

November 2, 1995

Mr. Ted C. Feigenbaum  
Senior Vice President  
and Chief Nuclear Officer  
North Atlantic Energy Service Corporation  
Post Office Box 300  
Seabrook, NH 03874

SUBJECT: AMENDMENT NO. 43 TO FACILITY OPERATING LICENSE NPF-86: MAIN STEAM SAFETY VALVE SETPOINTS AND MAXIMUM ALLOWABLE POWER RANGE HIGH FLUX SETPOINT WITH INOPERABLE MAIN STEAM SAFETY VALVES - LICENSE AMENDMENT REQUEST 95-04 (TAC NO. M93410)

Dear Mr. Feigenbaum:

The Commission has issued the enclosed Amendment No. 43 to Facility Operating License No. NPF-86 for the Seabrook Station, Unit No. 1, in response to your application dated September 5, 1995.

The amendment modifies the Appendix A Technical Specifications (TSs) for the Turbine Cycle Safety Valves. Specifically, the amendment changes Seabrook Station Appendix A Technical Specification Table 3.7-1 to reduce the Maximum Allowable Power Range Neutron Flux - High Setpoints with Inoperable Main Steam Safety Valves (MSSVs) and Table 3.7-2 to reduce the opening setpoints of the MSSVs.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Albert W. De Agazio, Sr. Project Manager  
Project Directorate I-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-443  
Serial No. SEA-95-024

Enclosures: 1. Amendment No. 43 to NPF-86  
2. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION:

Docket File	PMcKee	OGC	JRogge, RGI
PUBLIC	ADe Agazio	ACRS	CLiang
PDI-3 Plant	SNorris	GHill (2)	
SVarga	RJones	CGrimes	

**NRC FILE CENTER COPY**

DOCUMENT NAME: G:\DEAGAZIO\93410AMD

*\* see previous concurrence*

OFFICE	LA:PDI-3	PM:PDI-3	BC:SRXB	*	D:PDI-3	OGC
NAME	SNorris	ADe Agazio:bf	RJones		PMcKee	
DATE	10/17/95	10/17/95	10/17/95		10/17/95	10/27/95

9511080336 951102  
PDR ADOCK 05000443  
P PDR

FICIAL RECORD COPY

*DF011*

November 2, 1995

Mr. Ted C. Feigenbaum  
Senior Vice President  
and Chief Nuclear Officer  
North Atlantic Energy Service Corporation  
Post Office Box 300  
Seabrook, NH 03874

SUBJECT: AMENDMENT NO. 43 TO FACILITY OPERATING LICENSE NPF-86: MAIN STEAM SAFETY VALVE SETPOINTS AND MAXIMUM ALLOWABLE POWER RANGE HIGH FLUX SETPOINT WITH INOPERABLE MAIN STEAM SAFETY VALVES - LICENSE AMENDMENT REQUEST 95-04 (TAC NO. M93410)

Dear Mr. Feigenbaum:

The Commission has issued the enclosed Amendment No. 43 to Facility Operating License No. NPF-86 for the Seabrook Station, Unit No. 1, in response to your application dated September 5, 1995.

The amendment modifies the Appendix A Technical Specifications (TSs) for the Turbine Cycle Safety Valves. Specifically, the amendment changes Seabrook Station Appendix A Technical Specification Table 3.7-1 to reduce the Maximum Allowable Power Range Neutron Flux - High Setpoints with Inoperable Main Steam Safety Valves (MSSVs) and Table 3.7-2 to reduce the opening setpoints of the MSSVs.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Albert W. De Agazio, Sr. Project Manager  
Project Directorate I-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-443  
Serial No. SEA-95-024

Enclosures: 1. Amendment No. 43 to NPF-86  
2. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION:

Docket File	PMcKee	OGC	JRogge, RGI
PUBLIC	ADe Agazio	ACRS	CLiang
PDI-3 Plant	SNorris	GHill (2)	
SVarga	RJones	CGrimes	

DOCUMENT NAME: G:\DEAGAZIO\93410AMD

*\* see previous concurrence*

OFFICE	LA:PDI-3	PM:PDI-3	BC:SRXB	*	D:PDI-3	OGC
NAME	SNorris	ADe Agazio:bf	RJones		PMcKee	
DATE	10/17/95	10/17/95	10/17/95		10/17/95	10/22/95

OFFICIAL RECORD COPY

T. Feigenbaum  
North Atlantic Energy Service Corporation

Seabrook Station, Unit No. 1

cc:

