

August 6, 1986

Docket No. 50-293

Mr. James M. Lydon
Chief Operating Officer
Boston Edison Company
800 Boylston Street
Boston, Massachusetts 02199

Dear Mr. Lydon:

SUBJECT: CALIBRATION FREQUENCY CHANGE FOR CERTAIN ROSEMOUNT TRANSMITTERS
(TAC 57510)

Re: Pilgrim Nuclear Power Station

The Commission has issued the enclosed Amendment No. 97 to Facility Operating License No. DPR-35 for the Pilgrim Nuclear Power Station. This amendment is in response to your application dated April 12, 1985.

The amendment revises the Technical Specifications by changing the calibration frequency for the reactor water level, reactor pressure, and drywell pressure surveillance instrument channels from once every 6 months to once each refueling outage.

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notices.

Sincerely,

ORIGINAL SIGNED BY
John A. Zwolinski, Director
BWR Project Directorate #1
Division of BWR Licensing

Enclosures:

1. Amendment No. 97 to License No. DPR-35
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. James M. Lydon
Boston Edison Company

Pilgrim Nuclear Power Station

cc:

Mr. Alfred E. Pedersen, Station Manager
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BOSTON EDISON COMPANY

DOCKET NO. 50-293

PILGRIM NUCLEAR POWER STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 97
License No. DPR-35

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Boston Edison Company (the licensee) dated April 12, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-35 is hereby amended to read as follows:

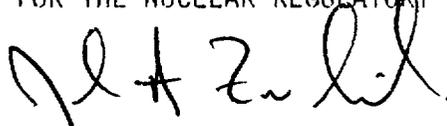
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B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 97, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective 30 days after the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John A. Zwolinski, Director
BWR Project Directorate #1
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 6, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 97

FACILITY OPERATING LICENSE NO. DPR-35

DOCKET NO. 50-293

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

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INSERT

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TABLE 4.2.F
MINIMUM TEST AND CALIBRATION FREQUENCY FOR SURVEILLANCE INSTRUMENTATION

	<u>Instrument Channel</u>	<u>Calibration Frequency</u>	<u>Instrument Check</u>
1)	Reactor Water Level	Each Refueling Outage	Each Shift
2)	Reactor Pressure	Each Refueling Outage	Each Shift
3)	Drywell Pressure	Each Refueling Outage	Each Shift
4)	Drywell Temperature	Once/6 Months	Each Shift
5)	Suppression Chamber Temperature	Once/6 Months	Each Shift
6)	Suppression Chamber Water Level	Once/6 Months	Each Shift
7)	Control Rod Position	NA	Each Shift
8)	Neutron Monitoring	(2)	Each Shift
9)	Drywell/Torus Differential Pressure	Once/6 Months	Each Shift
10)	{ Drywell Pressure { Torus Pressure	{ Once/6 Months } { Once/6 Months }	Each Shift
11)	Safety/Relief Valve Position Indicator (Primary/Secondary)	Each refueling outage	Once each day
12)	Safety Valve Position Indicator (Primary/ Secondary)	Each refueling outage	Once each day



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 97 TO FACILITY OPERATING LICENSE NO. DPR-35
BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

1.0 INTRODUCTION

By letter dated April 12, 1985, the Boston Edison Company (the licensee) requested a revision of Table 4.2.F of the Pilgrim Station Technical Specifications. The revision would change the calibration frequency for the reactor water level, reactor pressure, and drywell pressure surveillance instrument channels from once every 6 months to once each refueling outage.

The change in calibration frequency was proposed to reduce the number of calibrations performed and to eliminate the on-line valving of these instruments that is often required for 6-month calibrations. Thus, the risk of inadvertent actuation of safety systems would also be reduced.

2.0 EVALUATION

The licensee considers the proposed decrease in calibration frequency to be warranted because of the improved accuracy and stability provided by Rosemount analog transmitters which were installed for these instrument channels during the last refueling outage. The new transmitters replaced mechanically-operated transmitters which had become obsolete. Mechanical force transfer and problems associated with shock and vibration are eliminated in the Rosemount transmitters by direct electronic sensing with a sealed capacitance sensing element.

Similar analog transmitters were included in the Analog Transmitter/Trip Unit System (ATTUS) which was presented by the General Electric Company to the NRC staff for licensing review in topical report NEDO-21617-A. ATTUS provides the input for plant process parameters to the system logics for the reactor protection system, the primary containment isolation system, and the core standby cooling system. The staff reviewed the ATTUS and found it acceptable, as stated in its letter to General Electric dated June 27, 1978. That finding included Technical Specification changes proposed in NEDO-21617-A which would revise the test and calibration frequencies by requiring analog transmitter calibrations to be performed once per cycle during refueling outages. Together with such a change, a daily instrument check is required to provide assurance that the instrumentation is operable.

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The proposed amendment would apply the ATTUS calibration frequency (i.e., each refueling outage) to the reactor water level, reactor pressure, and drywell pressure surveillance instrumentation. An instrument check during each shift is already required for these channels. Thus, we find that the proposed Technical Specification changes, in conjunction with the recent installation of analog transmitters for these channels, are consistent with our requirements for ATTUS. The Technical Specification changes are, therefore, acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves only a change in a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor: Paul H. Leech

Dated: August 6, 1986