approval provided notification is made and a subsequent license amendment request is submitted for NRC review and approval.

2.0 DISCUSSION

The requested changes include TS Figures 6.2-1 and 6.2-2 which are revised to conform with the current nuclear organization at PNPS. These changes include the replacement of the Vice President - Nuclear Operations and Nuclear Operations Manager positions with a Station Director who holds overall responsibility for the safe operation of the facility, as stated in TS 6.1. The Operations Review Committee (ORC) reports to the new Station Director and the Quality Assurance and Emergency Preparedness Departments report directly to the Senior Vice President - Nuclear. In addition, the responsibilities of the departments reporting to the Station Director and the Vice President - Nuclear Engineering are restructured.

The ORC composition in Technical Specification 6.5.A.2 is revised to allow the Station Director to appoint in writing the ORC membership from appropriate plant disciplines, instead of listing the membership in Technical Specifications. The Pilgrim Station Organization in Technical Specification Figure 6.2-2 is revised accordingly to remove references to specific ORC members.

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The management titles throughout Technical Specification 6.0 are revised to adopt those currently in use in the nuclear organization.

The reporting requirements given in Specifications 6.5.A.6.i and 6.5.A.7.c for Technical Specification violations and ORC disagreements, respectively, are changed from the obsolete positions, Nuclear Operations Manager and Vice President - Nuclear Operations, to the Station Director and the Senior Vice President - Nuclear.

The authority for the approval of procedures in Specification 6.8 is changed from the obsolete position, Nuclear Operations Manager, to the "responsible department manager."

Several editorial changes are made to correct the Table of Contents page numbers, correct punctuation, add appropriate page headers, spell out the acronym "NSRAC," renumber the items listed under Specification 6.10.B, and correct the spelling of "preset" in Specification 6.13.1.b.

3.0 EVALUATION

An Integrated Assessment Team Inspection (IATI) was conducted at the PNPS during the period of August 8 through 24, 1988 and is documented in Inspection Report 50-293/88-21 dated September 7, 1988. The purpose of the inspection was to assess the facility's readiness for restart including the current organizational structure and management.

The IATI report concluded that the organizational structure provides for an appropriate distribution (span) of responsibilities and accountabilities for the activities being performed by the functional units within the organization. The depth (level) of managers in the functional areas should contribute to stability in the organization by providing for the development of technical and managerial skills within the functional areas and providing a framework for career growth.

The report also concluded that the redistribution of functional responsibilities and depth in management throughout the organization provides the framework necessary to enhance stability and support safe and reliable operation of the PNPS. This is reenforced by the evidence of management's effectiveness in creating a much improved nuclear safety ethic and bringing about improvements in the functional areas described in the subsequent sections of the report.

The resumes and position descriptions of key managers and selected personnel throughout the organization were audited during the inspection. The educational and experience backgrounds of the licensee's personnel were compared to the requirements of the positions held, as delineated in ANSI N18.1-1971, with a focus on the management experience of key personnel. No deficiencies were identified relating to the qualification requirements of the ANSI standard. More significantly, it should be noted that there has been an addition of talented management personnel with extensive and successful management experience.

The corporate policy relating to the Nuclear Organization contained in the Mission, Organization and Policy manual includes, among other goals, striving to achieve raising standards of performance, dedication to protecting the environment and public, and rigorous adherence to procedures. The IATI members, through observations, and interviews, noted a positive change in the attitude toward nuclear safety throughout the PNPS organization. This change was evident in the changing cultural attributes resulting in improved performance of safety-related activities. These improvements were acknowledged in the most recent SALP report previously discussed and have progressed further as discussed in the functional areas of the IATI report. The observations support management's effectiveness in communicating corporate goals and management's oversight in assuring that the goals are being supported and pursued.

The licensee has a variety of procedures to provide policy, control and coordination of the activities and actions of the organization. The procedures are, for the most part, current. They adequately identify corporate policy, organization, interfaces, functional requirements, responsibilities, accountabilities and qualifications necessary for the control of activities and coordination of actions within the organization.

The staff has determined that the proposed changes in the PNPS organization are acceptable; the PNPS organization is adequately staffed with qualified personnel; mechanisms are in place to enhance stability; and controls and programs are in place to support the organization. This conclusion is based on the details discussed above and supported by additional details in the IATI report. The revision to Section 6.5.A.2 to allow the Station Director to appoint in writing the ORC membership from the listed plant disciplines instead of listing the membership does not change the types of members but does allow changes of specific members. This change will reduce the number of administrative TS changes which would have been required while maintaining the representation by the appropriate disciplines, and therefore, is acceptable.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to recordkeeping, reporting or administrative procedures or requirements. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

6.0 ACKNOWLEDGEMENT

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Dated: September 26, 1988