Lillian M. Cuoco, Esq.  
Senior Nuclear Counsel  
Northeast Utilities Service Company  
P.O. Box 270  
Hartford, CT 06037

Office of the Attorney General  
One Ashburton Place  
20th Floor  
Boston, MA 02108

Mr. Peter Brann  
Assistant Attorney General  
State House, Station #6  
Augusta, ME 04333

Board of Selectmen  
Town of Amesbury  
Town Hall  
Amesbury, MA 01913

Resident Inspector  
U.S. Nuclear Regulatory Commission  
Seabrook Nuclear Power Station  
P.O. Box 1149  
Seabrook, NH 03874

Mr. Jack Dolan  
Federal Emergency Management Agency  
Region I  
J.W. McCormack P.O. &  
Courthouse Building, Room 442  
Boston, MA 02109

Jane Spector  
Federal Energy Regulatory Commission  
825 North Capital Street, N.E.  
Room 8105  
Washington, DC 20426

Mr. David Rodham, Director  
ATTN: James Muckerheide  
Massachusetts Civil Defense Agency  
400 Worcester Road  
P.O. Box 1496  
Framingham, MA 01701-0317

Mr. T. L. Harpster  
North Atlantic Energy Service  
Corporation  
P.O. Box 300  
Seabrook, NH 03874

Jeffrey Howard, Attorney General  
G. Dana Bisbee, Deputy Attorney  
General  
33 Capitol Street  
Concord, NH 03301

Town of Exeter  
10 Front Street  
Exeter, NH 03823

Mr. R. M. Kacich, Director  
Nuclear Planning, Licensing & Budgeting  
Northeast Utilities Service Company  
P.O. Box 128  
Waterford, CT 06385

Mr. George L. Iverson, Director  
New Hampshire Office of Emergency  
Management  
State Office Park South  
107 Pleasant Street  
Concord, NH 03301

Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, PA 19406



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

November 2, 1995

Mr. Ted C. Feigenbaum  
Senior Vice President  
and Chief Nuclear Officer  
North Atlantic Energy Service Corporation  
Post Office Box 300  
Seabrook, NH 03874

SUBJECT: AMENDMENT NO. 43 TO FACILITY OPERATING LICENSE NPF-86: MAIN STEAM SAFETY VALVE SETPOINTS AND MAXIMUM ALLOWABLE POWER RANGE HIGH FLUX SETPOINT WITH INOPERABLE MAIN STEAM SAFETY VALVES - LICENSE AMENDMENT REQUEST 95-04 (TAC NO. M93410)

Dear Mr. Feigenbaum:

The Commission has issued the enclosed Amendment No. 43 to Facility Operating License No. NPF-86 for the Seabrook Station, Unit No. 1, in response to your application dated September 5, 1995.

The amendment modifies the Appendix A Technical Specifications (TSs) for the Turbine Cycle Safety Valves. Specifically, the amendment changes Seabrook Station Appendix A Technical Specification Table 3.7-1 to reduce the Maximum Allowable Power Range Neutron Flux - High Setpoints with Inoperable Main Steam Safety Valves (MSSVs) and Table 3.7-2 to reduce the opening setpoints of the MSSVs.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Albert W. De Agazio".

Albert W. De Agazio, Sr. Project Manager  
Project Directorate I-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-443  
Serial No. SEA-95-024

Enclosures: 1. Amendment No. 43 to NPF-86  
2. Safety Evaluation

cc w/encls: See next page



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

NORTH ATLANTIC ENERGY SERVICE CORPORATION, ET AL\*

DOCKET NO. 50-443

SEABROOK STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 43  
License No. NPF-86

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by North Atlantic Energy Service Corporation, et al. (the licensee), dated September 5, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

---

\*North Atlantic Energy Service Company (NAESCO) is authorized to act as agent for the: North Atlantic Energy Corporation, Canal Electric Company, The Connecticut Light and Power Company, Great Bay Power Corporation, Hudson Light and Power Department, Massachusetts Municipal Wholesale Electric Company, Montaup Electric Company, New England Power Company, New Hampshire Electric Cooperative, Inc., Taunton Municipal Light Plant, and The United Illuminating Company, and has exclusive responsibility and control over the physical construction, operation, and maintenance of the facility.

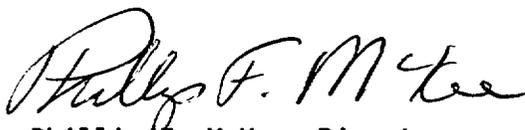
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-86 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 43, and the Environmental Protection Plan contained in Appendix B are incorporated into Facility License No. NPF-86. NAESCO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, to be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Phillip F. McKee, Director  
Project Directorate I-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: November 2, 1995

### 3/4.7 PLANT SYSTEMS

#### 3/4.7.1 TURBINE CYCLE

##### SAFETY VALVES

##### LIMITING CONDITION FOR OPERATION

---

3.7.1.1 All main steam line Code safety valves associated with each steam generator shall be OPERABLE with lift settings as specified in Table 3.7-2.

APPLICABILITY: MODES 1, 2, and 3\*.

##### ACTION:

With four reactor coolant loops and associated steam generators in operation and with one or more main steam line Code safety valves inoperable, operation in MODES 1, 2, and 3 may proceed, provided that within 4 hours either the inoperable valve is restored to OPERABLE status or the Power Range Neutron Flux High Trip Setpoint is reduced per Table 3.7-1; otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours .

##### SURVEILLANCE REQUIREMENTS

---

4.7.1.1 No additional requirements other than those required by Specification 4.0.5.

---

# Entry into this MODE is permitted for up to 24 hours to perform post modification or post-maintenance testing to verify OPERABILITY of components. ACTION requirements shall not apply until OPERABILITY has been verified.

TABLE 3.7-1

MAXIMUM ALLOWABLE POWER RANGE NEUTRON FLUX HIGH SETPOINT WITH  
INOPERABLE STEAM LINE SAFETY VALVES DURING FOUR-LOOP OPERATION

<u>MAXIMUM NUMBER OF INOPERABLE SAFETY VALVES ON ANY OPERATING STEAM GENERATOR</u>	<u>MAXIMUM ALLOWABLE POWER RANGE NEUTRON FLUX HIGH SETPOINT (PERCENT OF RATED THERMAL POWER)</u>
1	66
2	47
3	28

TABLE 3.7-2

STEAM LINE SAFETY VALVES PER LOOP

VALVE NUMBER

<u>Loop 1</u>	<u>Loop 2</u>	<u>Loop 3</u>	<u>Loop 4</u>	<u>LIFT SETTING* (<math>\pm 3\%</math>)**</u>	<u>ORIFICE SIZE</u>
V6	V22	V36	V50	1185 psig	16.0 sq. in.
V7	V23	V37	V51	1195 psig	16.0 sq. in.
V8	V24	V38	V52	1205 psig	16.0 sq. in.
V9	V25	V39	V53	1215 psig	16.0 sq. in.
V10	V26	V40	V54	1225 psig	16.0 sq. in.

\*The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

\*\*Within  $\pm 1\%$  following main steam line Code safety valve testing.

### 3/4.7 PLANT SYSTEMS

#### BASES

---

#### 3/4.7.1 TURBINE CYCLE

##### 3/4.7.1.1 SAFETY VALVES

The OPERABILITY of the main steam line Code safety valves ensures that the Secondary System pressure will be limited to within 110% (1320 psia) of its design pressure of 1200 psia during the most severe anticipated system operational transient. The maximum relieving capacity is associated with a Turbine trip from 100% RATED THERMAL POWER coincident with an assumed loss of condenser heat sink (i.e., no steam bypass to the condenser).

The specified valve lift settings and relieving capacities are in accordance with the requirements of Section III of the ASME Boiler and Pressure Code, (1974 Edition, including the Summer 1975 Addenda). The total relieving capacity for all valves on all of the steam lines is  $1.816 \times 10^7$  lbs/hr which is 120% of the total secondary steam flow of  $1.514 \times 10^7$  lbs/hr at 100% RATED THERMAL POWER. A minimum of two OPERABLE safety valves per steam generator ensures that sufficient relieving capacity is available for the allowable THERMAL POWER restriction in Table 3.7-1.

STARTUP and/or POWER OPERATION is allowable with safety valves inoperable within the limitations of the ACTION requirements on the basis of the reduction in Secondary Coolant System steam flow and THERMAL POWER required by the reduced Reactor trip settings of the Power Range Neutron Flux channels. The Reactor Trip Setpoint reductions are derived on the following bases:

For four loop operations:

$$Hi \phi = (100/Q_{rated}) \times \left[ \frac{(W_s \times h_{fg} \times N)}{K} - Q_{rcp} \right]$$

where:

- |             |   |   |
|-------------|---|---|
| Hi $\phi$   | = | Safety Analysis power range high neutron flux setpoint, percent of RATED THERMAL POWER  |
| $Q_{rated}$ | = | RATED THERMAL POWER, Mwt  |
| $Q_{rcp}$   | = | Reactor coolant pump heat, Mwt  |
| K           | = | Conversion factor, $3.412 \times 10^6$ (Btu/hr)/Mwt   |
| $h_{fg}$    | = | heat of vaporization for steam at 110% of the Secondary System design pressure, Btu/lbm   |
| N           | = | Number of loops in plant  |
| $W_s$       | = | Minimum total steam flow rate, lbm/hr, of the operable MSSVs on any one steam generator at the MSSV inlet pressure which assures all Secondary System pressures are no greater than 110% of design. |

## PLANT SYSTEMS

### BASES

#### 3/4.7.1 TURBINE CYCLE (Continued)

##### 3/4.7.1.1 SAFETY VALVES

The steam flow rate through each operable MSSV is calculated with consideration of 1) opening setpoint with allowance for as-found setpoint tolerance, 2) accumulation, and 3) inlet pressure. For example, if the maximum number of inoperable MSSVs on any one steam generator is one, then  $W_s$  is a summation of the calculated flow through each MSSV at the appropriate valve inlet pressure, excluding the MSSV with the highest flow. If the maximum number of inoperable MSSVs per steam generator is three, then  $W_s$  is a summation of the calculated flow through each MSSV at the appropriate valve inlet pressure, excluding the three MSSVs with the highest flows. The following plant specific safety valve flow rates were used:

SG Safety Valve Number (Bank No.)	Main Steam System	
	Set Pressure (psia)	Flow (lbm/hr per loop)
1	1200	893,160
2	1210	900,607
3	1220	908,055
4	1230	915,502
5	1240	922,950

The Safety Analysis limit values of the power range high neutron flux setpoints calculated from this algorithm are adjusted lower for use in Technical Specification 3.7.1.1. to account for instrument and channel uncertainties.

## PLANT SYSTEMS

### BASES

---

#### 3/4.7.1 TURBINE CYCLE (Continued)

##### 3/4.7.1.2 AUXILIARY FEEDWATER SYSTEM

The OPERABILITY of the Auxiliary Feedwater System ensures that the Reactor Coolant System can be cooled down to less than 350°F from normal operating conditions in the event of a total loss-of-offsite power.

The electric motor-driven emergency feedwater pump is capable of delivering a total feedwater flow of 650 gpm at a pressure of 1221 psig to the entrance of the steam generators. The steam-driven emergency feedwater pump is capable of delivering a total feed water flow of 650 gpm at a pressure of 1221 psig to the entrance of the steam generators. The startup feedwater pump serves as the third auxiliary feedwater pump and can be manually aligned to be powered from an emergency bus (Bus 5). The startup feedwater pump is capable of taking suction on the dedicated emergency feedwater volume of water in the condensate storage tank and delivering a total feedwater flow of in excess of 650 gpm at a pressure of 1221 psig to the entrance of the steam generator via either the main feedwater header or with manual alignment to the emergency feed water flow path. This capacity is sufficient to ensure that adequate feedwater flow is available to remove decay heat and reduce the Reactor Coolant System temperature to less than 350°F when the Residual Heat Removal System may be placed into operation.

##### 3/4.7.1.3 CONDENSATE STORAGE TANK

The OPERABILITY of the condensate storage tank with the minimum water volume ensures that sufficient water is available to cool the RCS to a temperature of 350°F. The OPERABILITY of the concrete enclosure ensures this availability of water following rupture of the condensate storage tank by a tornado generated missile. The contained water volume limit includes an allowance for water not usable because of tank discharge line location or other physical characteristics.

##### 3/4.7.1.4 SPECIFIC ACTIVITY

The limitations on Secondary Coolant System specific activity ensure that the resultant offsite radiation dose will be limited to a small fraction of 10 CFR Part 100 dose guideline values in the event of a steam line rupture. This dose also includes the effects of a coincident 1 gpm reactor-to-secondary tube leak in the steam generator of the affected steam line. These values are consistent with the assumptions used in the safety analyses.

##### 3/4.7.1.5 MAIN STEAM LINE ISOLATION VALVES

The OPERABILITY of the main steam line isolation valves ensures that no more than one steam generator will blow down in the event of a steam line rupture. This restriction is required to: (1) minimize the positive reactivity effects of the Reactor Coolant System cooldown associated with the blowdown, and (2) limit the pressure rise within containment in the event the steam line rupture occurs within containment. The OPERABILITY of the main steam isolation valves within the closure times of the Surveillance Requirements are consistent with the assumptions used in the safety analyses.

ATTACHMENT TO LICENSE AMENDMENT NO. 43

FACILITY OPERATING LICENSE NO. NPF-86

DOCKET NO. 50-443

Replace the following pages of Appendix A, Technical Specifications, with the attached pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. Overleaf pages have been provided.

<u>Remove</u>	<u>Insert</u>
3/4 7-1*	3/4 7-1*
3/4 7-2	3/4 7-2
B 3/4 7-1	B 3/4 7-1
-	B 3/4 7-1A
B 3/4 7-2*	B 3/4 7-2*



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 43 TO FACILITY OPERATING LICENSE NO. NPF-86

NORTH ATLANTIC ENERGY SERVICE CORPORATION

SEABROOK STATION, UNIT NO. 1

DOCKET NO. 50-443

1.0 INTRODUCTION

By letter dated September 5, 1995, North Atlantic Energy Service Corporation (North Atlantic) proposed an amendment to the Appendix A Technical Specifications (TSs) for the Seabrook Station, Unit 1 (Seabrook). The proposed changes would revise TS Table 3.7-1 by lowering the maximum allowable power range neutron flux high setpoint when one or more main steam safety valves (MSSVs) are inoperable. The proposed changes would also revise the Bases for TS 3/4.7.1.1 to include the algorithm used for determining the new setpoint values. The proposed changes also would revise TS Table 3.7-2 to reduce the lift setpoints of MSSVs.

2.0 EVALUATION

Westinghouse Electric Corporation (Westinghouse) has determined that the maximum allowable power range neutron flux high setpoints given in TS Table 3.7-1 may not be low enough to prevent a secondary side overpressurization during a loss of load/turbine trip. In Nuclear Safety Advisory Letter (NSAL) 94-001 dated January 20, 1994, Westinghouse reported their determination that the maximum allowable initial power level is not a linear function of available MSSV relief capacity. It was further determined that the current TS provisions for reduced reactor power levels with inoperable MSSVs may not preclude the secondary side pressure from exceeding 110% of its design value during a loss of main feedwater transient, particularly at lower power levels. NSAL 94-001 also provided licensees with an algorithm for determining revised neutron flux high setpoints. North Atlantic has developed a revised algorithm similar to the one provided by Westinghouse. The major difference is that North Atlantic's revised algorithm would lead to slightly more accurate calculated nuclear power by excluding heat input from reactor coolant pumps from the thermal power of the nuclear steam supply systems. We find the North Atlantic's revised algorithm conservative and acceptable.

North Atlantic has calculated new neutron flux high setpoint values using its revised algorithm. The new values were lower than the values in the current TS. This process resulted in high neutron flux reactor trip setpoint values

of 66%, 47%, and 28% of rated thermal power for a maximum of one, two, and three inoperable MSSVs, respectively, on any operating steam generator. Current TS values are 87%, 65% and 43% for those same conditions.

The staff has found that North Atlantic's revised algorithm ensures that the maximum power level allowed for operation with inoperable MSSVs is below the heat removing capability of the operable MSSVs. This ensures that the secondary system pressure will not exceed 110% of its design value. In addition, the new setpoints are more conservative than the previous setpoints. Therefore, the staff finds that the proposed changes to TS Table 3.7-1 and Bases 3/4.7.1.1 are acceptable.

North Atlantic also proposed revised TS Table 3.7-2 to reduce lift setpoints of the MSSVs. There are five MSSVs in each of the four main steam lines. North Atlantic proposed setpoints for the four out of five MSSVs in each of the four main steam lines with slightly lower setpoints than that in the current TS and keep the MSSV with the lowest setting at the same setpoint of 1200 psia as that in the current TS. North Atlantic has stated that since the capacities of the MSSVs are not being changed and the modified lifting setpoints will cause the MSSVs to open earlier in a loss of load event, the consequences of a loss of load transient with the new MSSV setpoints is still within the current design basis at Seabrook. For postulated events where excessive reactor coolant system cooldown via MSSV blowdown could be a concern, there is no effect since the lift setpoint of the lowest set MSSV in each loop is not changed. We have reviewed North Atlantic's submittal and agree with its assessment.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Hampshire and Massachusetts State officials were notified of the proposed issuance of the amendment. The State officials had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (60 FR 51505). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

## 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Chu-Yu Liang

Date: November 2, 1995